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Summary Record of 22nd Meeting of the Committee, Paris, 1st - 5th October 1979.

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Informal Memorandum on fast reactor short-term fission-product decay heat and its uncertainty. R.W. Peelle, J.K. Dickens

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a. "Reactor Physics Activities in Canada"; by M.F. Duret, May 1974
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Available Reactor Physics Data from Power Reactors - Material Utilized by the Reactor Physics Section at Risø (RP-2-74), compiled by T. Petersen, May 1974

"Thermal Benchmark Experiment Compilation"; compiled by G. Casini, May 1974


Proceedings of the NEACRP Meeting of a Monte Carlo Study Group at Argonne National Laboratory on 1st-3rd July 1974" (ANL-75-2)

Summary Record of the Seventeenth Meeting of the NEACRP (Technical Sessions), Cadarache, France; 4th-7th June 1974"; compiled by E. Hellstrand

Australia: "Reactor Physics Activities in Australia", May 1974-May 1975; D.B. McCulloch

Belgium: "Reactor Physics Activities in Belgium"; compiled by J. Debrue

Canada: "Reactor Physics Activities in Canada"; M.F. Duret

Commission of the European Communities: "Reactor Physics Activities at JRC-Euratom/Ispra"; June 1974-May 1975, G. Casini

France: "Reactor Physics Activities in France"; June 1974-May 1975, J.Y. Barré

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Italy: "Summary of Reactor Physics Activities in Italy in the Period July 1974-June 1975"; edited by U. Farinelli

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Norway: "Reactor Physics Activities in Norway"; June 1974-May 1975; J.O. Berg and S. Børresen

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Switzerland: "Reactor Physics Activities in Switzerland"; June 1974-May 1975; R. Richmond
n. United Kingdom: "Reactor Physics in the United Kingdom"; C.G. Campbell and P.J. Fayers
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NEACRP-L-121 Butler, J. and McCracken, A.K.: "The NEA/Euratom Restricted Meeting of Specialists on Shielding Benchmark Experiments held at AEE, Winfrith"; April 1975; Summary Report prepared for the NEA-CRP Meeting to be held at Bologna, 9th-13th June 1975

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Neutronics of Laser Fission-Fusion Systems. Spain

A Fast Reactor Benchmark Problem in Two and Three Space Dimensions. G. Buckel, R. Froehlich, K. Kufner, B. Stehle, Germany

Activities concerning the TNR Fast Reactor Benchmark Problem. G. Buckel, K. Kufner, Germany


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Homogenisation Problems in Thermal and Fast Reactors. H. Küsters, Germany

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Results and Main Conclusions of the IAEA-Advisory Group Meeting on Transactinium Nuclear Data, held at Karlsruhe, November 1975. H. Küsters, M. Lalovic, Germany

Specialist Meeting on inelastic scattering and fission neutron spectra. Harwell 1975. B.H. Armitage, Harwell

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Long term uranium demand and supply in Canada
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Neutronic characteristics of a 1200 MWe fuelled with (ThO₂ - 233 U)O₂

Preliminary study of the sub-assembly reshuffling in fast breeders. JC Cabrillat

KEMA : Calculations for compact storage racks for PWR fuel. W.J. Oosterkamp

An appraisel of the NEA Collaborative programme of uncertainty analysis and shielding benchmark experiments October 1979. J. Butler and H. Rief

Technical Sessions of the 22nd Meeting of the Committee, 1st - 5th October 1979, Paris.
Fuel Pin and Subassembly Heterogeneity Effect on Neutronics Properties of a Fast Power Reactor

Validation of KENO-II ANISN and Hansen-Roach Cross-Section Set on Plutonium Solution Systems

Calculational Investigations on Designing Methods of Fuel Thicknesses of Annular Tanks for Plutonium Solutions

Summary Record (Technical Sessions) of 23rd Meeting of the Committee. Argonne Nat. Lab. Idaho Falls, 22/26 Sept. 80
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The Danish Solution of the NRF Benchmark Problem 1980. Control Rod Ejection in a PWR

An International Intercomparison of Results for the Reactivity Effect of Steam Ingress into the Core of a GCFR

Decay Heat Calculations based on theoretical estimation of Average beta and gamma energies released from short-lived

Non-destructive testing of burn-ups and cooling times on PWR spent fuels

Reactor Physics Activity Reports of Member Countries 1980-1981

Summary Record of the 24th Meeting of NEACRP - 1981 (Technical Sessions)

Summary SMORN III Benchmark Test on Reactor Noise Analysis Methods - Hirosha and Shinchara (29.3.82)

Reactor Physics Activities in NEA Member Countries Oct. 81 - Sept. 82 (cover ordered at OECD on 2nd August)

Adjustment of neutron multigroup cross sections with error covariance matrices to deep penetration integral experiments - G. Hehn et al.

A study on the potential safety advantage of large heterogeneous LMFBRs. - K. Suzuki et al.

ENDF/B5 Cross Section measurement standards. A.D. Carlson and M.R. Bhat

Summary Record (Technical Sessions) of 25th Meeting of NEACRP 13th-17th September 1982, Karlsruhe, F.R. Germany.