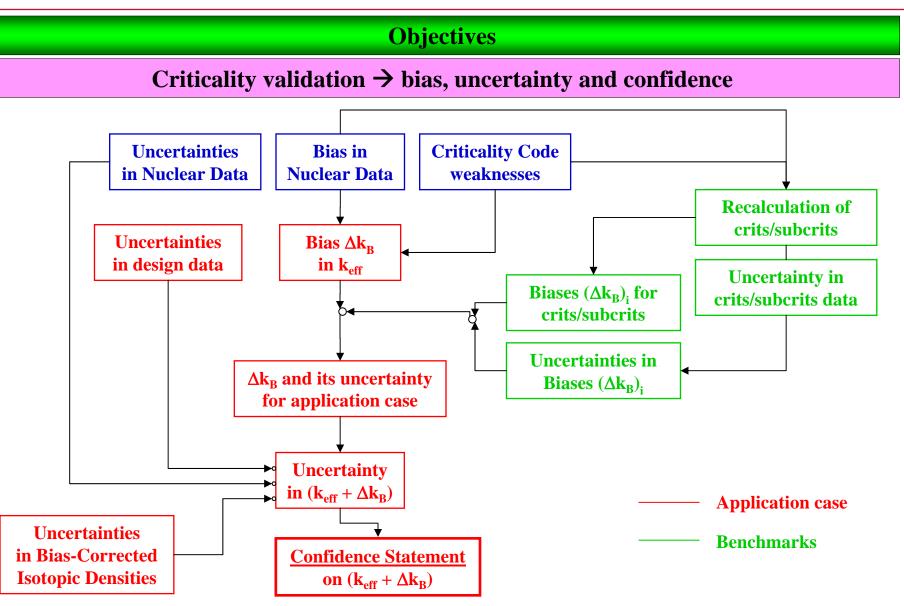
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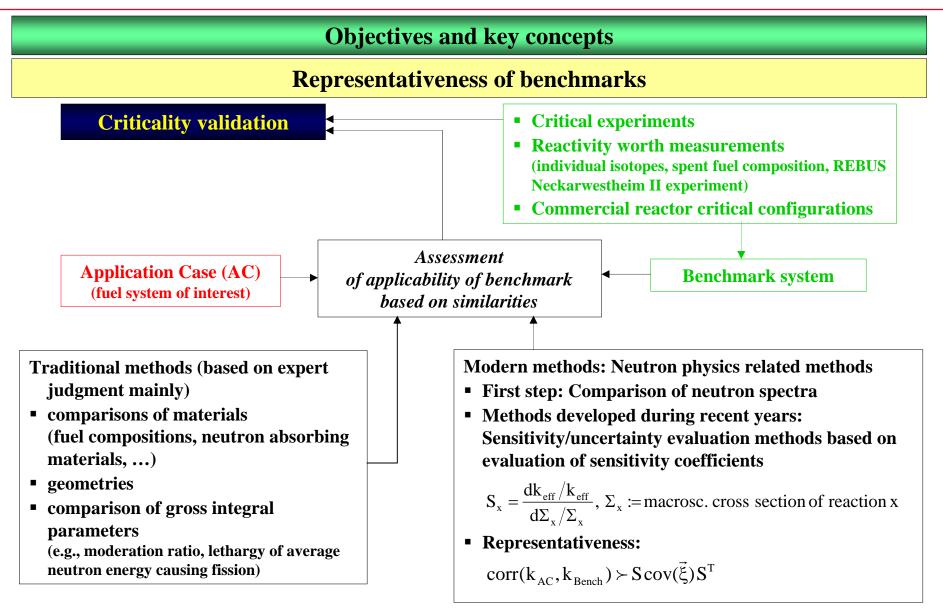
Sensitivity/Uncertainty Analyses

- Chair Overview: Objectives, key concepts, and methods -

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Key concepts and methods

Traditional concepts and methods of bias and uncertainty estimation

Test on consistency of k_{eff} values of the benchmark systems analyzed:

- is a statistical test (χ^2 -test) on equal expectation $E[k_{eff}]$ by using the weighted mean of all benchmark k_{eff} outcomes (special case of the weighted linear least squares method)
- statistical elimination of outliers (← physics arguments?) -

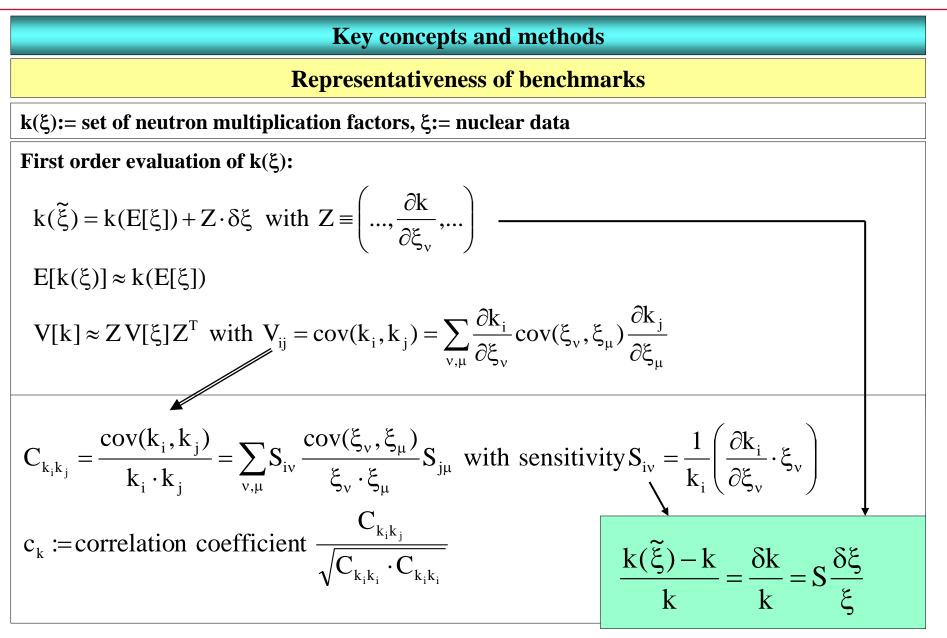
Trending analysis of the k_{eff} **values of the benchmark systems analyzed:**

- k_{eff} as a function of traditional parameters
- Requires goodness-of-fit test

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• Outliers elimination (← physics arguments?) —

→ Neutron-physics-based benchmark selection arguments **\$**



Key concepts and methods

Experimental information (m) \rightarrow **Increase of information on the nuclear data**

Bayes' theorem

 $p(\xi \mid m) \succ L(m \mid \xi) \cdot \pi(\xi)$

plus nuclear data adjustment based on the maximum likelihood (ML) procedure.

Due to the linearity assumption (\leftarrow first order evaluation) ($\delta k/k$) = S ($\delta \xi/\xi$) \rightarrow ML estimator provides best estimates. For normal distributions $\pi(\xi)$ and L(m| ξ) one gets:

$$\frac{d\xi}{\xi} = \widetilde{\mathbf{V}}_{\xi} \mathbf{S}^{\mathrm{T}} \left(\mathbf{C}_{kk} + \mathbf{C}_{mm} \right)^{-1} \frac{\delta k}{k} \text{ with } \frac{\delta k}{k} = \frac{k - m}{k}$$
$$\widetilde{\mathbf{V}}_{\xi}^{\text{posterior}} = \widetilde{\mathbf{V}}_{\xi} - \widetilde{\mathbf{V}}_{\xi} \mathbf{S}^{\mathrm{T}} \left(\mathbf{C}_{kk} + \mathbf{C}_{mm} \right)^{-1} \mathbf{S} \widetilde{\mathbf{V}}_{\xi}$$

Computational bias of application case:

$$\frac{\delta k_{A}}{k_{A}} = S_{A} \frac{\delta \xi}{\xi} , \quad cov \left(\frac{\delta k_{A}}{k_{A}}\right) = S_{A} \widetilde{V}_{\xi}^{posterior} S$$

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Key concepts and methods

Experimental information \rightarrow **Increase of information on the nuclear data**

Generalized Linear Least Squares minimizes (GLLSM) quadratic form

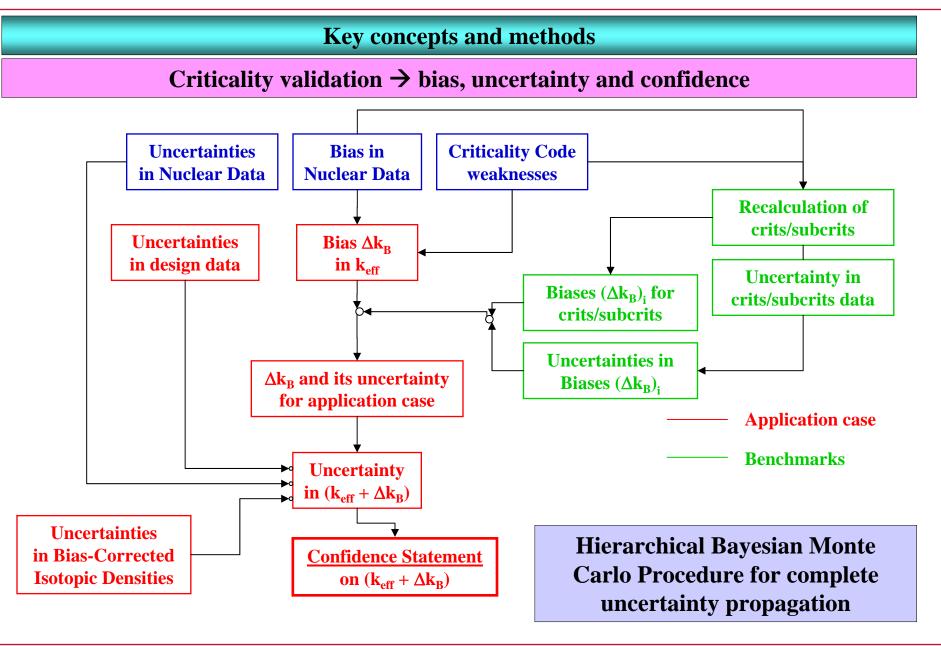
$$Q^{2}(\psi,\zeta) = (\psi,\zeta)^{T} \begin{pmatrix} C_{mm} & 0 \\ 0 & \tilde{V}_{\xi} \end{pmatrix}^{-1} (\psi,\zeta) \text{ with } \psi = \frac{k(\xi^{adj}) - m}{k(\xi)} \text{ and } \zeta = \frac{d\xi}{\xi}$$

$$\zeta = \tilde{V}_{\xi} S^{T} (C_{kk} + C_{mm})^{-1} \frac{\delta k}{k}, \quad \psi = C_{mm} (C_{kk} + C_{mm})^{-1} \frac{\delta k}{k}$$

$$C_{\zeta\zeta} = \tilde{V}_{\xi} S^{T} (C_{kk} + C_{mm})^{-1} S \tilde{V}_{\xi}, \quad C_{\psi\psi} = C_{mm} (C_{kk} + C_{mm})^{-1} C_{mm}$$

$$\tilde{V}_{\xi}^{\text{posterior}} = \tilde{V}_{\xi} - C_{\zeta\zeta}$$
Computational bias of application case: $\frac{\delta k_{A}}{k_{A}} = S_{A} \frac{\delta \xi}{\xi}, \quad \text{cov}\left(\frac{\delta k_{A}}{k_{A}}\right) = S_{A} \tilde{V}_{\xi}^{\text{posterior}} S_{A}^{T}$

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Some references

N.L. Broadhead, B.T. Rearden, and C.M. Hopper, J.J. Wagschal and C.V. Parks, *"Sensitivity- and Uncertainty-Based Criticality Safety Validation Techniques"*, Nuclear Science and Enginieering: 146, 340-366 (2004).

Christophe Vanard, Alain Santamarina, Alexandre Leclainche, Claude Mounier, *"The R.I.B. Tool for the Determination of Computatiional Bias and Associated Uncertainty in the CRISTAL Criticality-Safety Package"*,
2009 ANS Nuclear Criticality Safety Division Topical Meeting, NCSD 2009, September 13-17, 2009, Richland, Washington, USA.

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"MOCADATA Monte Carlo Aided Design and Tolerance Analysis: General hierarchical Bayesian procedure for calculating the bias and the a posteriori uncertainty of neutron multiplication factors including usage of TSUNAMI in a hierarchical Bayesian procedure for calculating the bias and the a posteriori uncertainty of k_{eff} ", International Workshop on Advances in Applications of Burnup Credit for Spent Fuel Storage, Transport, Reprocessing, and Disposition, organized by the Nuclear Safety Council of Spain (CSN) in cooperation with the International Atomic Energy Agency (IAEA), Córdoba, Spain, 27-30 October, 2009