

Status of TENDL

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Contents

- TENDL-2017
- Examples: Differential data
- Examples: Integral data
- Future of TALYS and TENDL

TENDL: Complete nuclear data library for innovative nuclear science and technology

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³*NRG, Westerduinweg 3, 1755 LE Petten, Netherlands*

⁴*Nuclear Energy Agency, OECD, 92100 Boulogne-Billancourt, France*

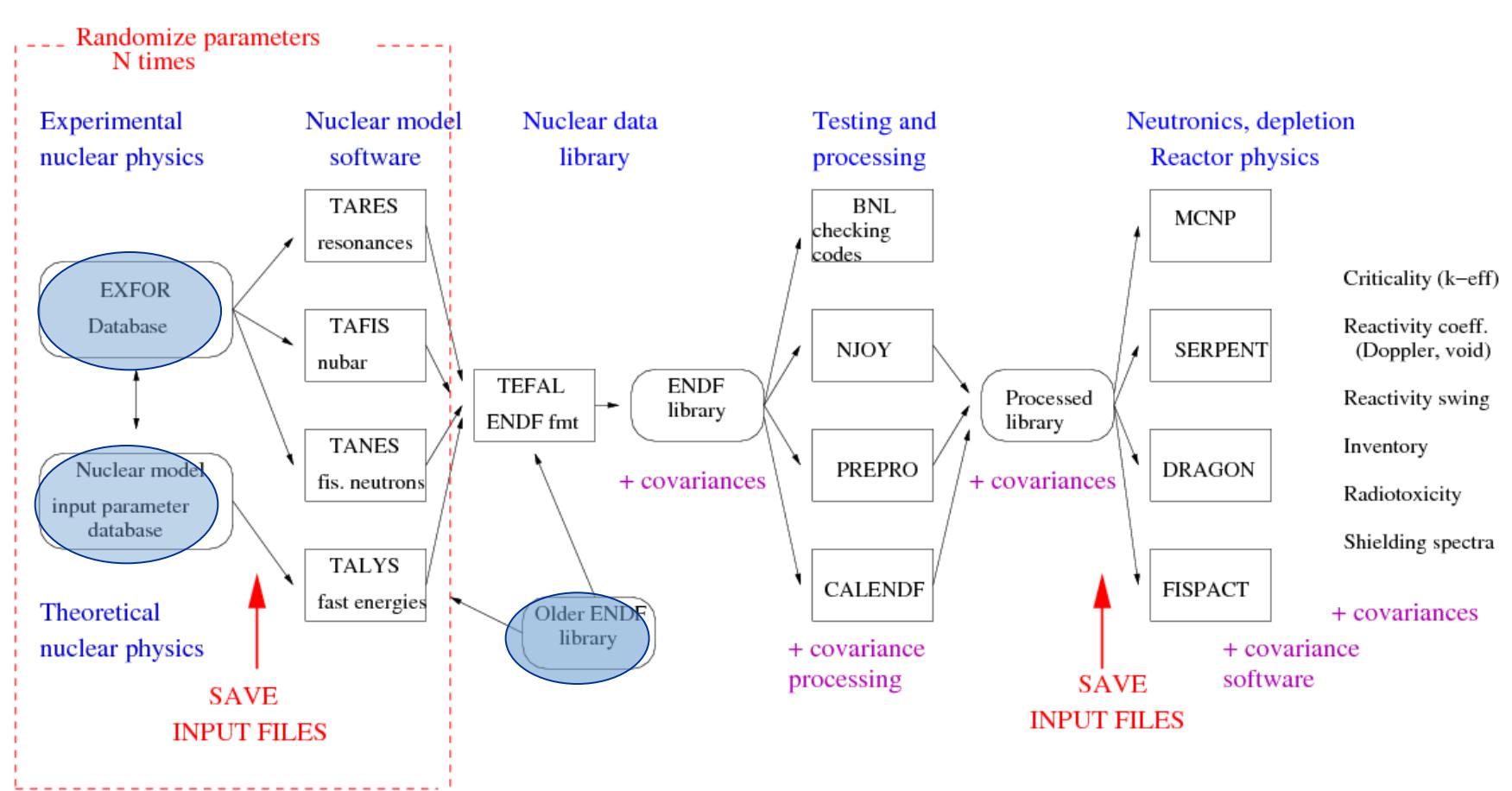
⁵*United Kingdom Atomic Energy Authority, Culham Science Centre, Abingdon OX14 3DB, United Kingdom*

(Dated: May 11, 2018; Received xx July 2018; revised received xx September 2018; accepted xx October 2018)

- In preparation for Nuclear Data Sheets,
January 2019

TENDL nuclear data library

Loop over nuclides : TENDL

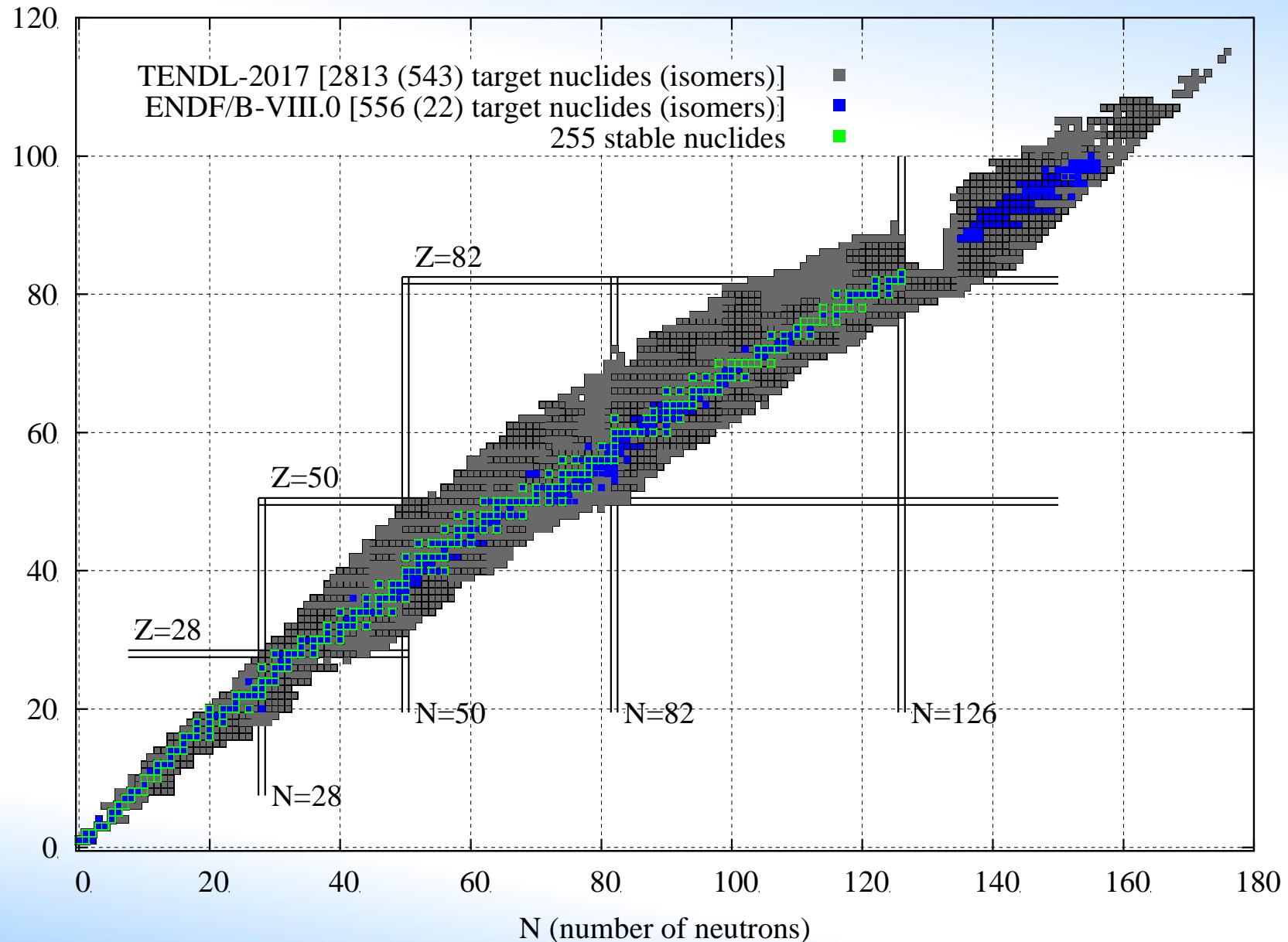


A.J. Koning and D. Rochman , "Modern nuclear data evaluation with the TALYS code system", Nuclear Data Sheets 113, 2841 (2012).

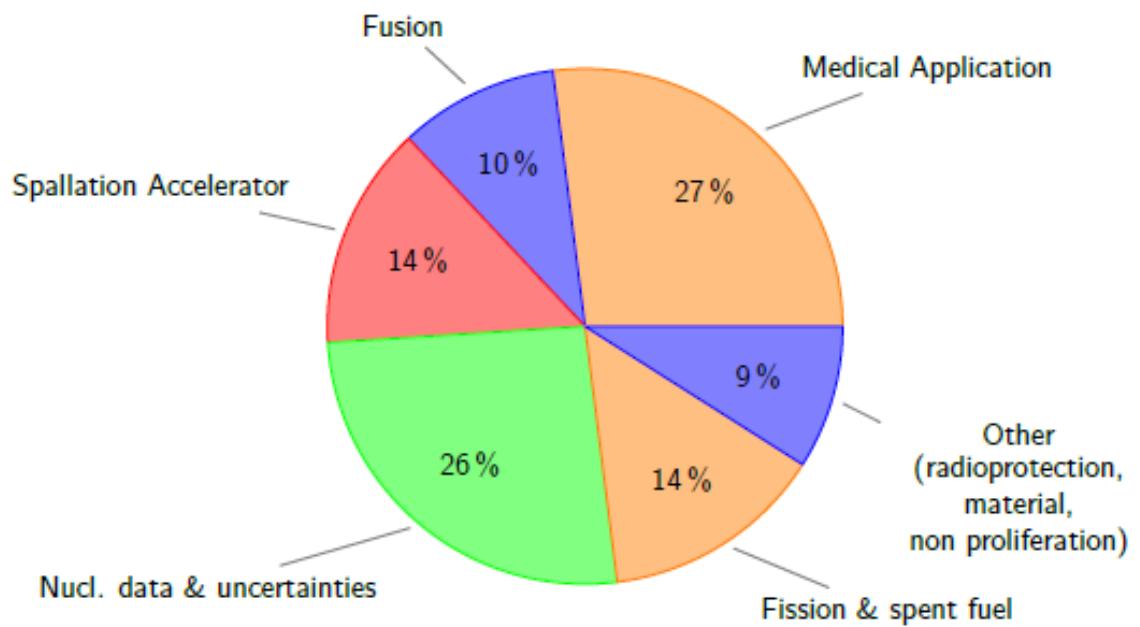
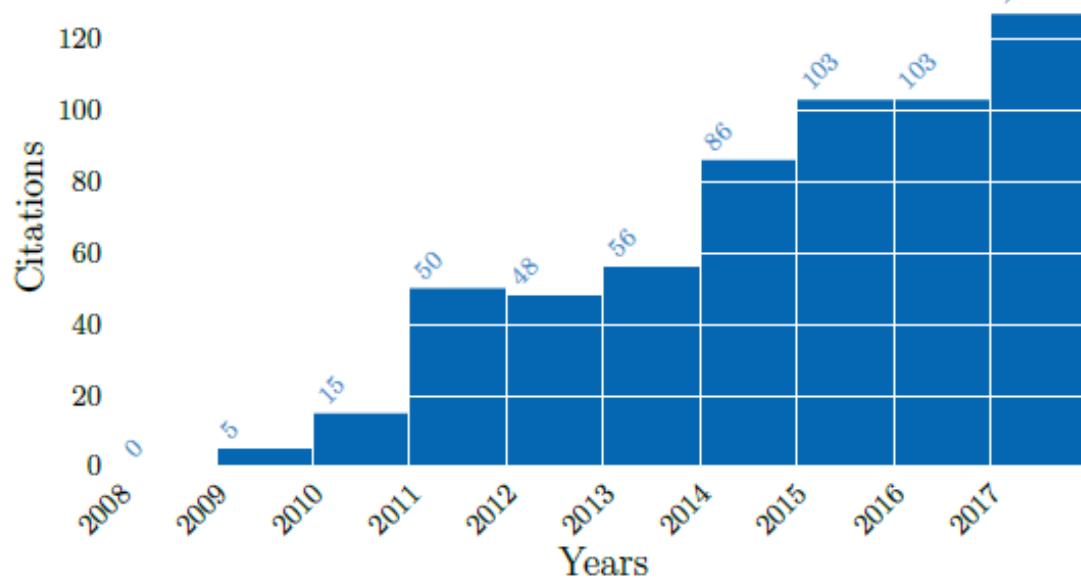
Nuclear landscape: Isotopic targets



IAEA Atoms for Peace and Development



TENDL citations



TENDL-2015 additions to other libraries

- ENDF/B-VIII: 28 nuclides
- JEFF-3.3:
 - >150 nuclides for incident neutrons
 - Photonuclear
 - Charged-particles

TENDL-2017 Home

- https://tendl.web.psi.ch/tendl_2017/tendl2017.html

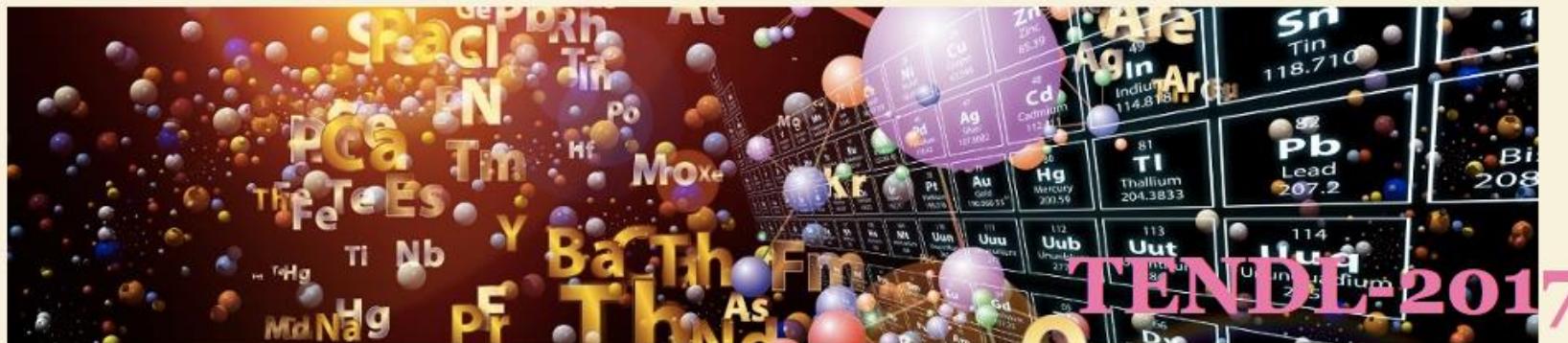


TALYS-based evaluated nuclear data library

Home

Reference & us

Citations



“ We believe that our great goal can be achieved with systematism and reproducibility. We are so outside the box, that the box is a point ”

How to reference

Sub-library files

1. neutron
2. Proton
3. Deuteron
4. Triton
5. He3
6. Alpha
7. Gamma

tar & Ace files

Random files

1. Random fission yields

TENDL-2017: (release date: December 30, 2017)

Last update: 29 December 2017

TENDL is a nuclear data library which provides the output of the TALYS nuclear model code system for direct use in both basic physics and applications. The 9th version is TENDL-2017, which is based on both default and adjusted TALYS calculations and data from other sources (previous releases can be found here: [2008](#), [2009](#), [2010](#), [2011](#), [2012](#), [2013](#), [2014](#), and [2015](#)).

Up to 2014, TENDL was produced at NRG Petten. Since 2015, TENDL is mainly developed at PSI and the IAEA (Nuclear Data Section). Still, many people contribute to TENDL with the testing and processing of the files.

TENDL contains evaluations for seven types of incident particles, for all isotopes living longer than 1 second (about 2800 isotopes), up to 200 MeV, with covariances.

TENDL is **not** a default or shadow library. Not a single neutron evaluation is based on

TENDL-2017 forms, ENDF-6 and other containers

Tabulated data (fast neutron range)

1. [Tabular elastic angular distributions](#) (En - angle - cross section)
2. [Tabular \(n,inl_1\) angular distributions](#) (En - angle - cross section)
3. [Tabular gamma-ray intensities](#) (En - Eg - cross section)
4. [Tabular residual cross sections](#) (En - Residual product - cross section)
5. [Tabular spectra](#) (En - Eout - particle - cross section)
6. [Tabular total and partial cross sections](#) (En - cross section)

Evaluated formatted data (i.e. ENDF)

1. [The TENDL file](#)
2. Pointwise cross sections at 293 K ([pendf](#))
3. ACE file at 293 K ([ace.gz](#) and [xsdir](#))
4. Special [ENDF](#) file with MF32c and MF12/MT102 (so-called s20 file)
5. [EAF file](#) (European Activation File) and associated [covariances](#)
6. [ACF file](#) (Activation File)

Processed plots

1. Plots of the cross sections and other quantities: [plot](#)
2. Plots from PREPRO: [plot](#)
3. Comparison TENDL and ENDF/B-VIII.0: [plot](#)
4. Comparison TENDL and JEFF-3.3: [plot](#)
5. Comparison TENDL and JENDL-4.0: [plot](#)
6. Comparison TENDL and IRDFF-1.0: [plot](#)
7. Comparison TENDL and CENDL-3.1: [plot](#)
8. Processed cross sections and covariances with NJOY in 187 groups: [matrix](#) and [plot](#)
9. Processed cross sections and covariances with NJOY in 44 groups: [matrix](#) and [plot](#)
10. Processed cross sections and covariances with NJOY in 33 groups: [matrix](#) and [plot](#)
11. Processed angular distribution and covariances with NJOY in 187 groups: [matrix](#)
12. Processed angular distribution and covariances with NJOY in 187 groups: [plots](#)
13. Processed angular distribution and covariances with NJOY in 33 groups: [matrix](#)
14. Processed angular distribution and covariances with NJOY in 33 groups: [plots](#)

TENDL-2017 forms, ENDF-6 and other containers

- Mirror: <https://www-nds.iaea.org/public/download-endf/TENDL-2017/Original/TENDL-2017.htm>

tar files

Last update: 26 April 2018

The TENDL-2017 library can be retrieved with tar (*.tgz) files for each sub-library. To untar the files, use the command: tar -zxvf.

1. Neutron

2813 ENDF [files](#) (3.1 Gb),

List of MAT numbers: [iso-mat.tendl-n.txt](#).

List of 556 ZAID numbers for ACE: [Ace-Readme.tendl17c.txt](#).

556 ACE files [files](#) (2.1 Gb),

2. Proton

2804 ENDF [files](#) (2.0 Gb)

283 stable targets ACE [files](#) (160 Mb),
and special [files](#) "so" (1.1 Gb).

3. Deuteron

2811 ENDF [files](#) (2.5 Gb)

The ACE forms also serve:
SERPENT and
OpenMc

+soon the
TRIPOLI forms

Current TENDL: 2017

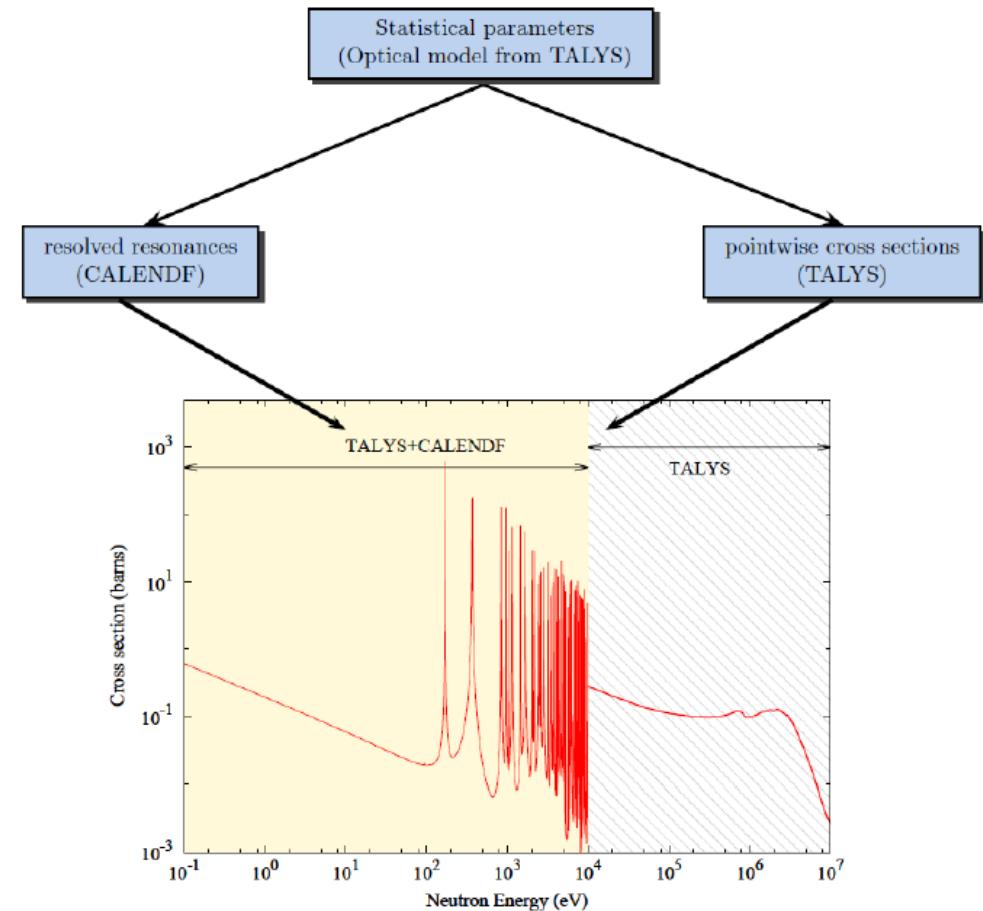
- December 2017: Release of TENDL-2017
 - Neutrons:
 - Improvement of cross section values by Natalia Dzysiuk for fusion and fission product nuclides
 - General improvement of resonance parameters by Dimitri Rochman
 - Last (?) remaining ENDF format deficiencies removed
 - No more MF32, all covariance data pointwise in MF33 down to 1e-5 eV.
 - Photons:
 - Adjustment of TALYS parameters to experimental data
 - Protons, deuterons, tritons, He-3, alpha particles:
 - ENDF format completion for recoils

TENDL in the resonance range

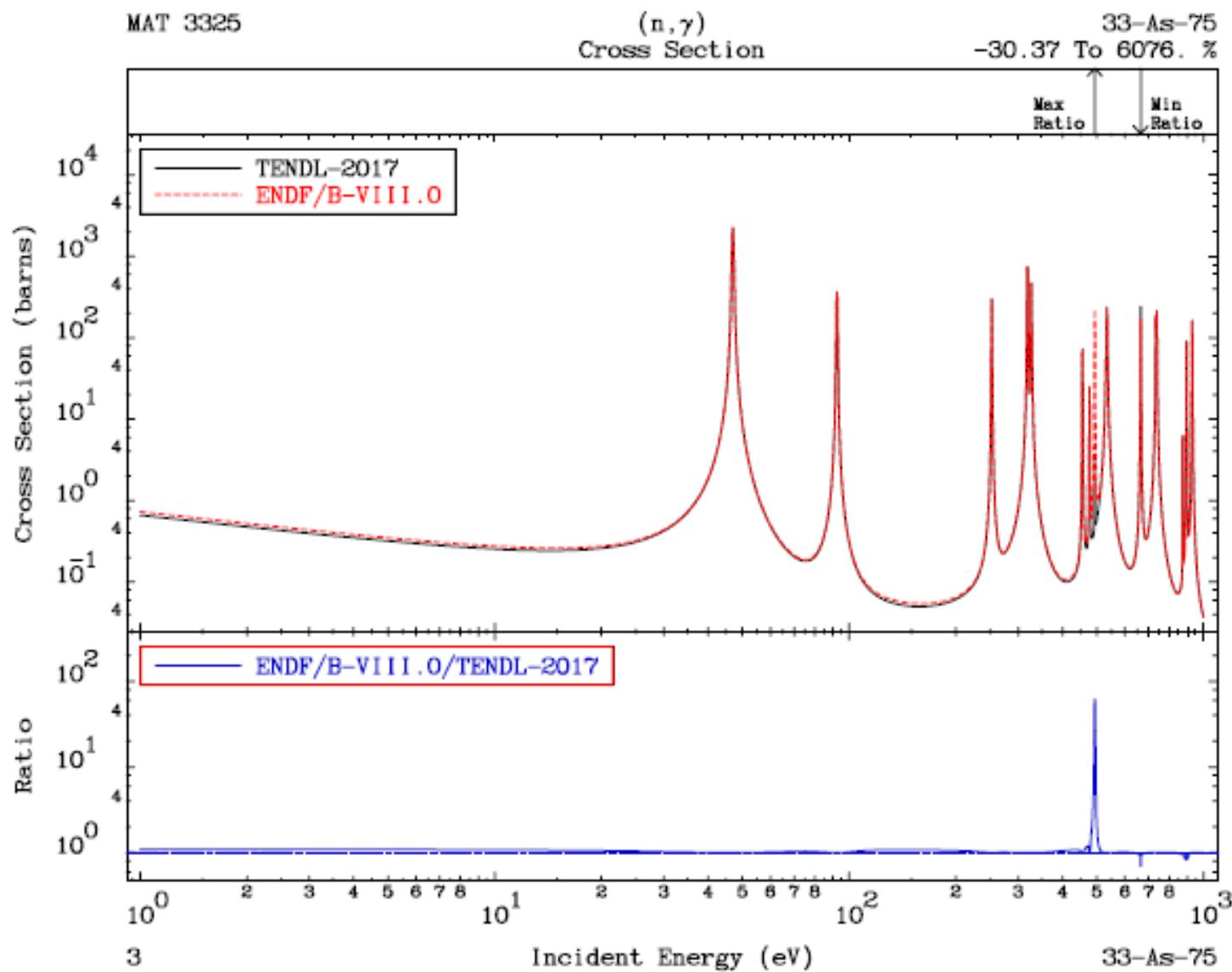
ment

- In TENDL, all 2800 isotopes have unique resonances
 - Only about **10 %** of the resonances are measured,
 - The rest comes from the HFR method (statistical resonances),

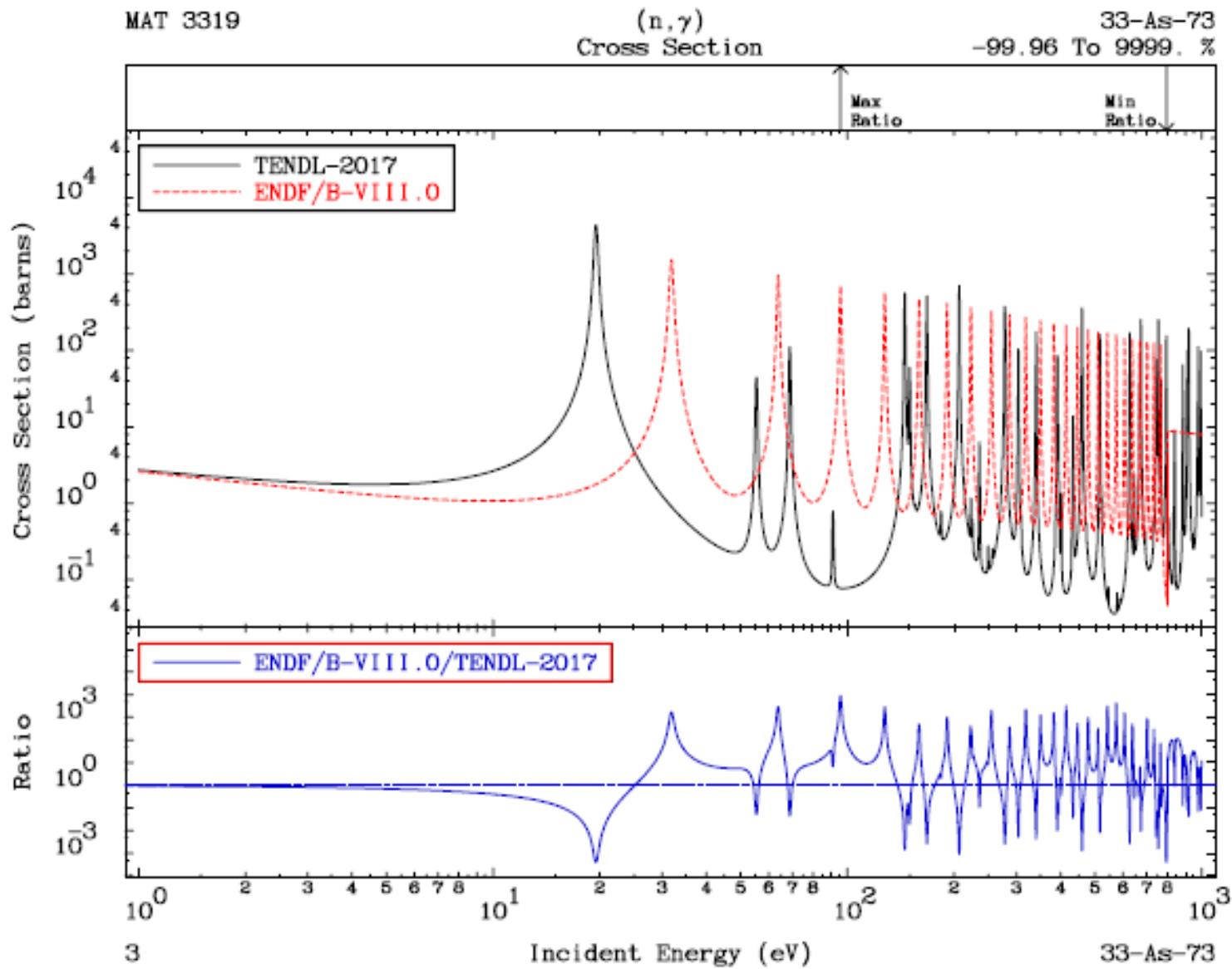
- Presented in ANE 50 (2013) 60
 - Combine the 3 previous models (ld, omp and γ -str) to produce statistical resonances
- Uses the following scheme:
 - TALYS (input: ld + omp + γ -str)
 - CALENDF (input: TALYS output)
 - Output: statistical resonances



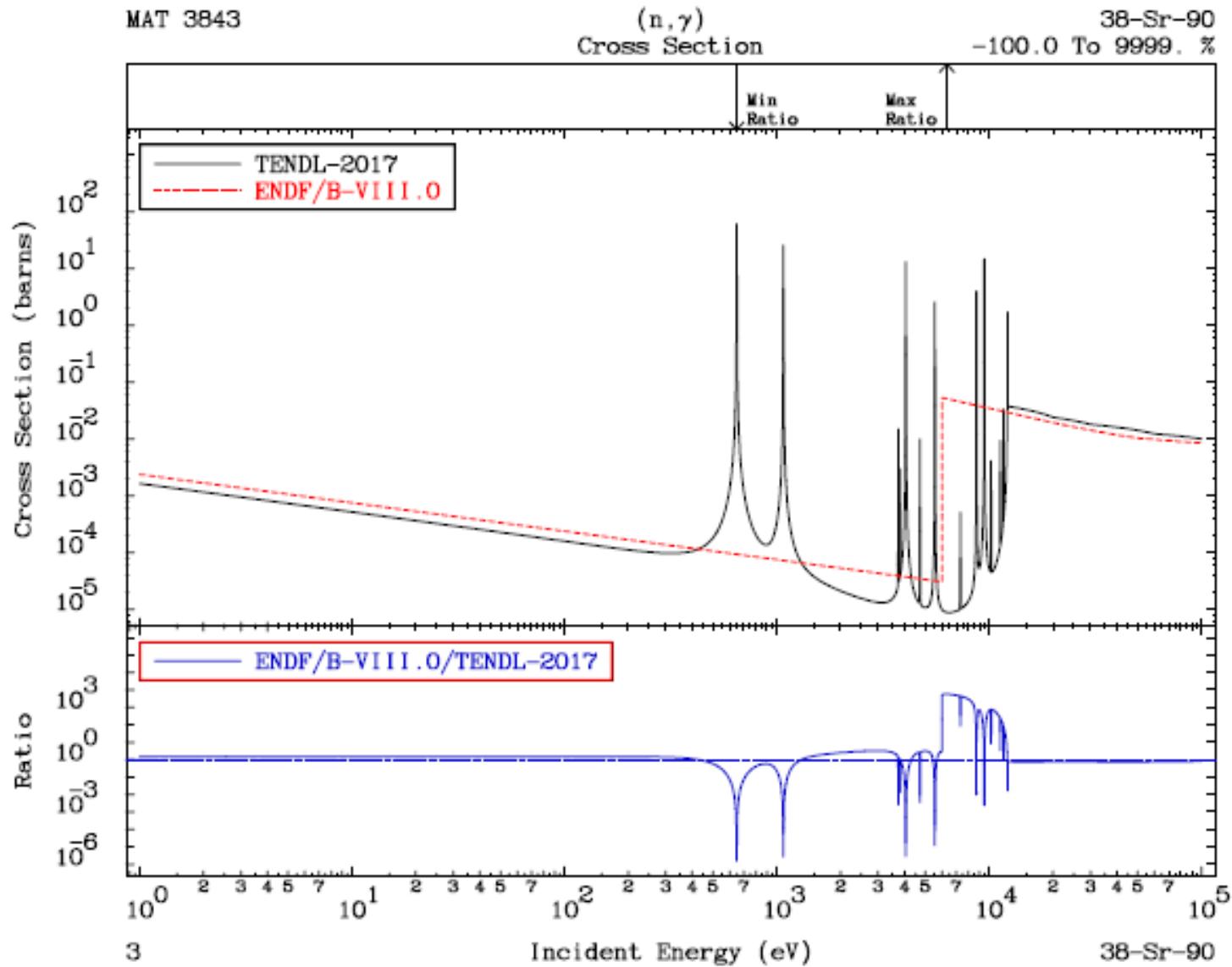
Comparison with ENDF/B-VIII: As-75



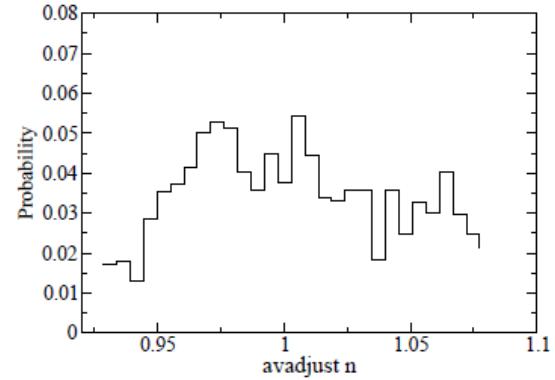
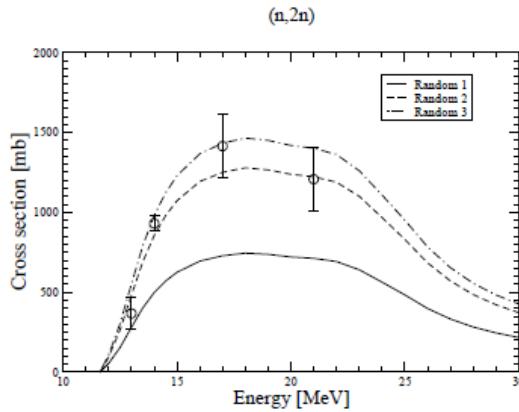
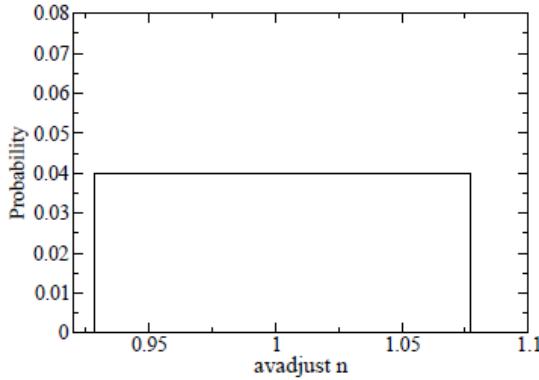
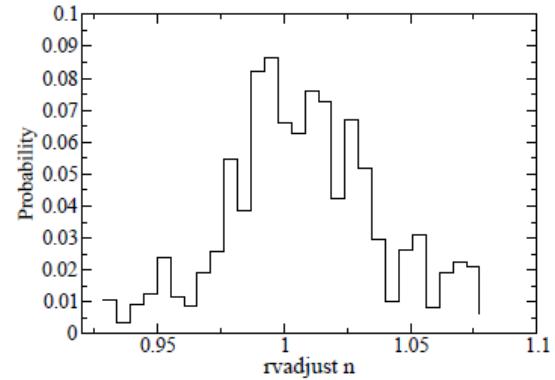
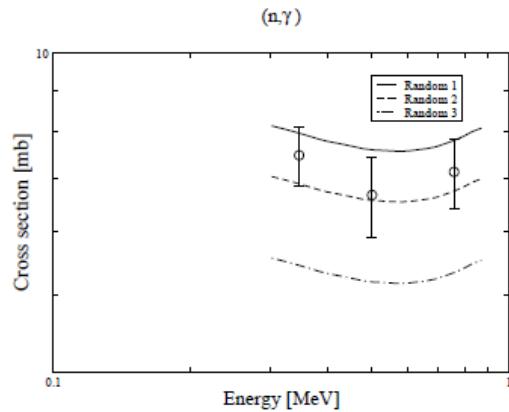
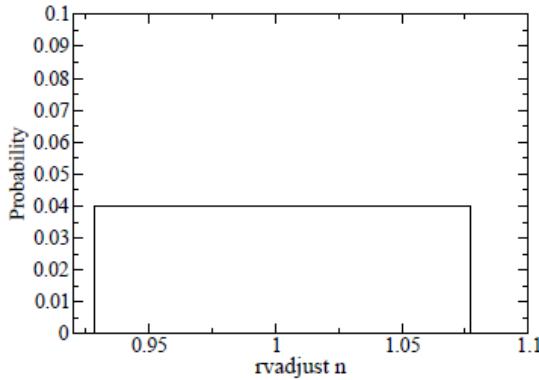
Comparison with ENDF/B-VIII: As-73



Comparison with ENDF/B-VIII: Sr-90



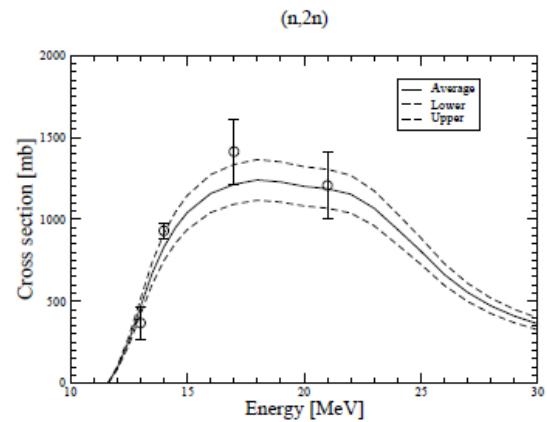
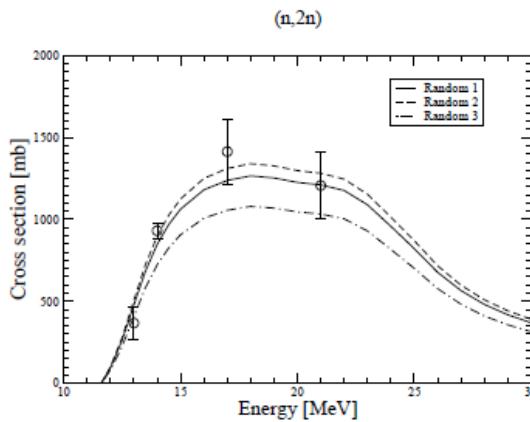
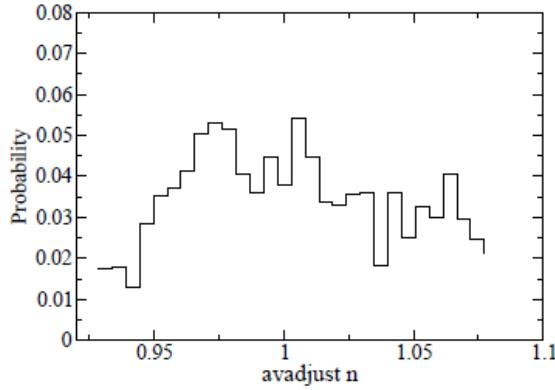
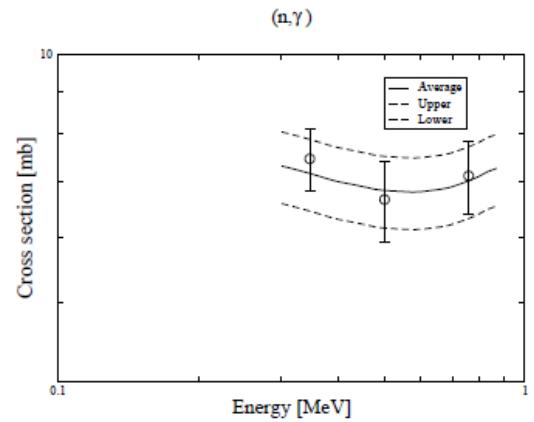
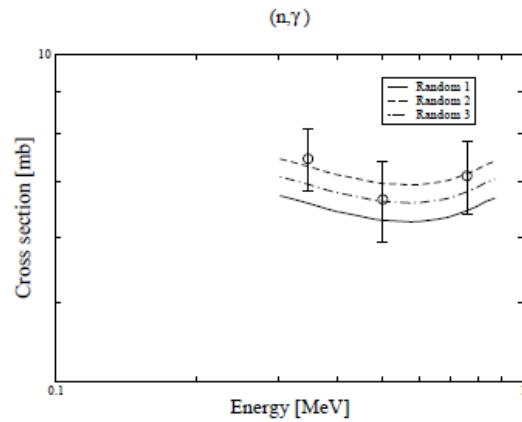
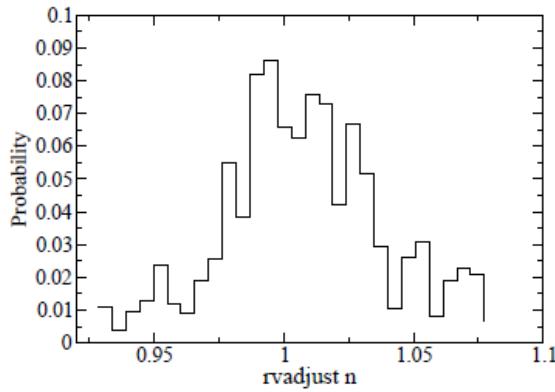
Bayesian Monte Carlo with TALYS: model parameter distribution (step 1)



$$L(\mathbf{p}; \mathbf{x}) = \exp(-\chi^2/2),$$

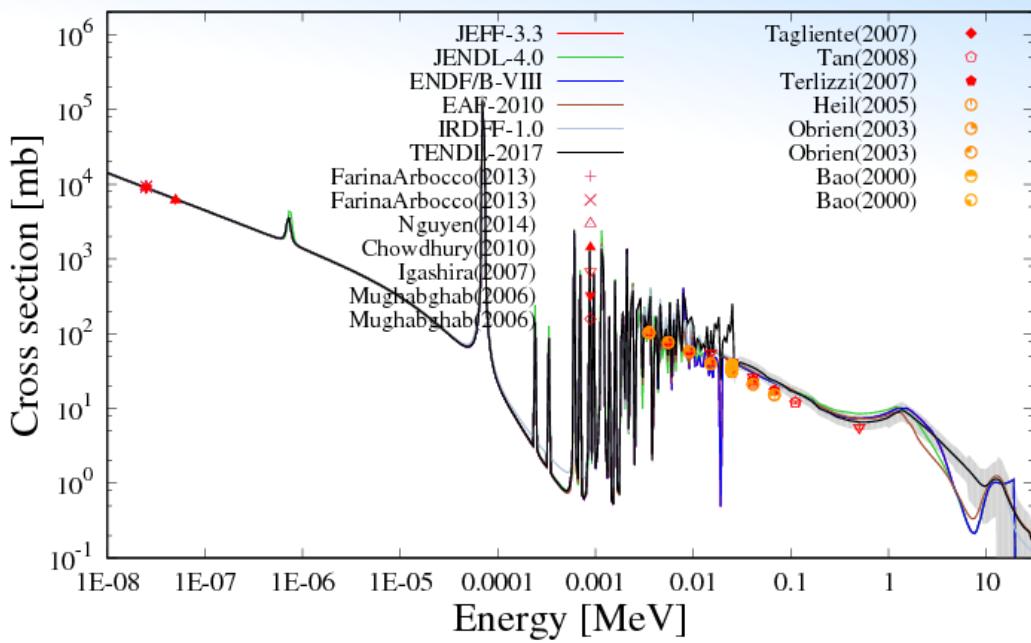


Bayesian Monte Carlo with TALYS: cross section distribution (step 2)

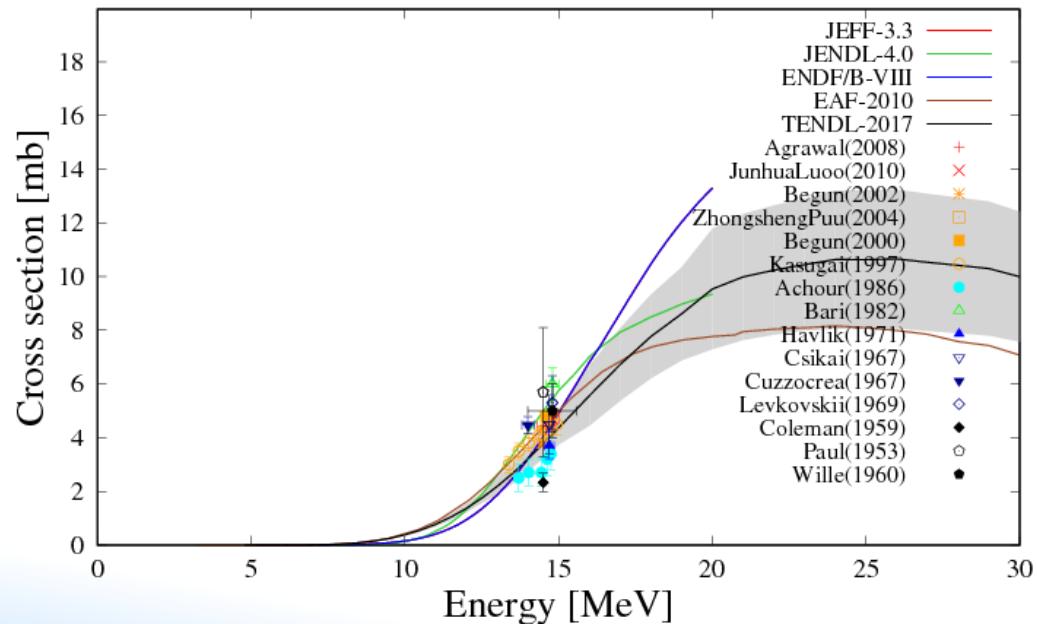


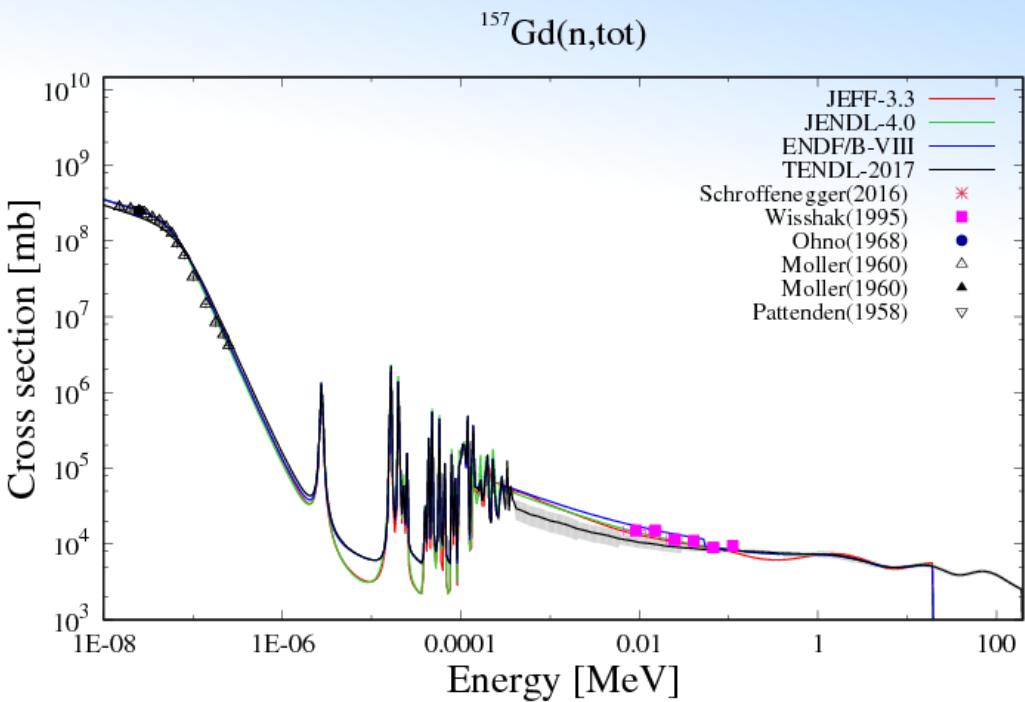
$$L(\mathbf{p}; \mathbf{x}) = \exp(-\chi^2/2),$$



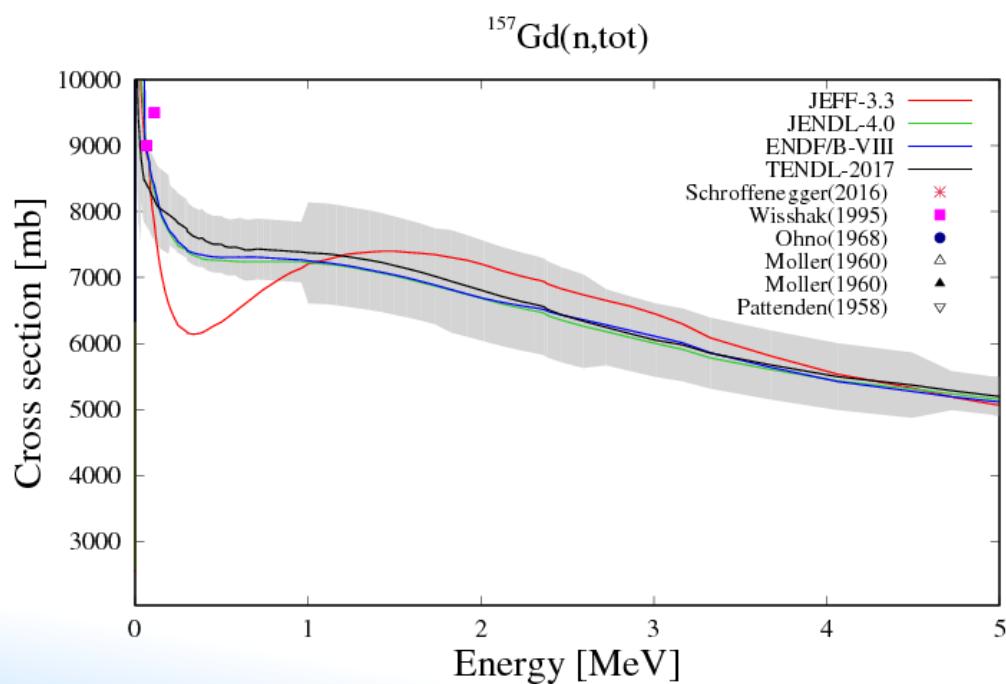
$^{139}\text{La}(\text{n},\gamma)^{140}\text{La}$


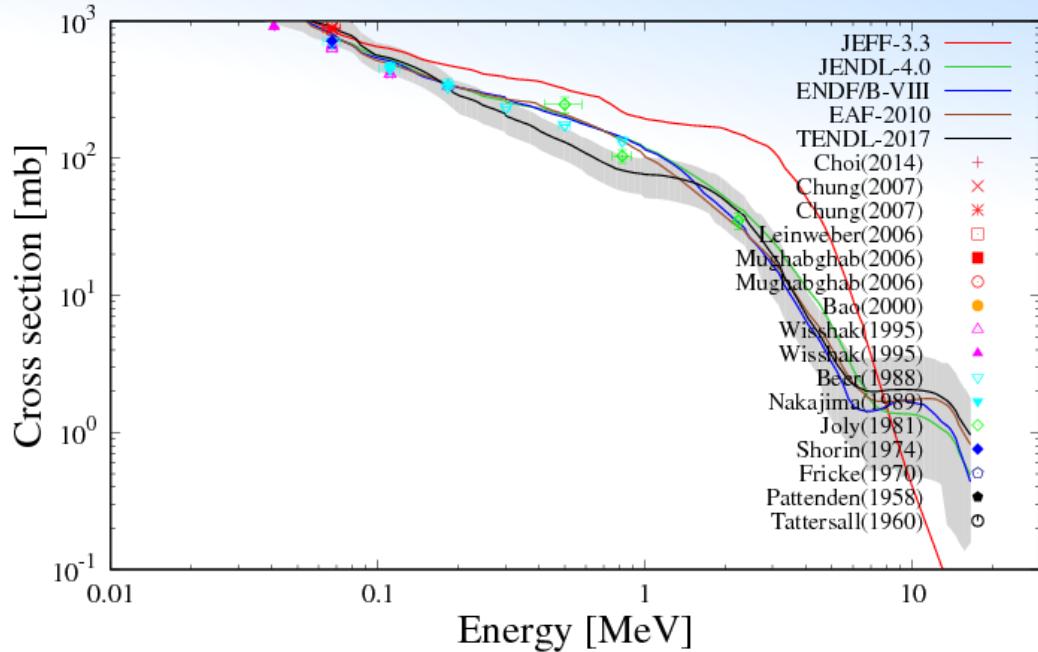
Plots for **all** particles, target nuclides, reaction channels and nuclear data libraries available at <ftp://ftp.nrg.eu/pub/www/talys/plots>

 $^{139}\text{La}(\text{n},\text{p})^{139}\text{Ba}$


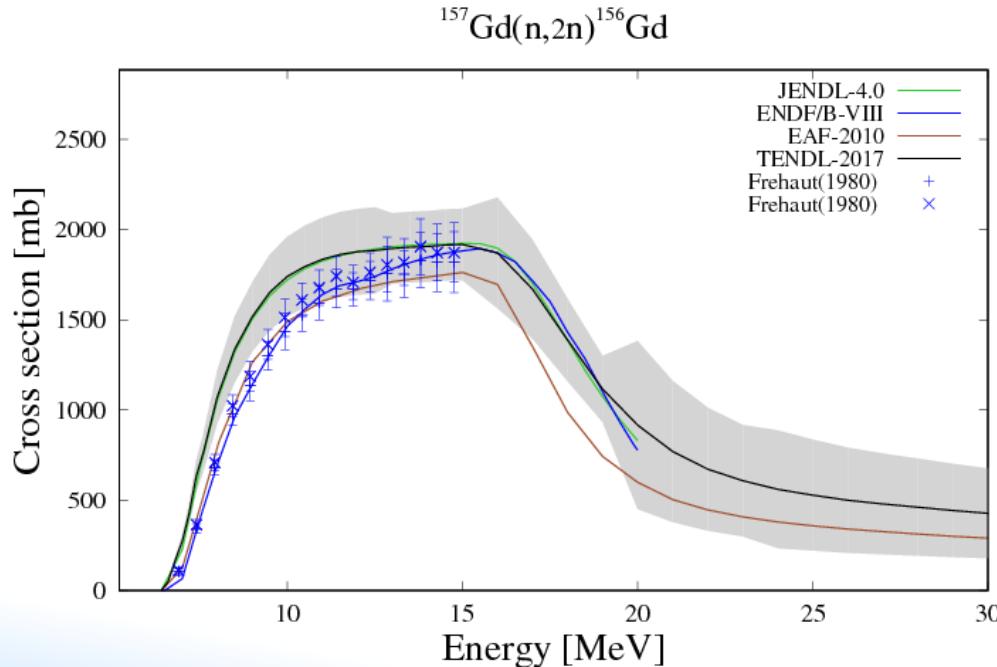


Plots for **all** particles, target nuclides, reaction channels and nuclear data libraries available at <ftp://ftp.nrg.eu/pub/www/talys/plots>



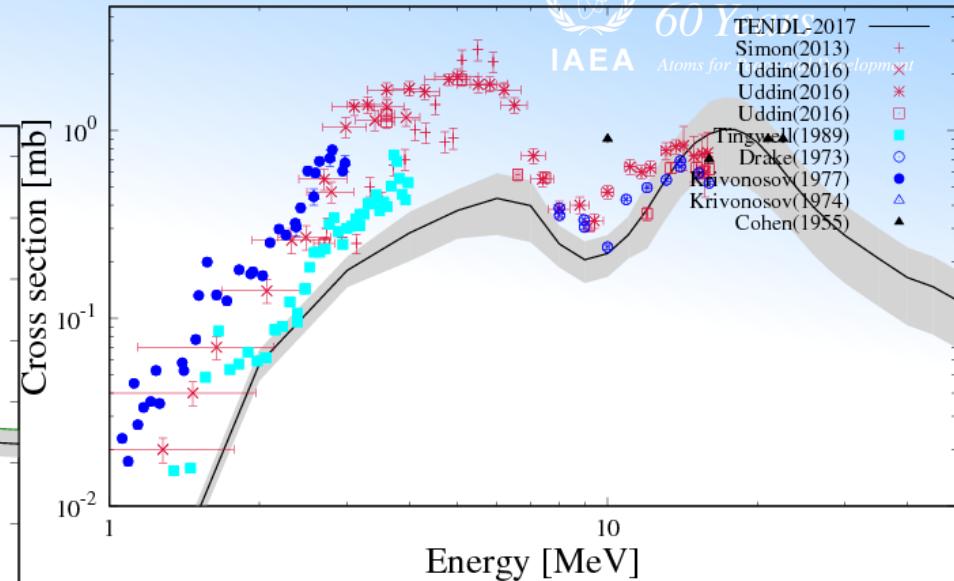
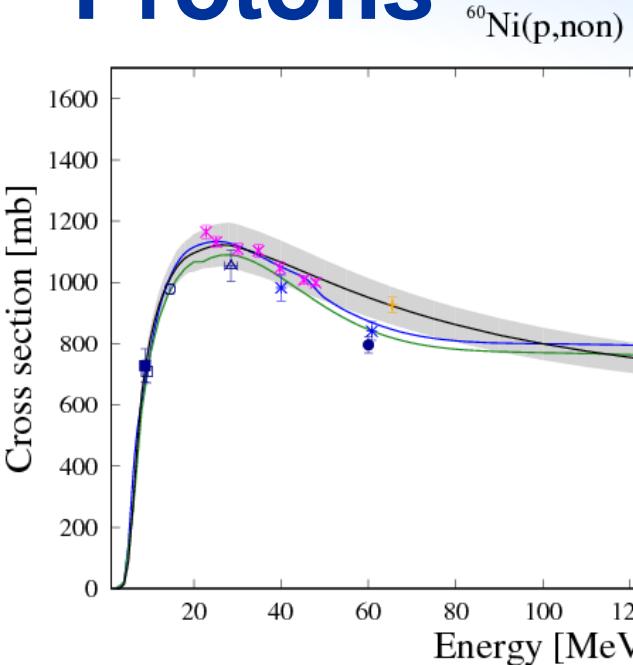


Plots for **all** particles, target nuclides, reaction channels and nuclear data libraries available at <ftp://ftp.nrg.eu/pub/www/talys/plots>

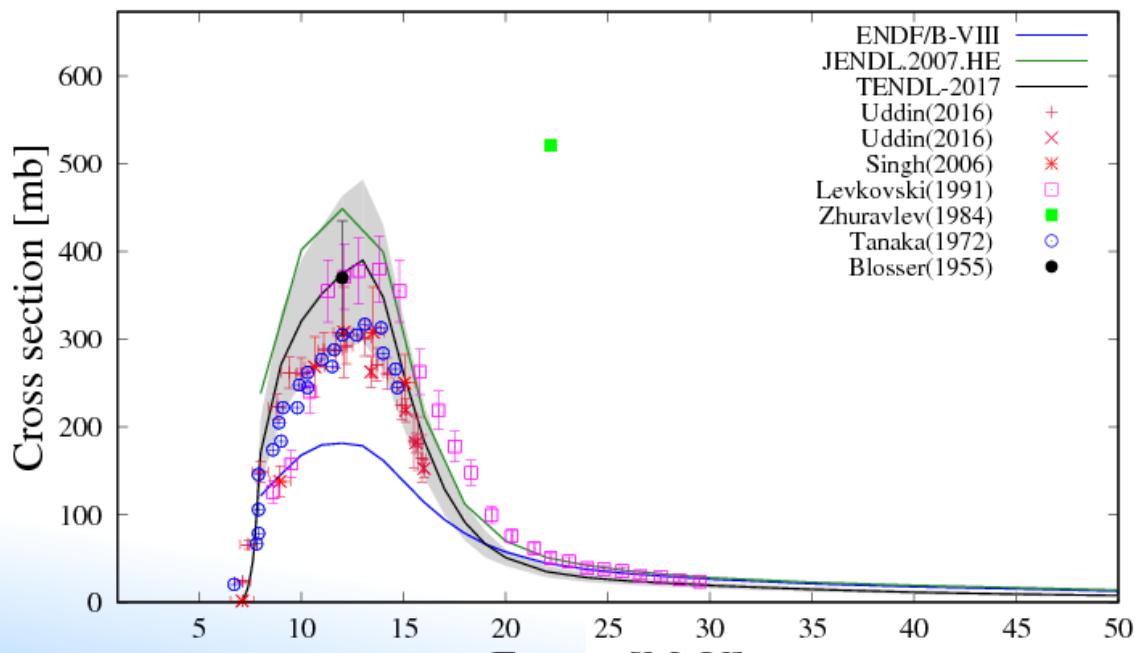




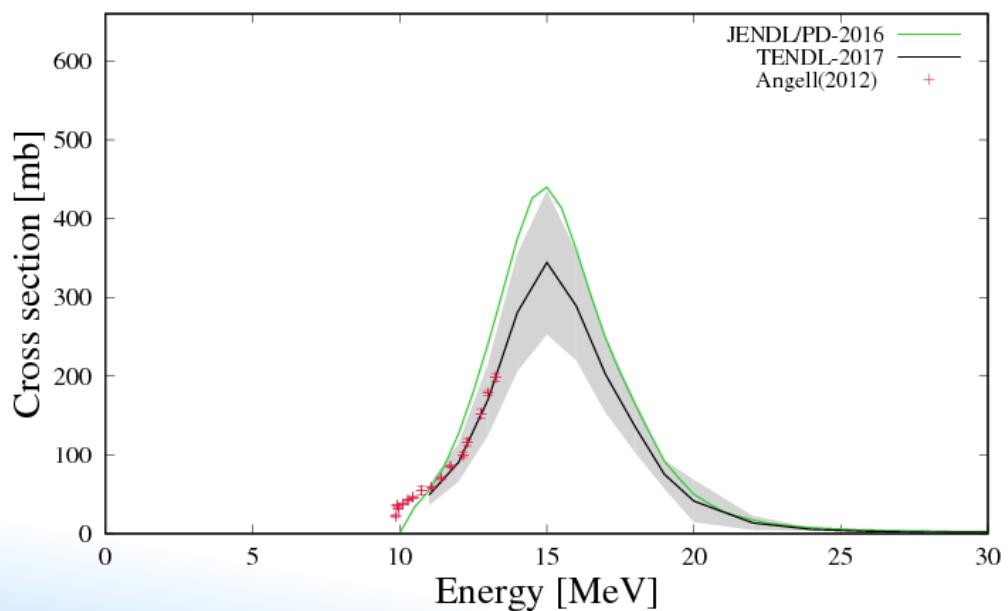
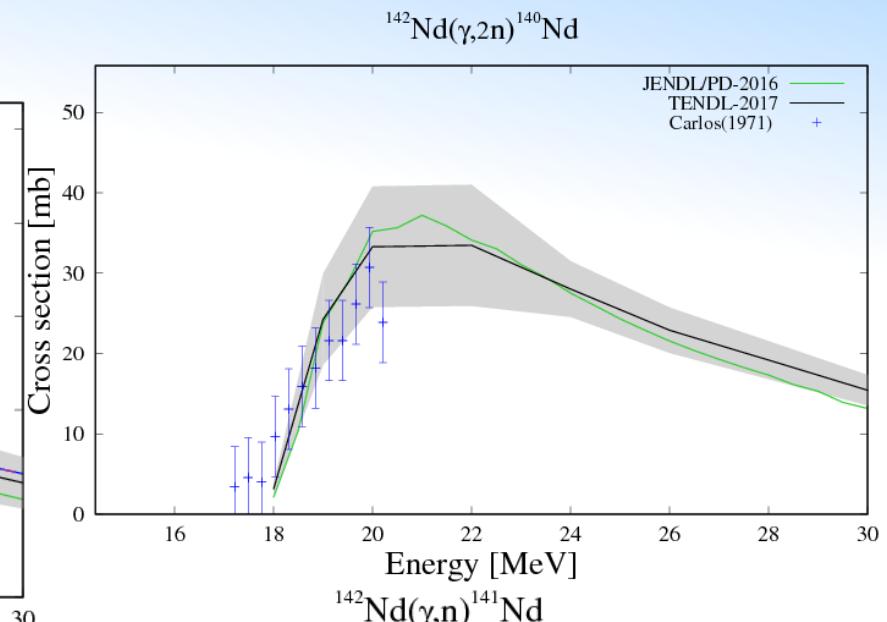
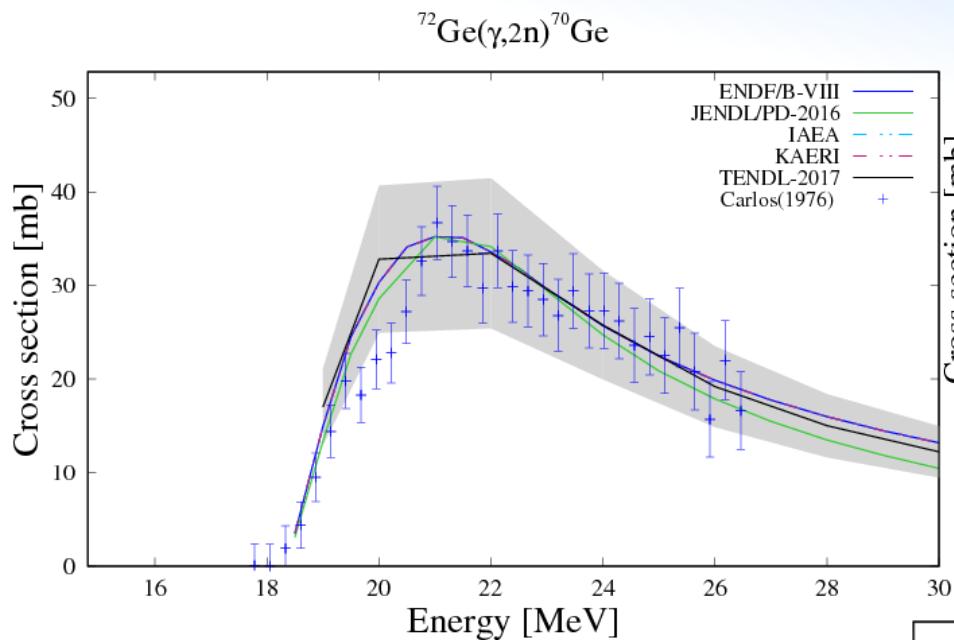
Protons



Plots for **all** particles, target nuclides, reaction channels and nuclear data libraries available at <ftp://ftp.nrg.eu/pub/www/talys/plots>

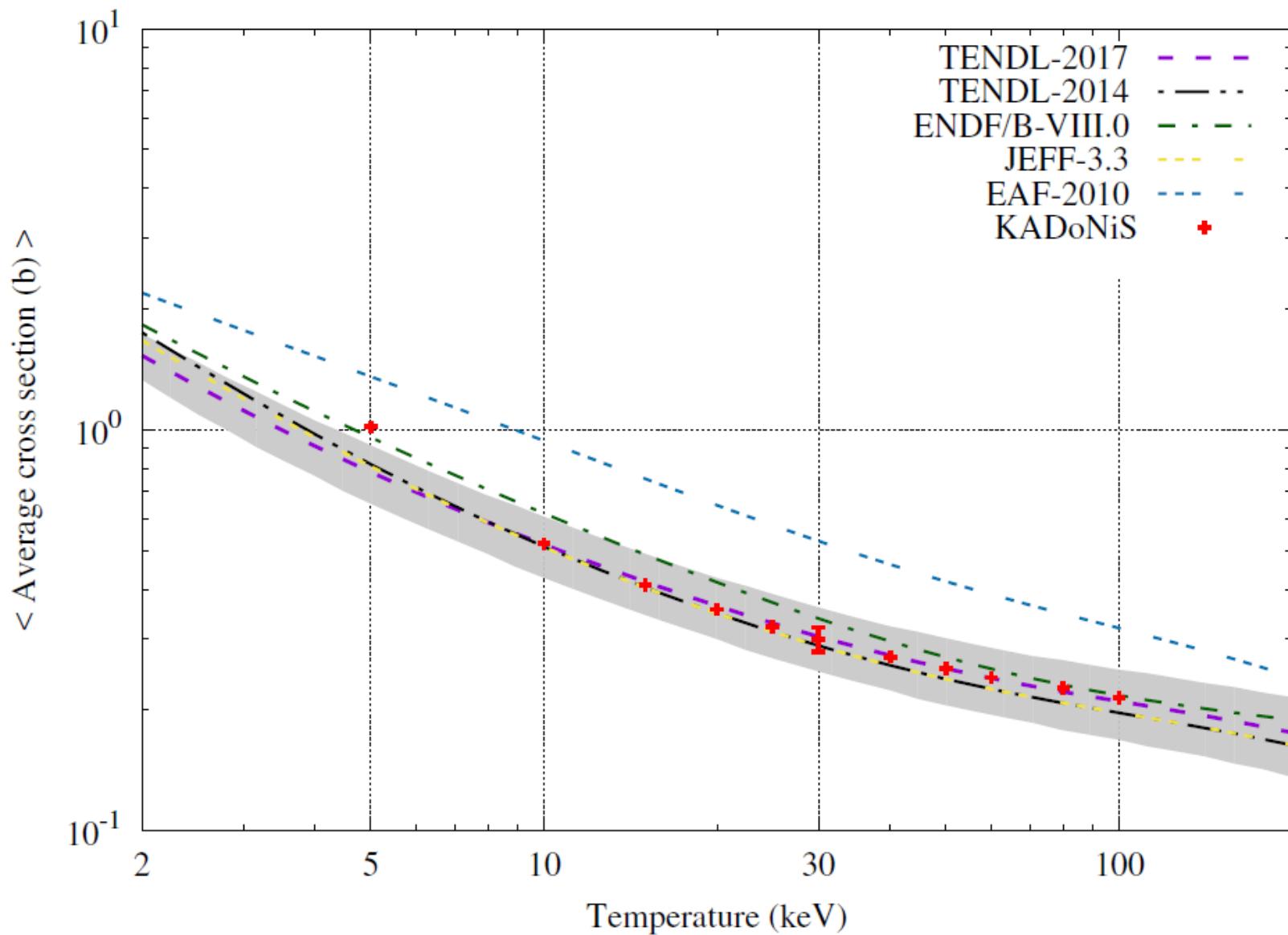


Photonuclear

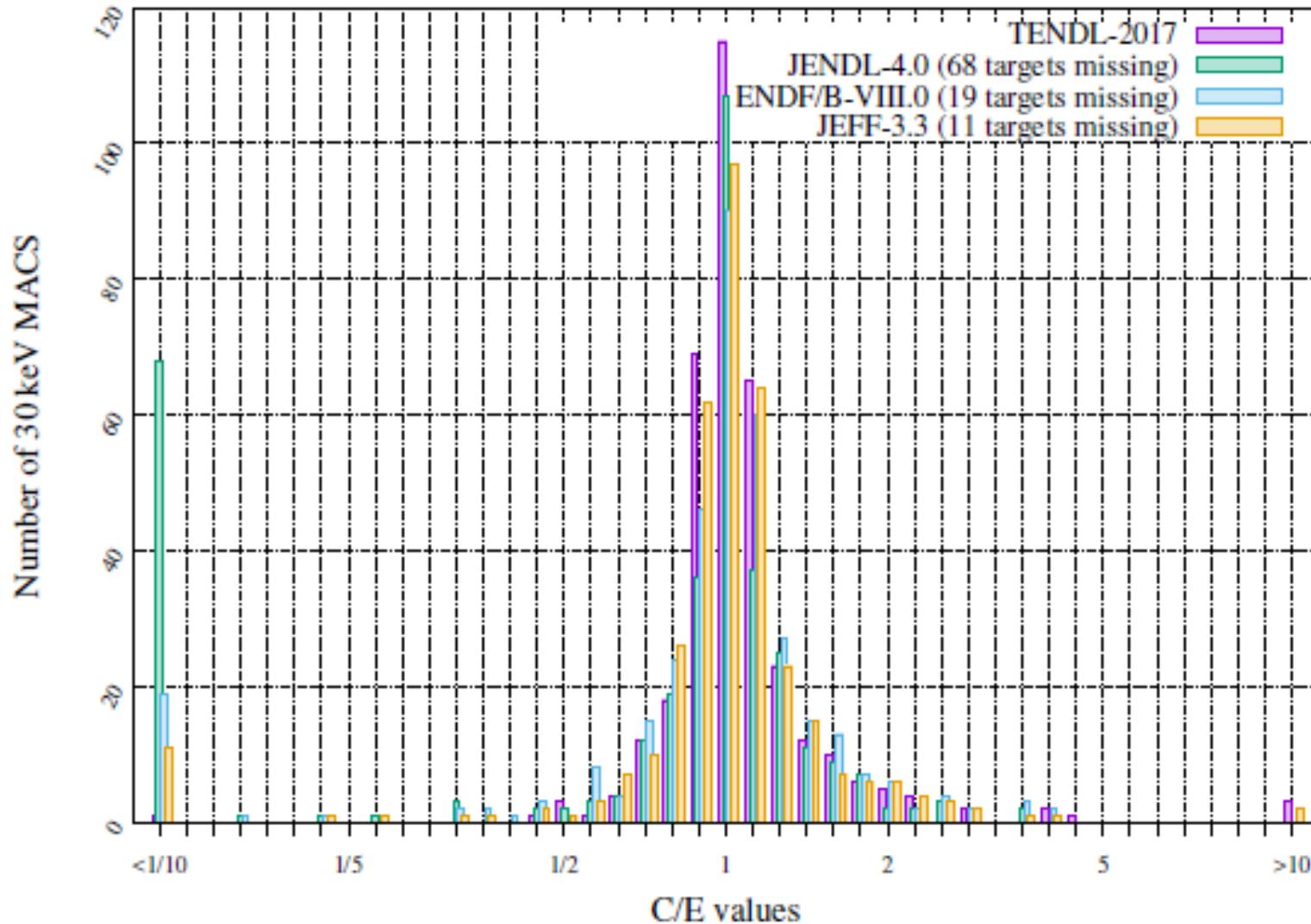


Plots for **all** particles, target nuclides, reaction channels and nuclear data libraries available at <ftp://ftp.nrg.eu/pub/www/talys/plots>

Maxwellian Averaged cross sections: Ce-136



Neutron capture: global 30 keV comparison



TENDL approach

Requires things which are also useful for the other approaches:

- Revision of the thermal, MACS, RI databases (partly derived from EXFOR) (**tbd**)
- Systematic evaluation of EXFOR that is quantified and reproducible, unlike an ENDF table (**tbd**)
- Direct availability of all historical nuclear data in an efficient form

The 'libraries' database (need better name)

Transform the most important ENDF libraries and EXFOR into simple accessible x-y(-dy) tables.

All created by a program called ENDFTABLES

Benefits:

- Cut in the middle: Reading and interpreting an ENDF-6/EXFOR library is done in separate step
- ENDF and EXFOR can be used efficiently by non-ENDF people
- Very efficient for plotting
- Reproducible use in adoption of data into other data libraries
- Automatic EXFOR quality scoring procedures
- Medical isotope production
- Collection of all EXFOR vs library plots
- Etc.

‘Libraries’

- Huge: about 200 Gb untarred, 8 million files
- <ftp://ftp.nrg.eu/pub/www/talys/libraries>
- get libraries*tar; tar zxf libraries*tar

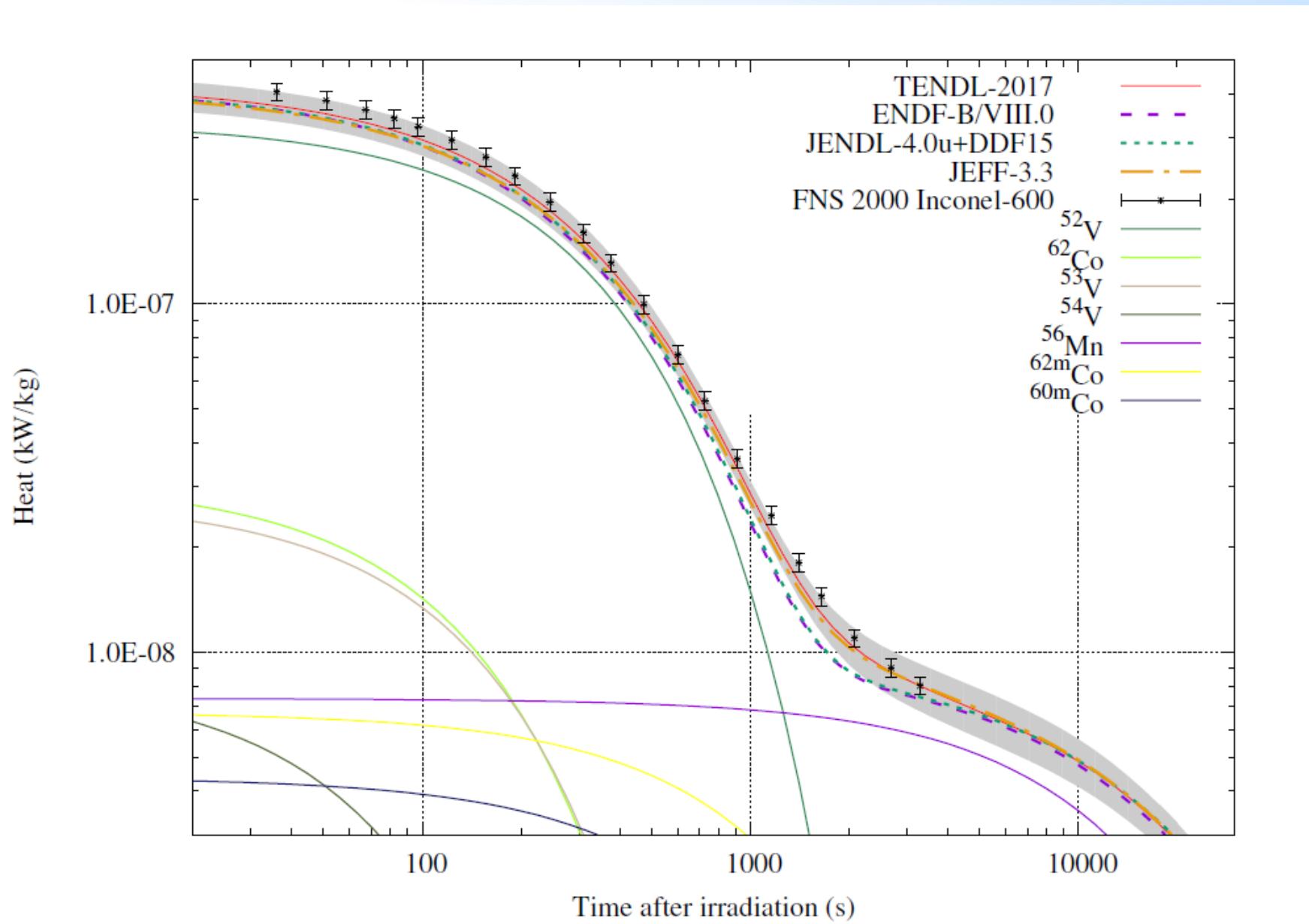
```
n/Fe054/jendl4.0/tables/angle/  
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n/Fe054/jendl4.0/tables/info/  
n/Fe054/jendl4.0/tables/resonance/  
n/Fe054/jendl4.0/tables/xs/
```

Cross sections

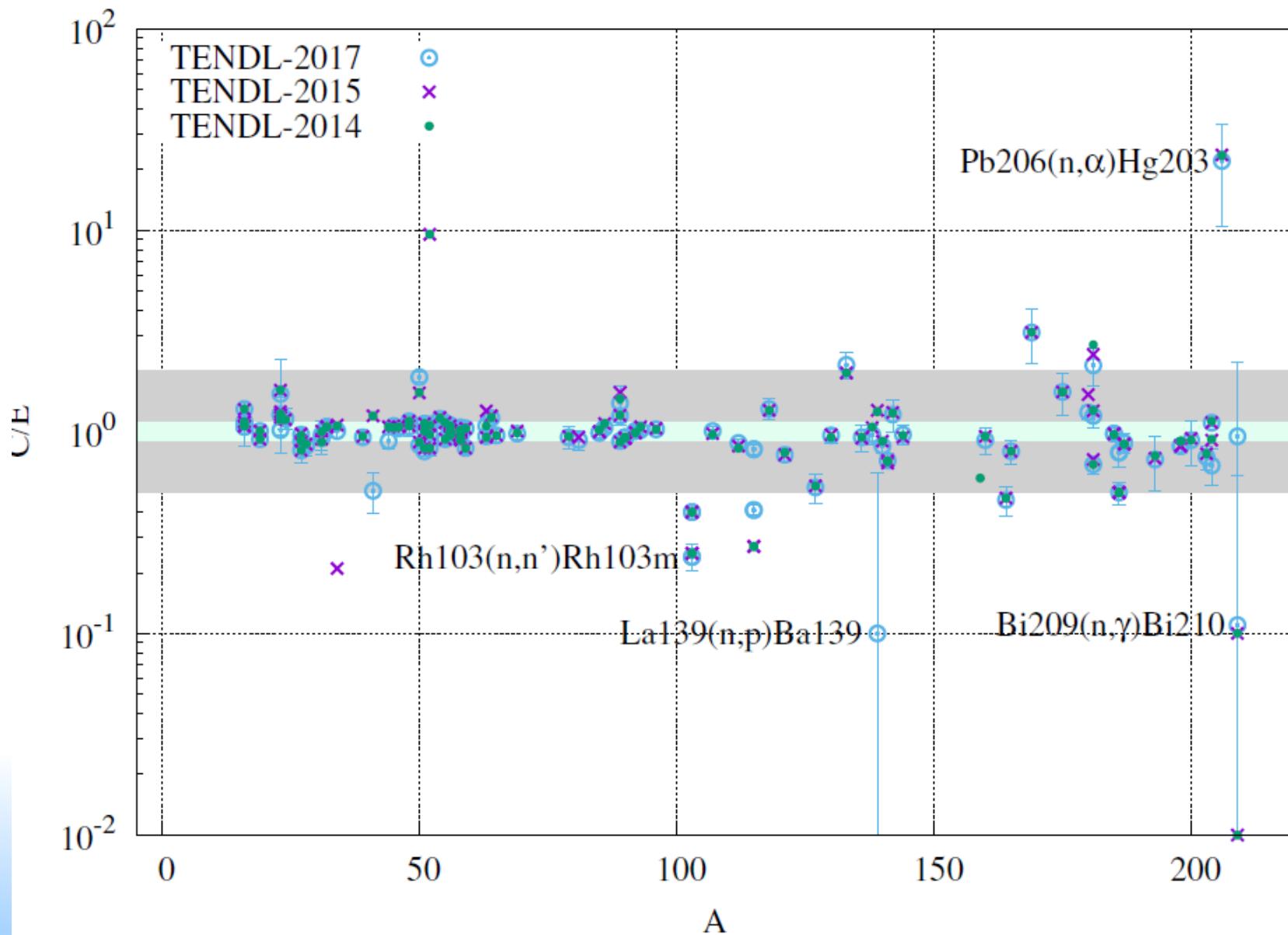
If we take the xs/ directory as the first example, we see the subdirectory looks as follows

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```

Fusion decay heat



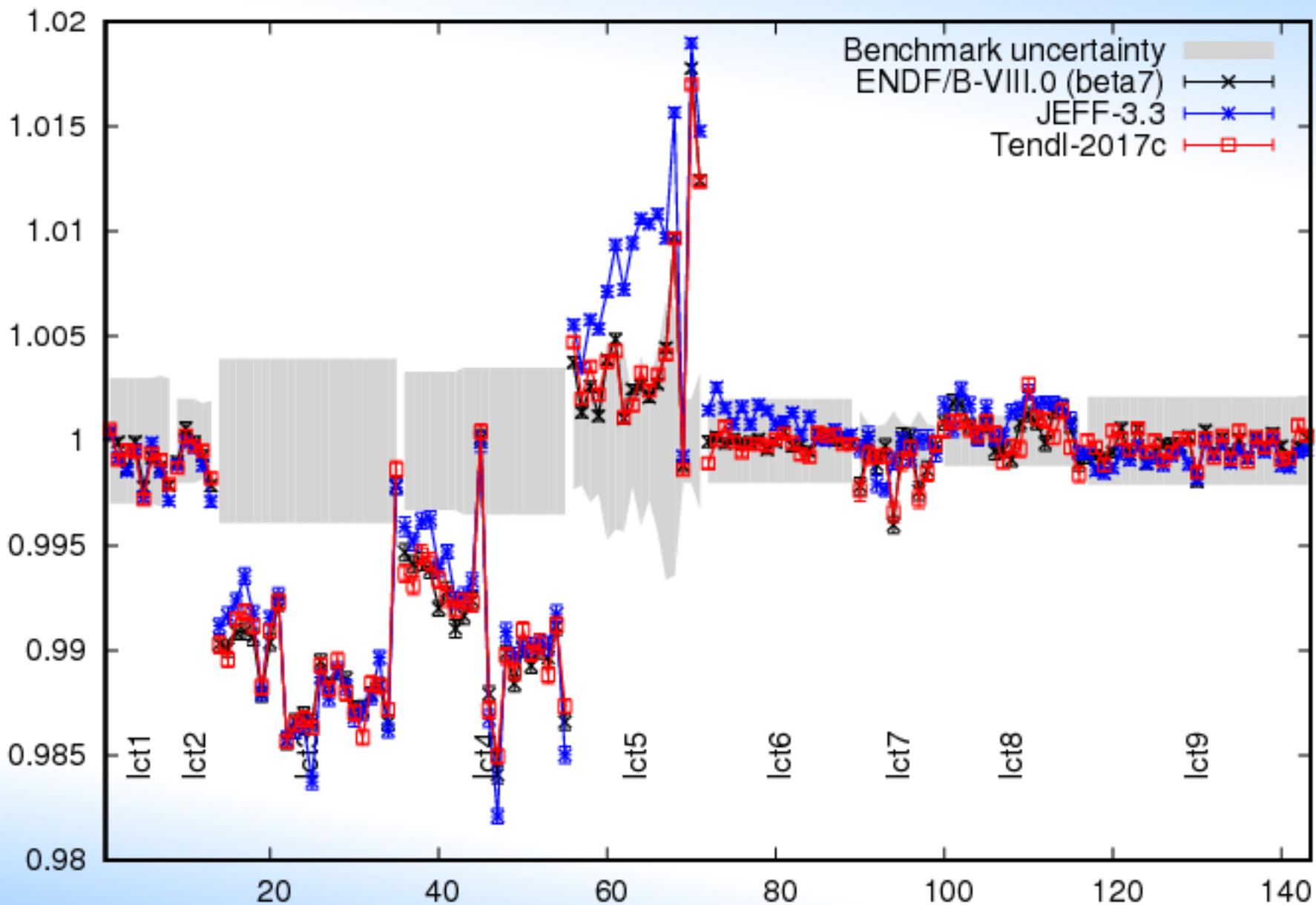
Fusion decay heat: global comparison



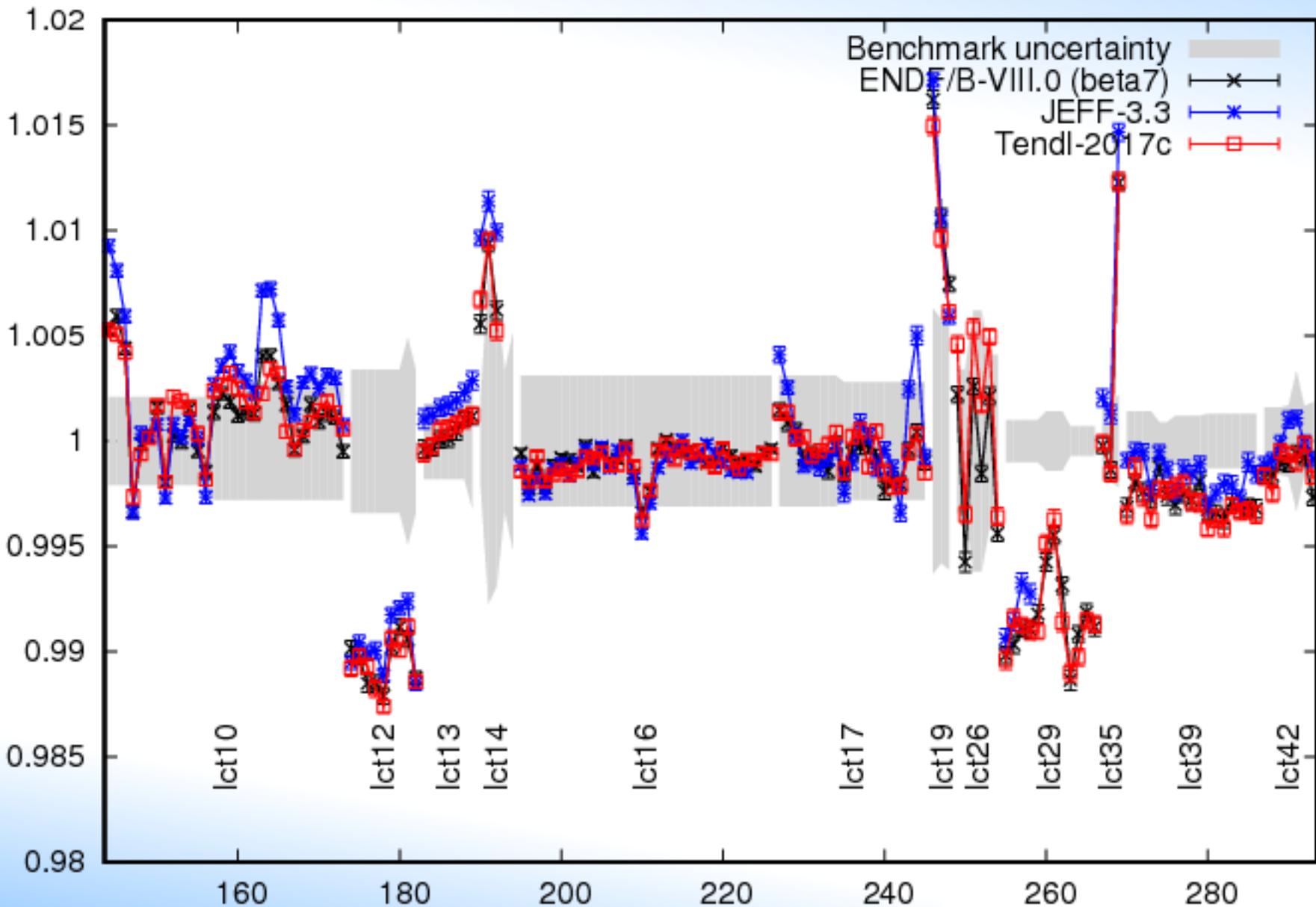
Criticality benchmarking by NRG (Steven van der Marck)

- Remember: TENDL-2017 neutron library adopts ^1H – ^{19}F , ^{232}Th , $^{233,235,238}\text{U}$, ^{239}Pu from ENDF/B-VIII
- This will thus test all other non-fissile materials for performance in criticality
- MCNP for ~2550 ICSBEP cases
- Observations: Improvement needed for Ni in fast range, Cu and Pb

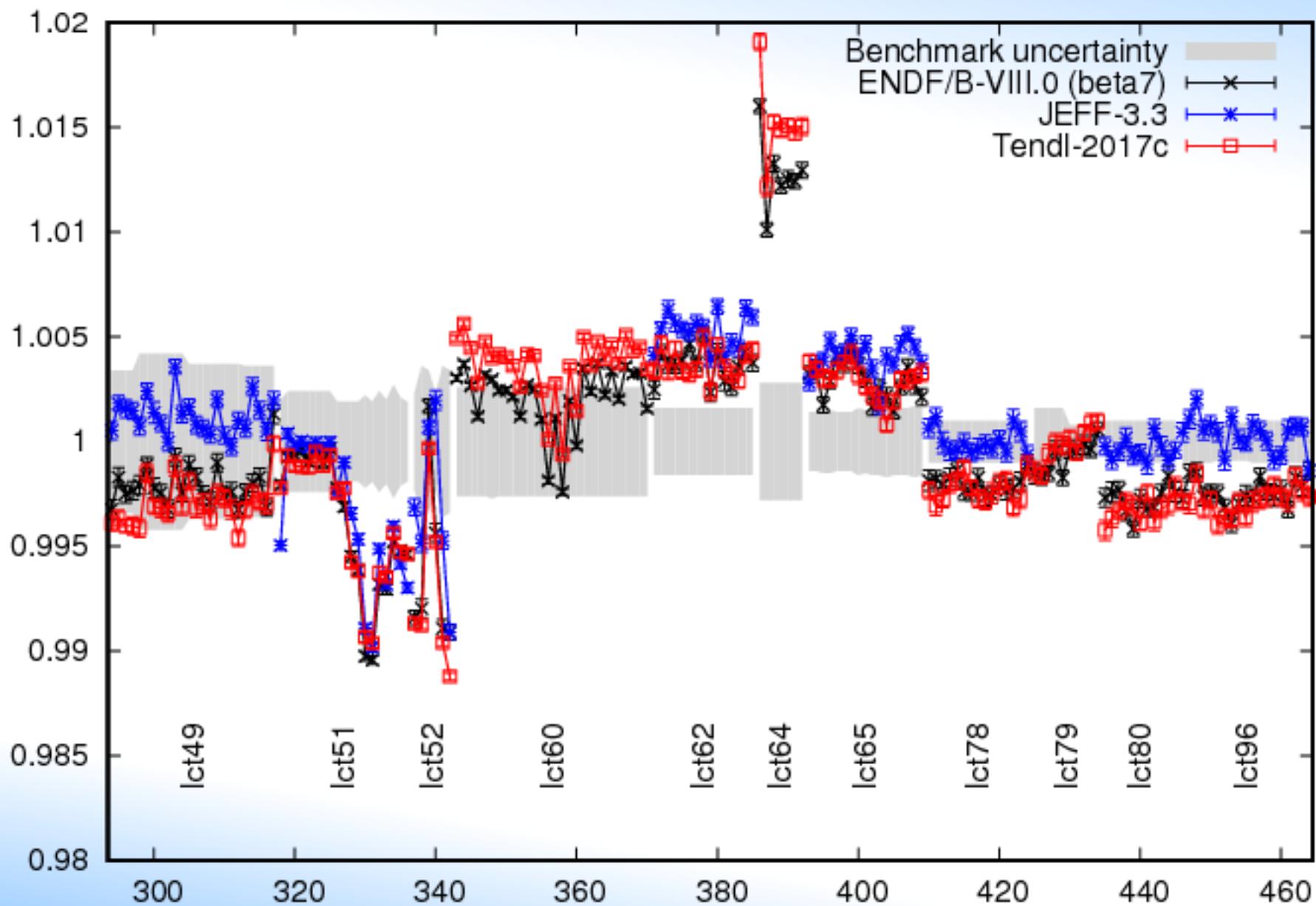
leu-comp-therm



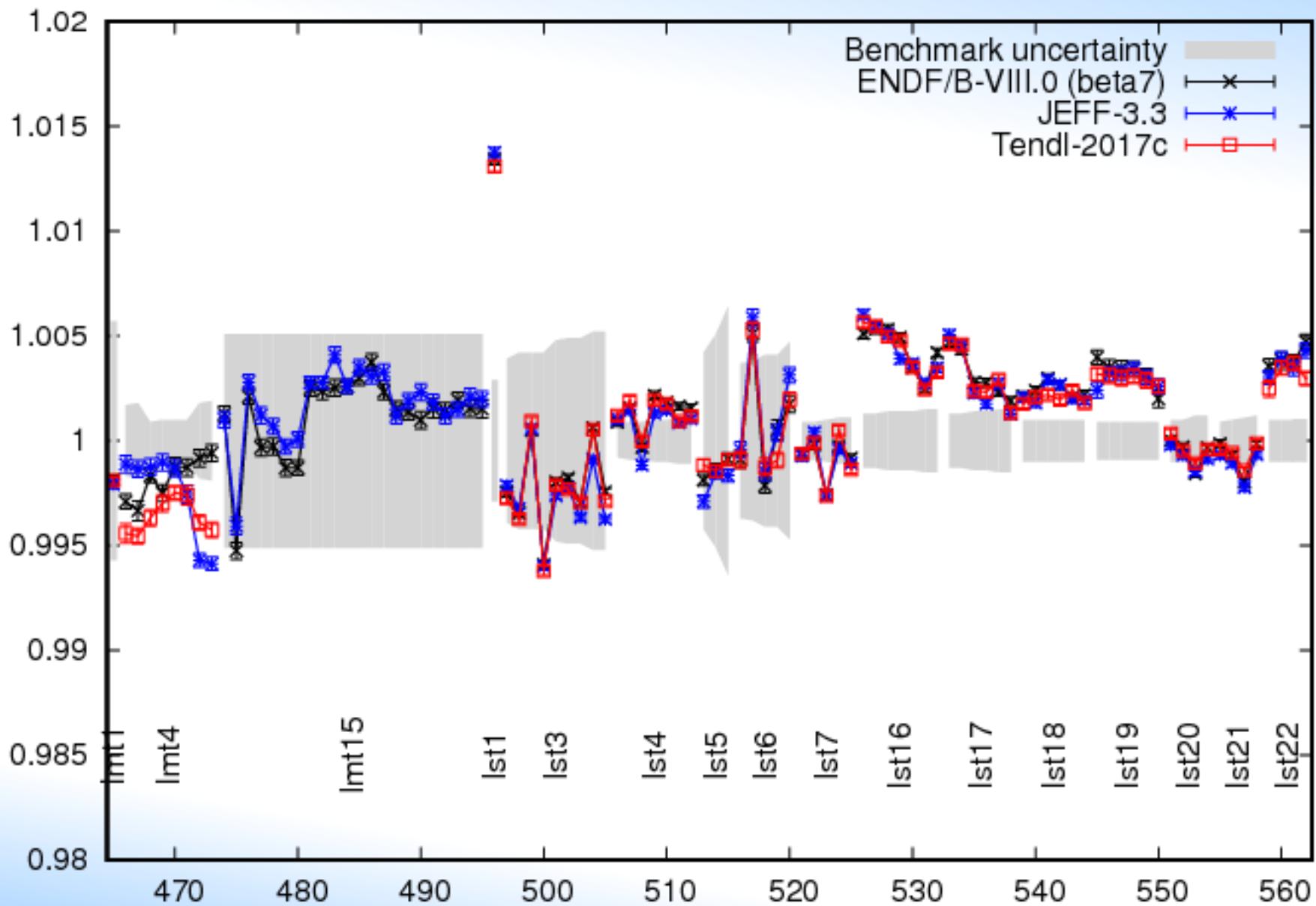
leu-comp-therm (2)



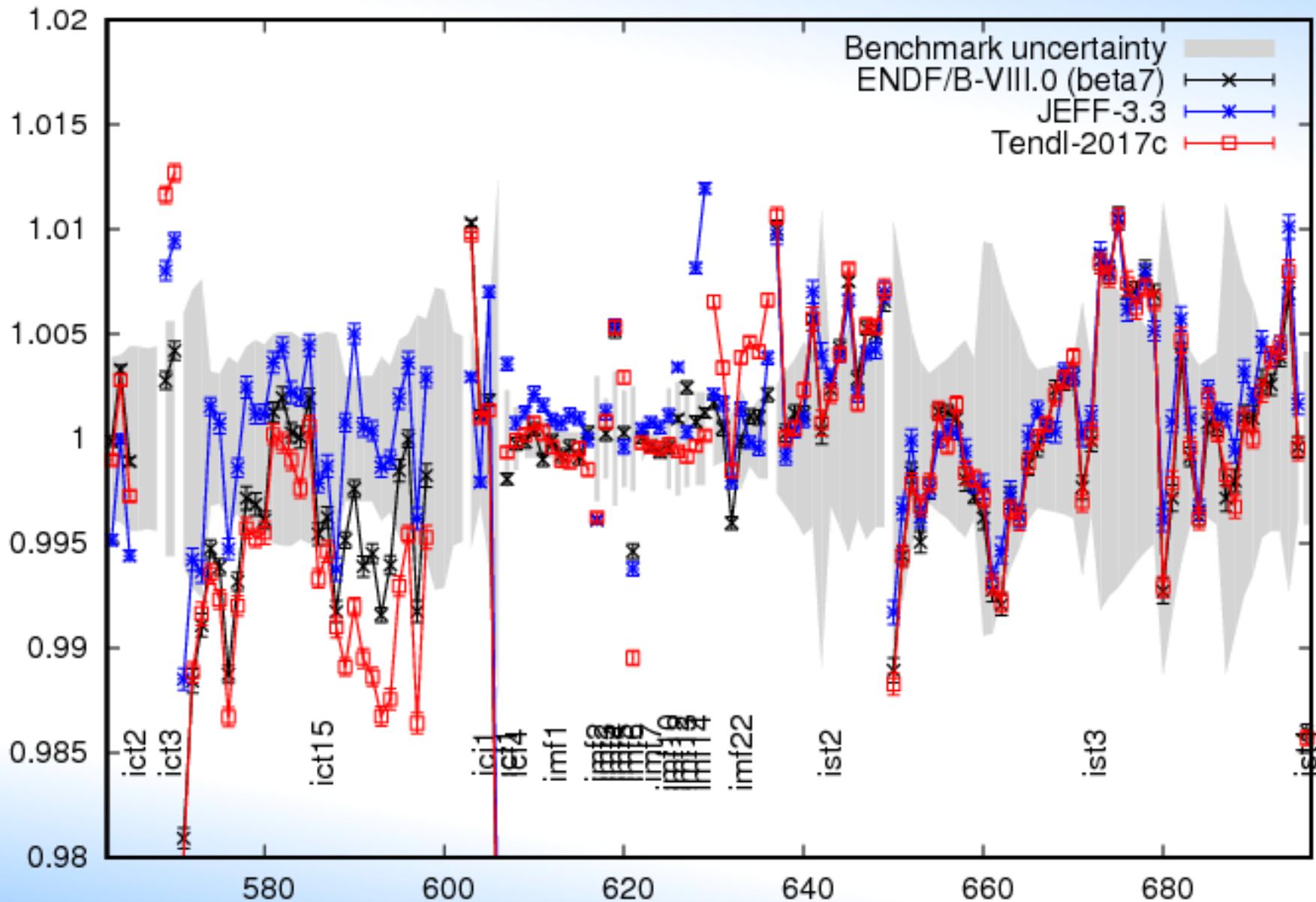
leu-comp-therm (3)



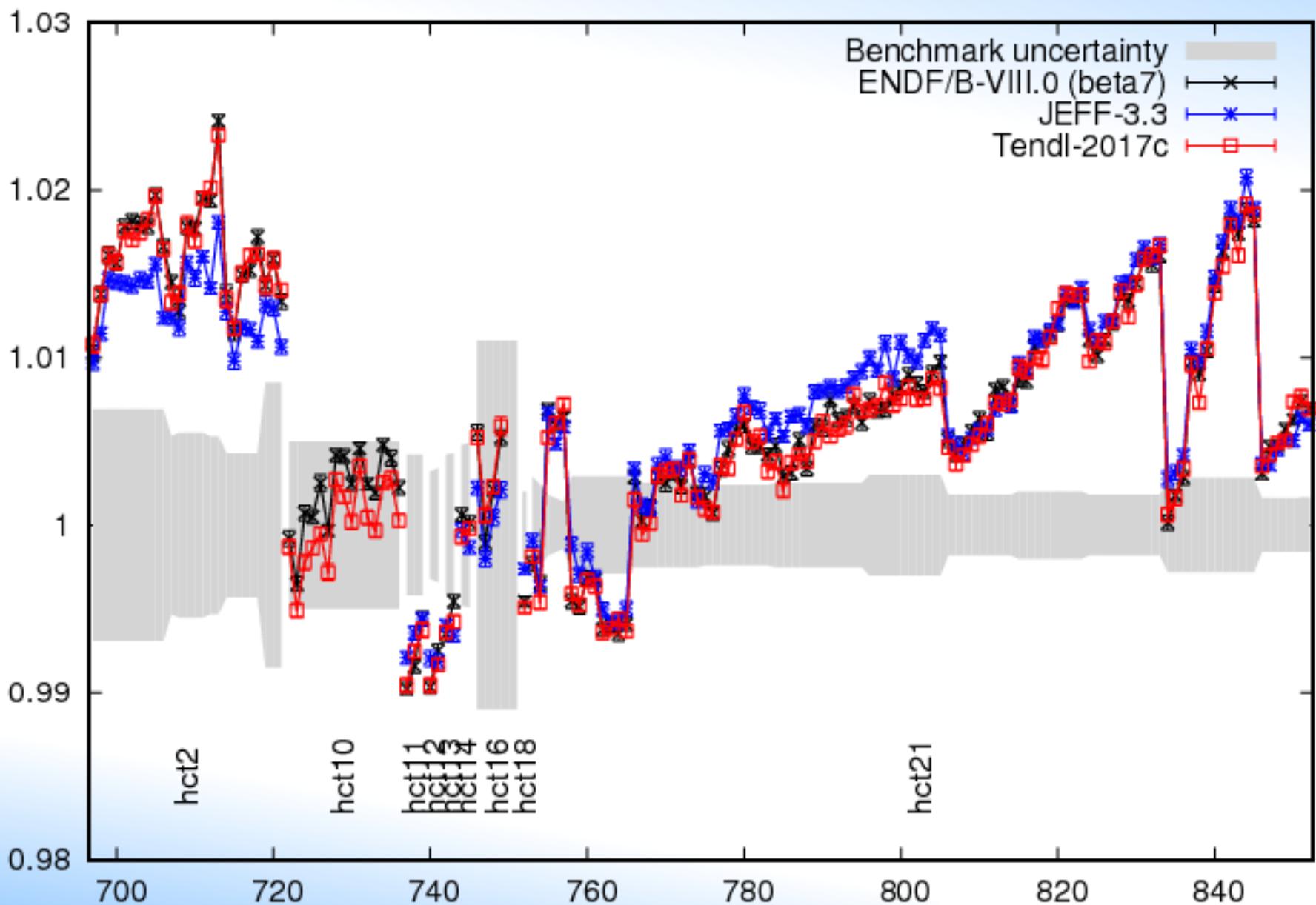
leu-met/sol-therm



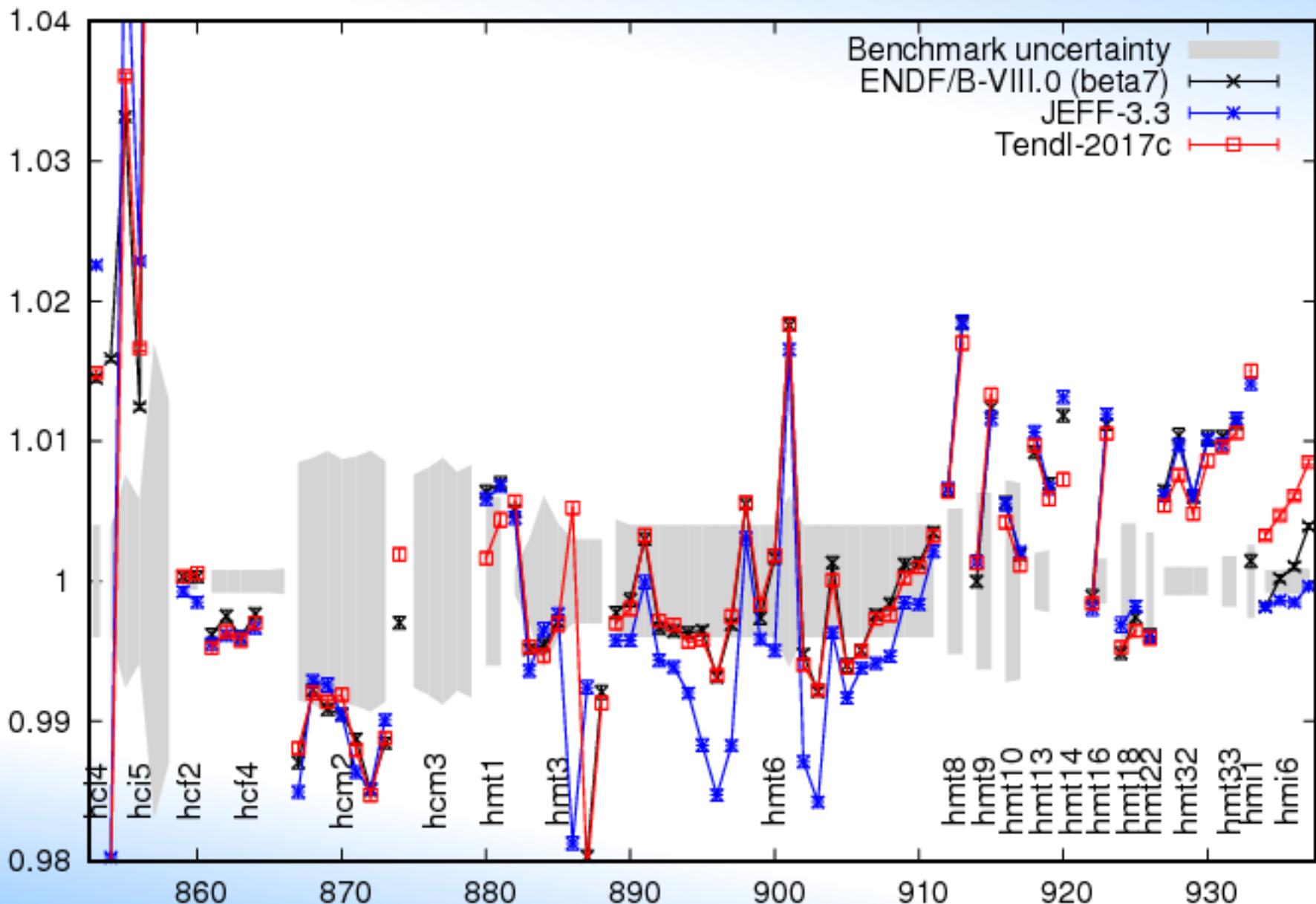
ieu



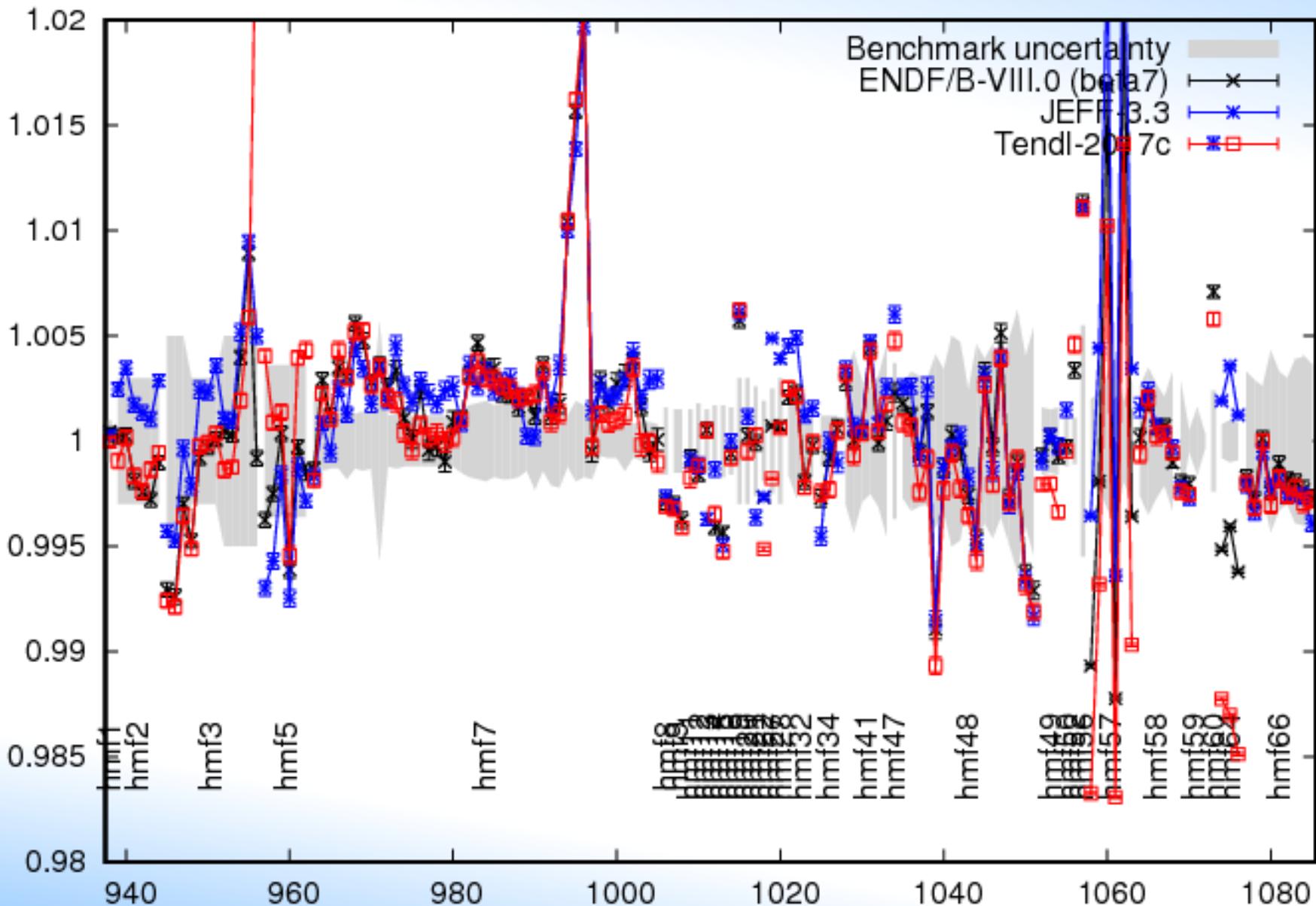
heu-comp-therm



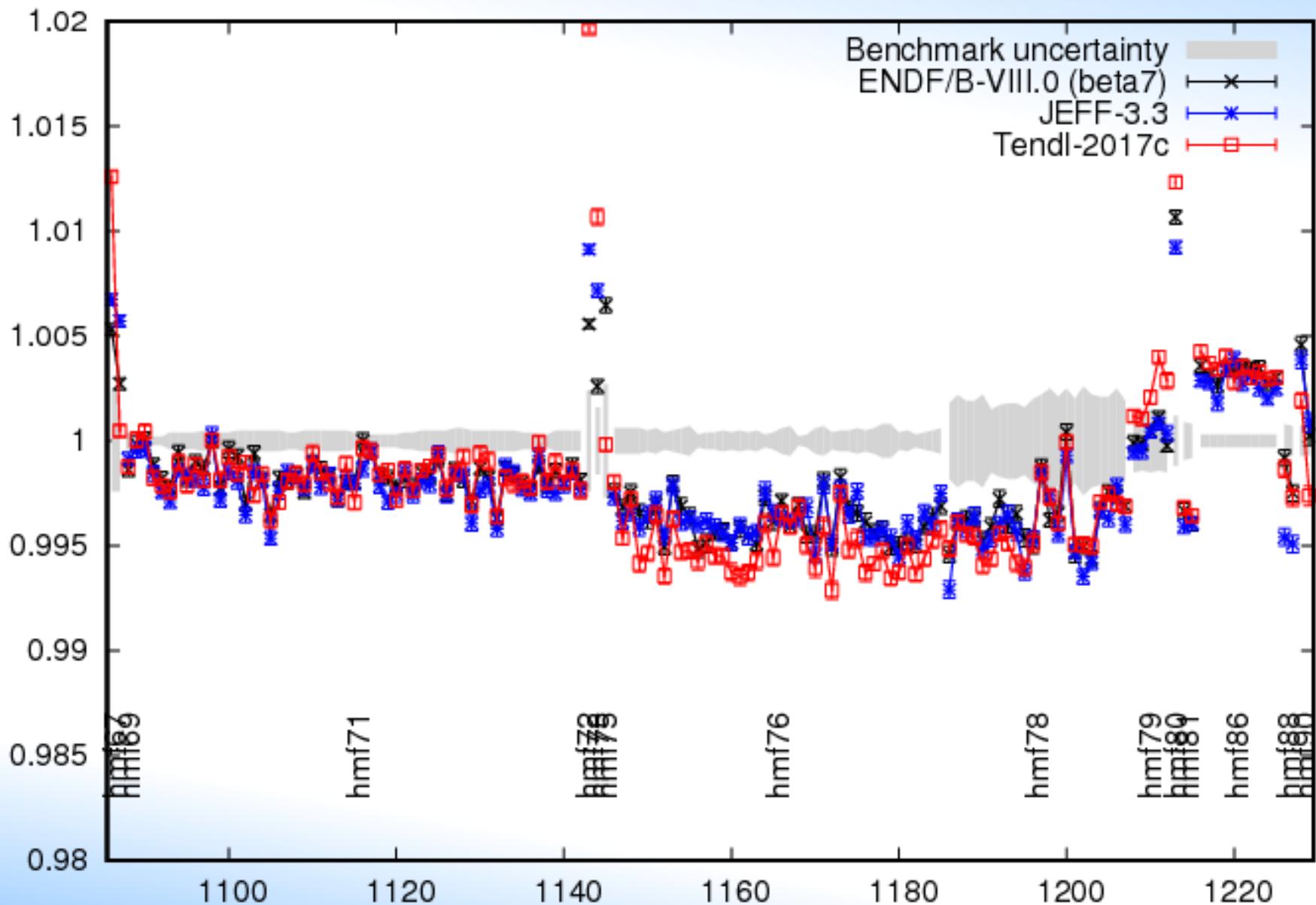
heu-comp



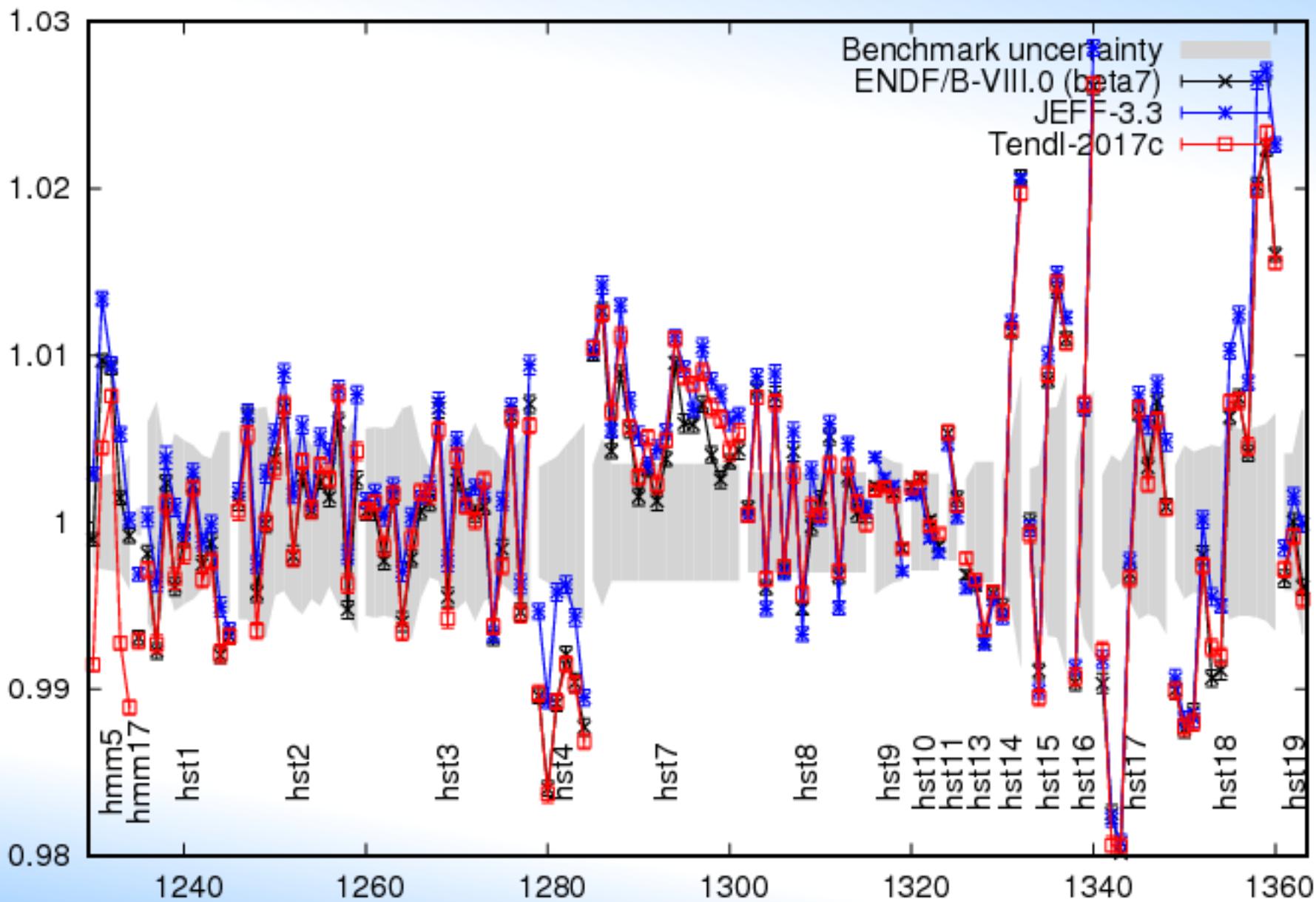
heu-met-fast



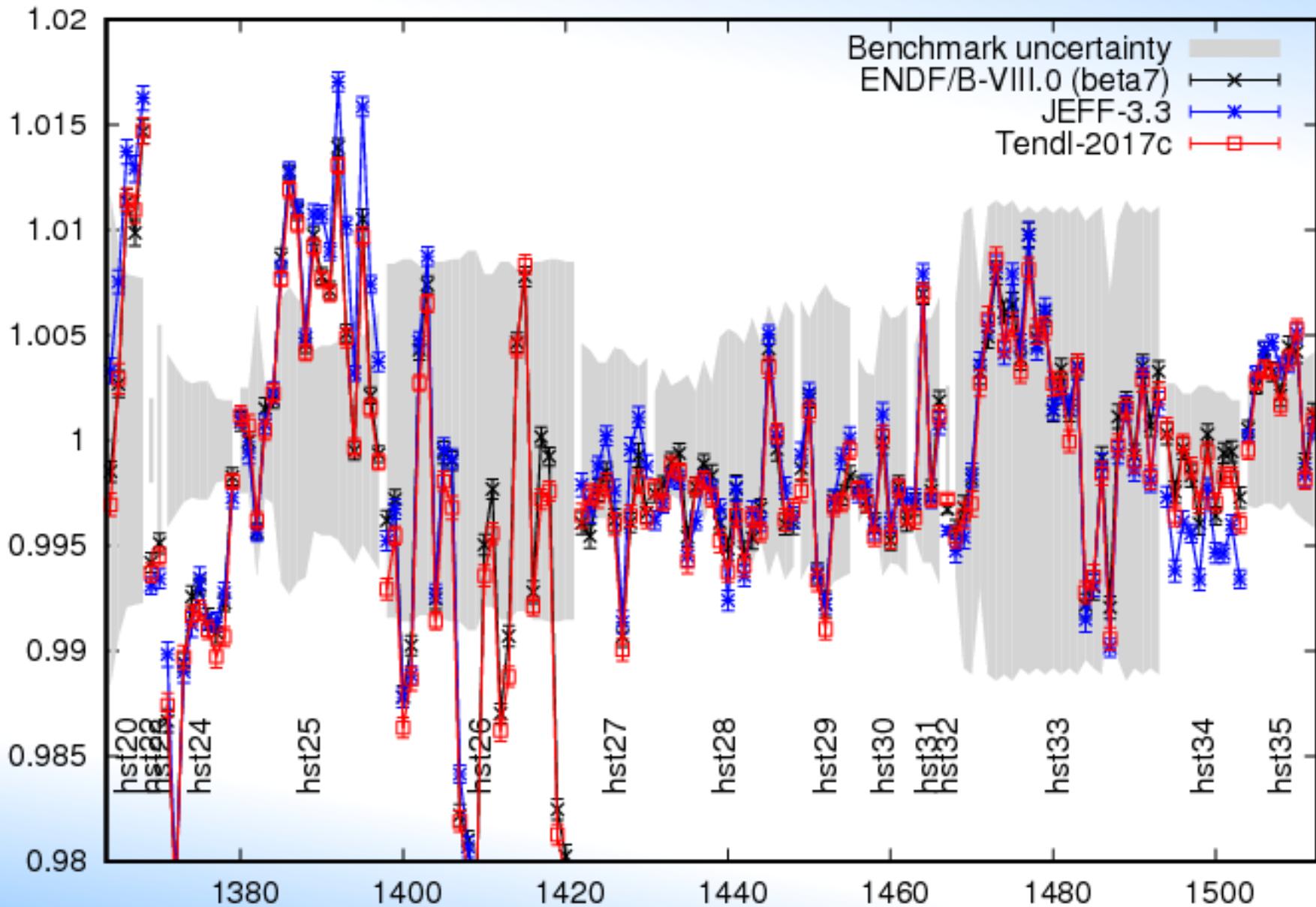
heu-met-fast (2)



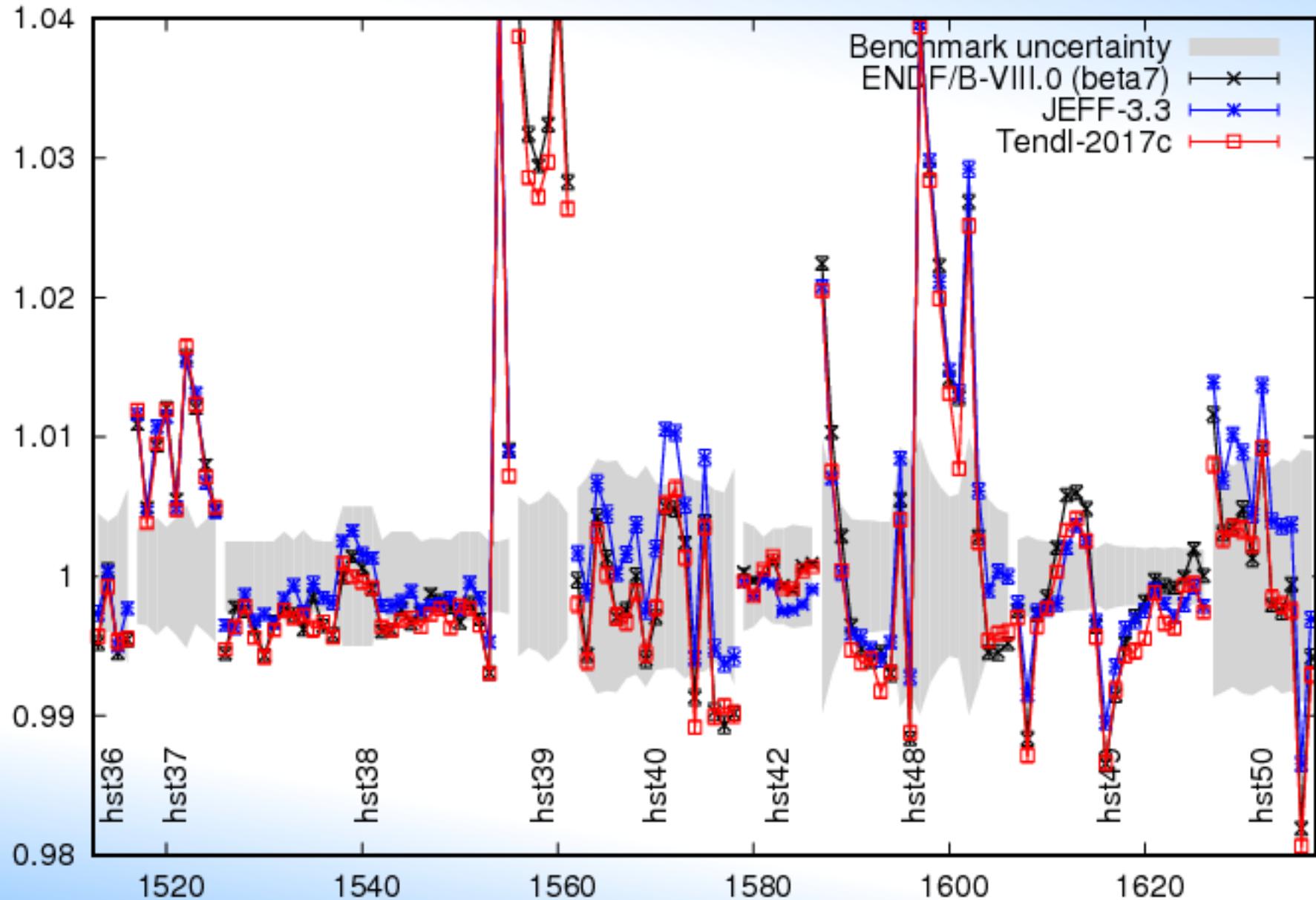
heu-sol-therm



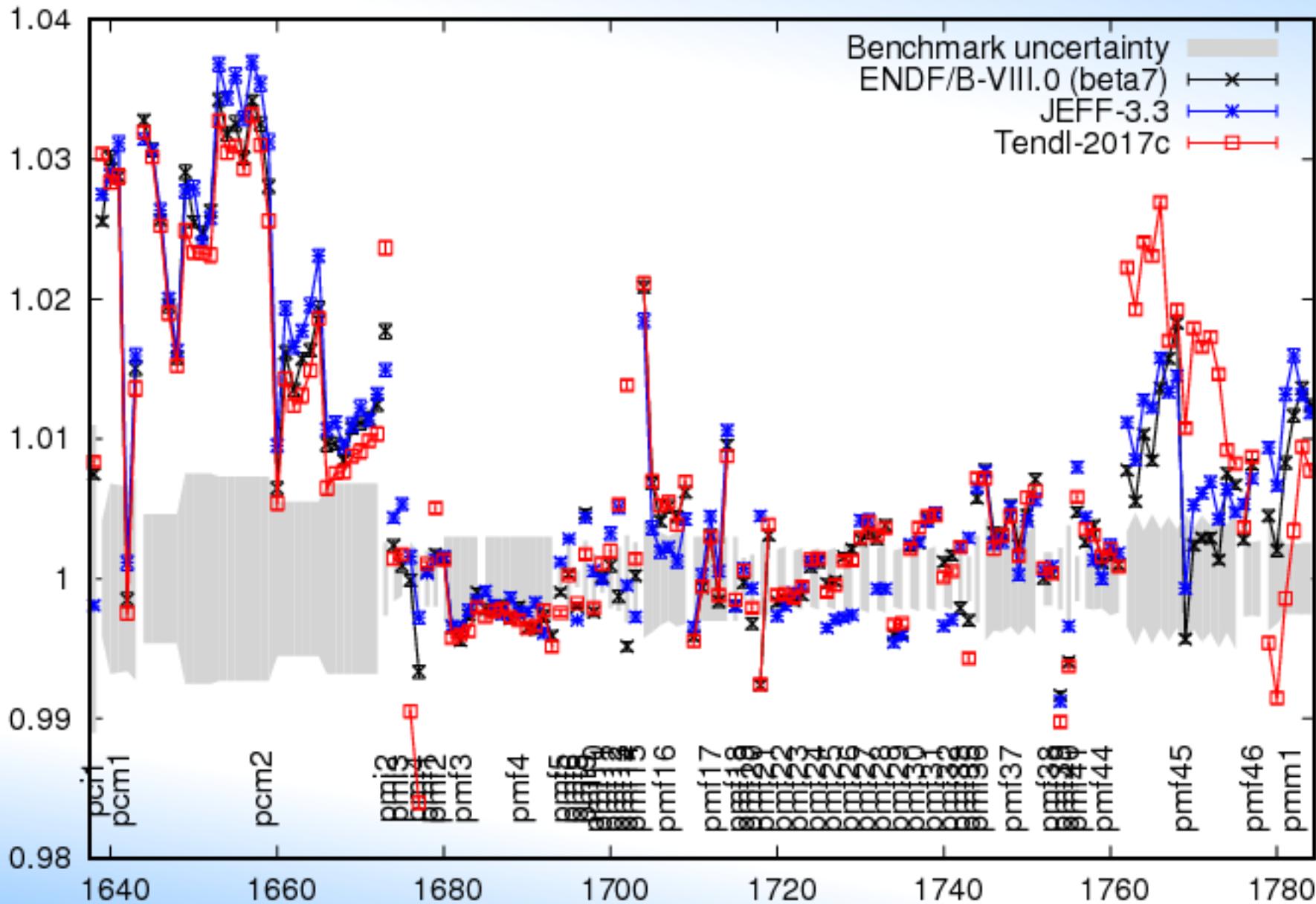
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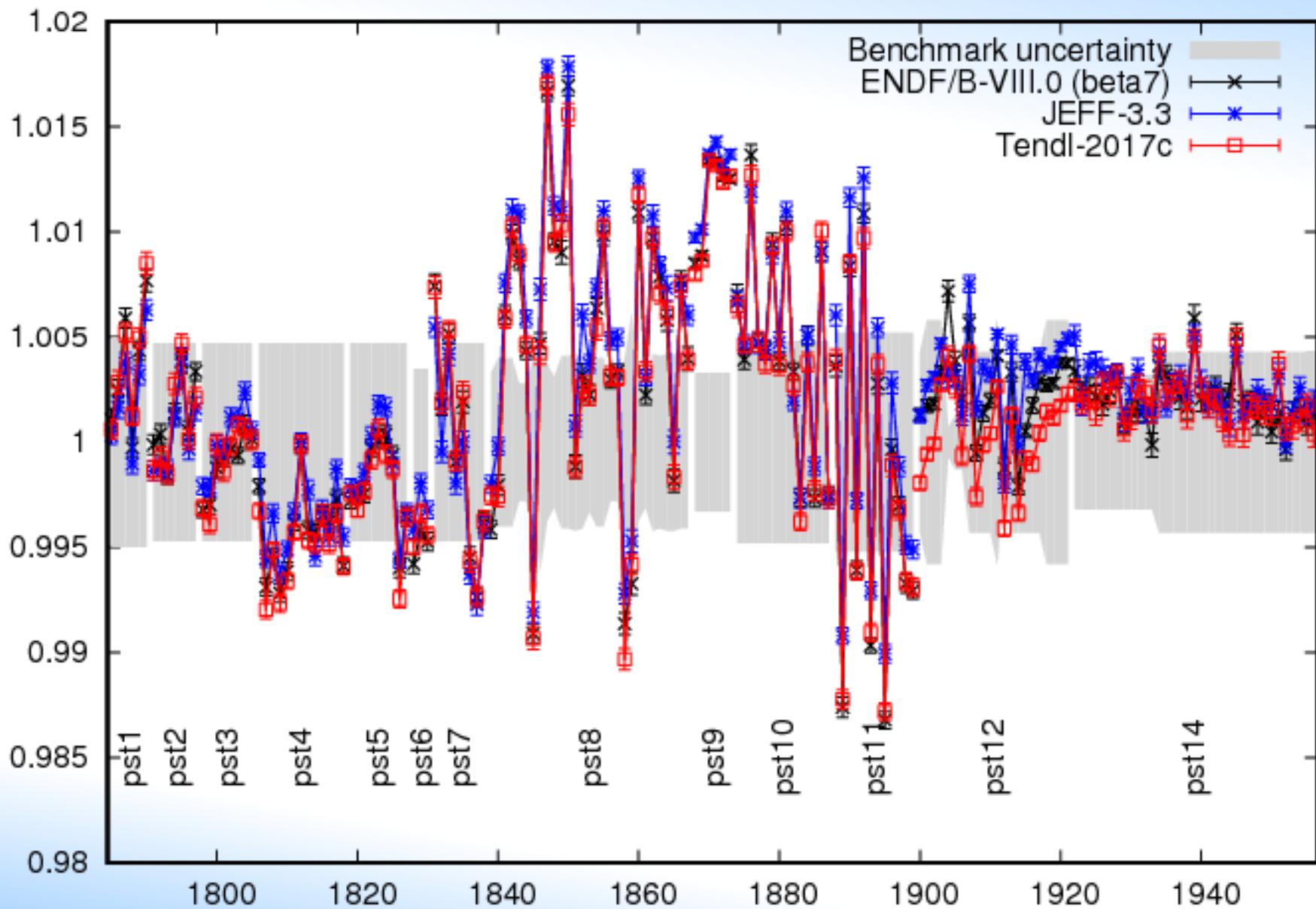
heu-sol-therm (3)



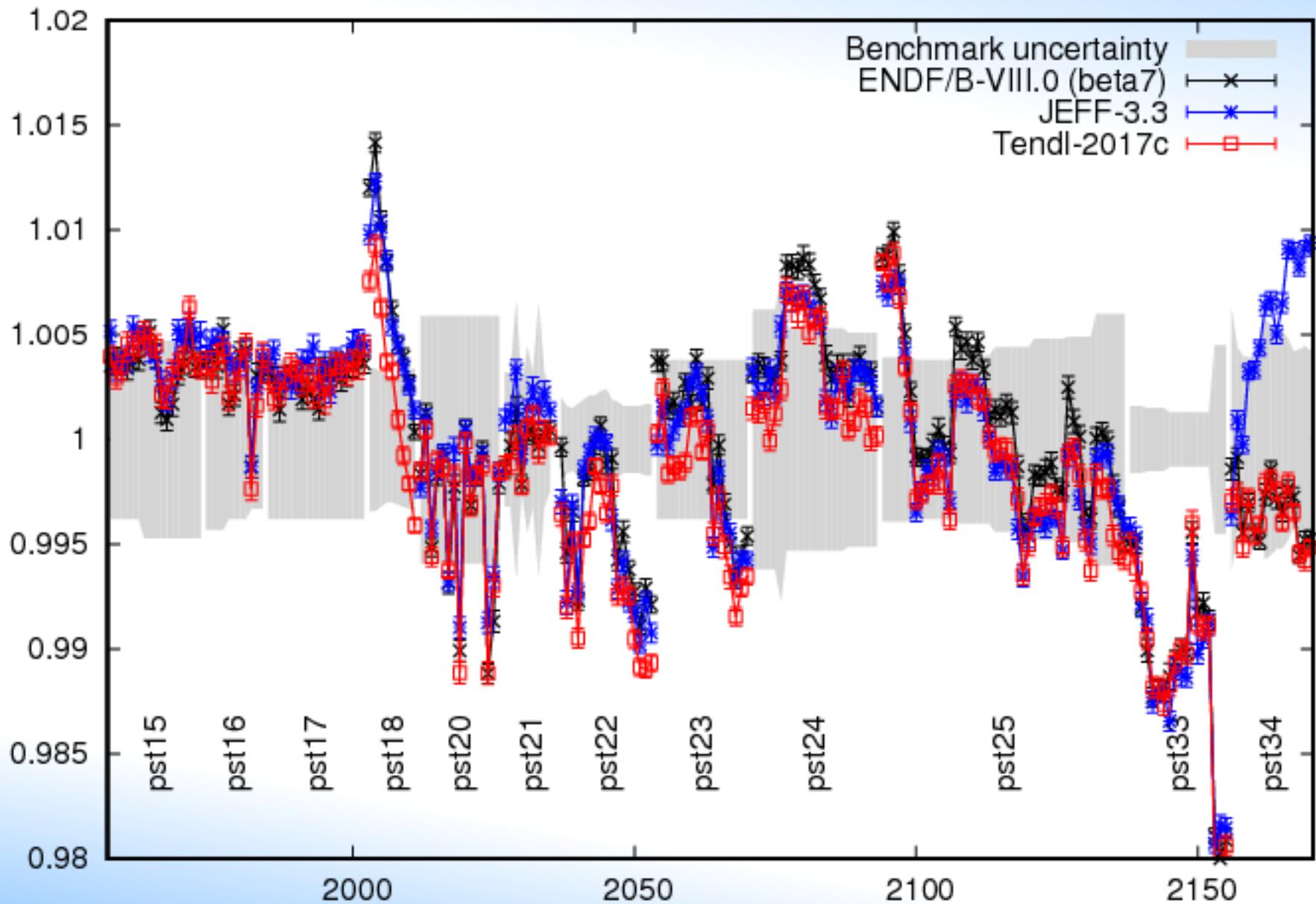
pu-met-fast



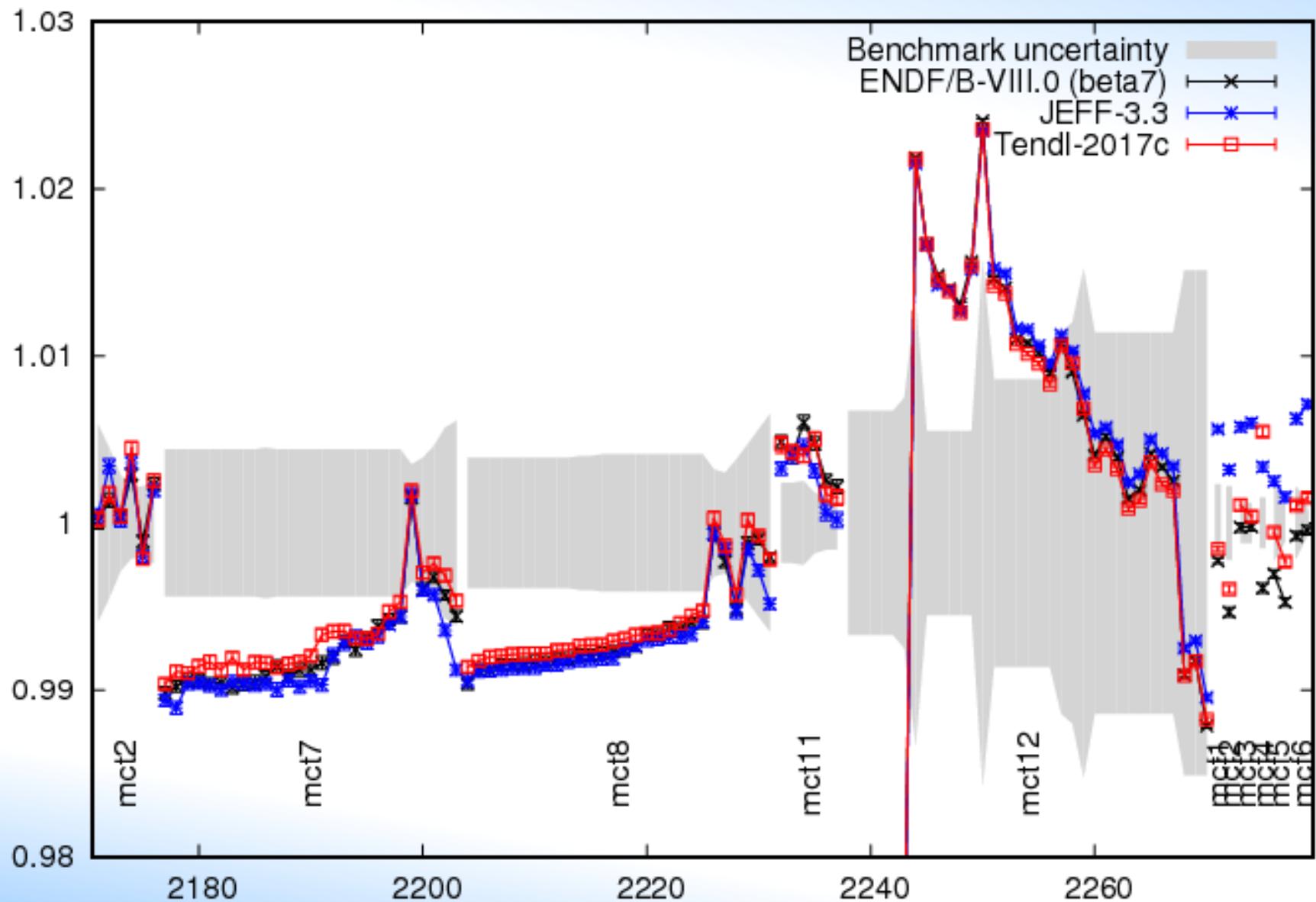
pu-sol-therm



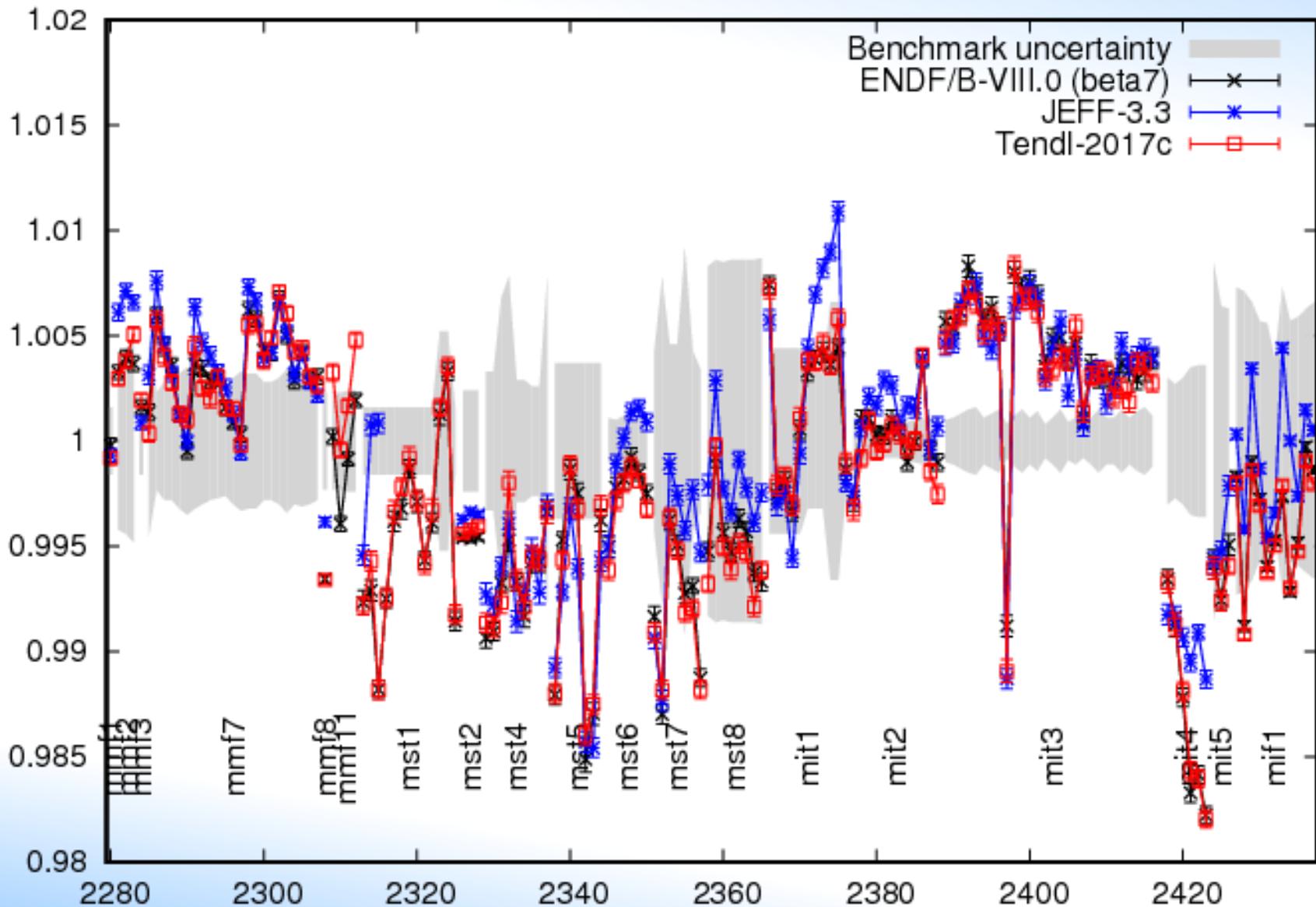
pu-sol-therm (2)



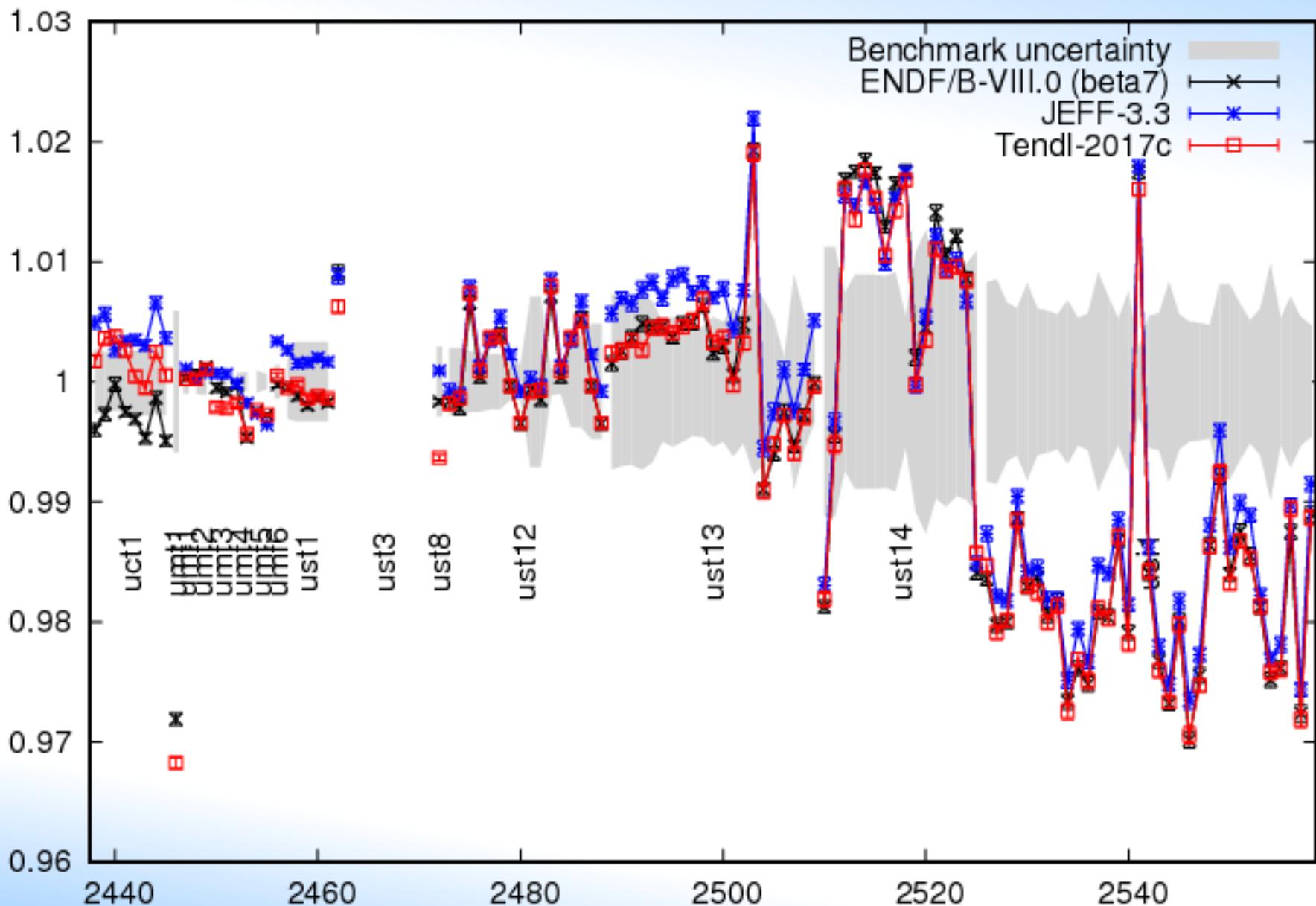
mix-comp-fast



mix-misc-fast



u233-sol-inter



Next TALYS: 2018-2019

- TALYS-2.0
 - Full Fortran-95
 - TEFAL code included:
 - Complete ENDF formatting
 - Allows TALYS users to create ENDF data libraries
 - TASMAN code included:
 - Uncertainties, covariances, sensitivity profiles, Bayesian Monte Carlo, Total Monte Carlo
 - Allows TALYS users to perform uncertainty analyses, random nuclear data libraries and covariance evaluations
 - Resonance parameters via external database
 - Opens up TENDL-like production to all users

Next TENDL: 2018

- Produced with TALYS-2.0
- Correct remaining deficiencies:
 - (n,p) and (n,alpha) resonances for low-energy positive Q-value reactions
 - Isomeric ratio for thermal (n,gamma) reactions
- Systematic validation scheme:
 - CCFE: Integral activation measurements, decay heat, radiation damage, etc. (Gilbert et al.)
 - NRG: van der Marck criticality (ICSBEP) and shielding (SINBAD) benchmarking suite
 - PSI: EXFOR, thermal, MACS and resonance validation, library optimization with the “Petten method”, FPY (Rochman)
 - IAEA: “EASY-database”, processing, all non-criticality validation (Sublet), criticality (Trkov), differential (Koning) validation
 - Try to minimize the turnaround time. Ideally: integral testing during evaluation
- TENDL paper in Nuclear Data Sheets, January 2019.



IAEA

60 Years

Atoms for Peace and Development

Thank you!

