
Nuclear Science Opportunities at LANSCE/Lujan Center

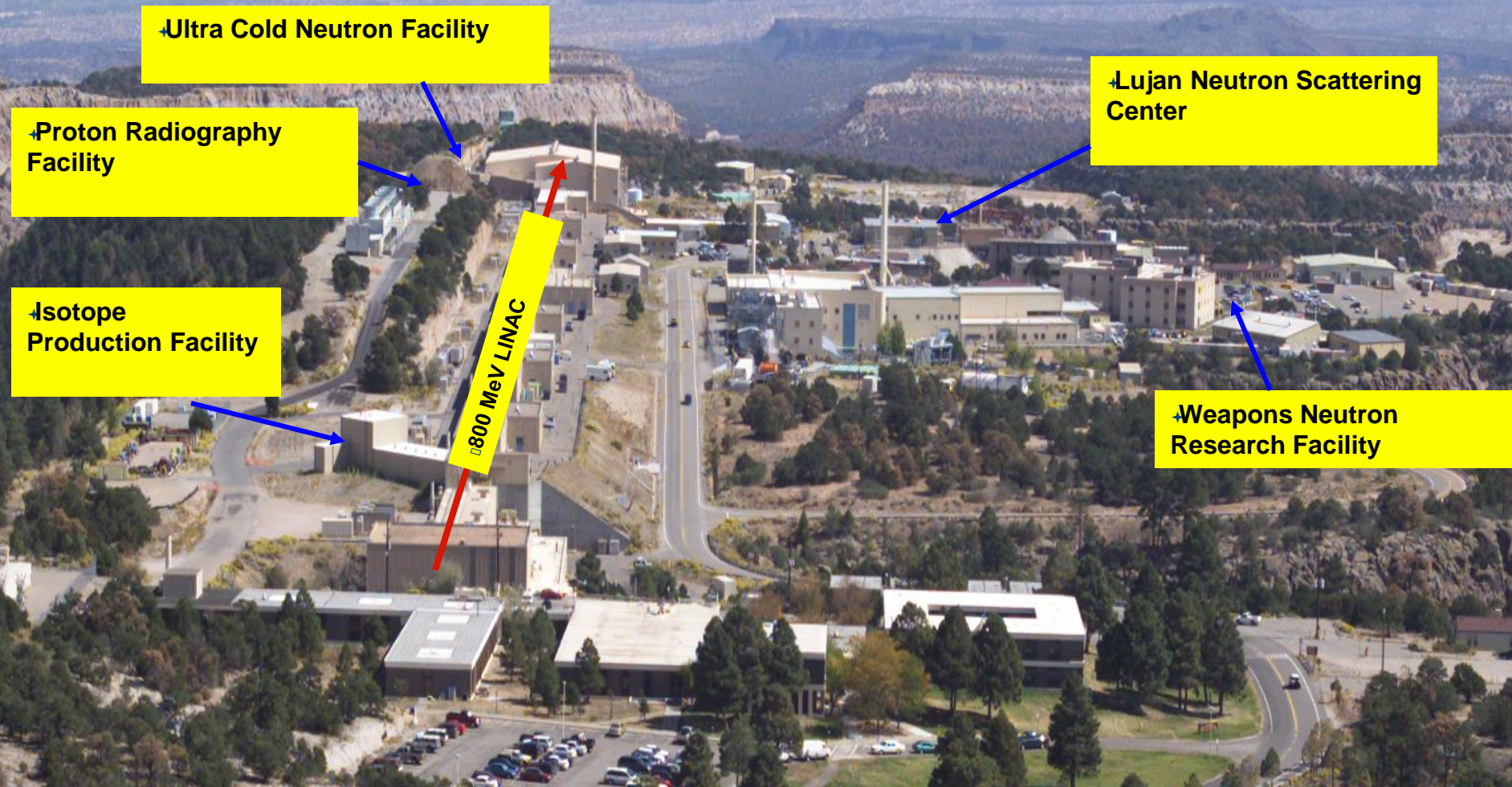
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Talk at the Workshop on Perspectives on Nuclear Data for the Next
Decade, October 14, 2014, Bruyeres-le-Chatel, Paris, CEA

Abstract

A future research program planned for LANSCE/Lujan is described. This has been motivated by changes at Lujan following the DOE decision to end the materials science user program. We have an opportunity to design a new target, for improved measurements of nuclear cross sections in the 1-500 keV region. Possible future measurements are described.

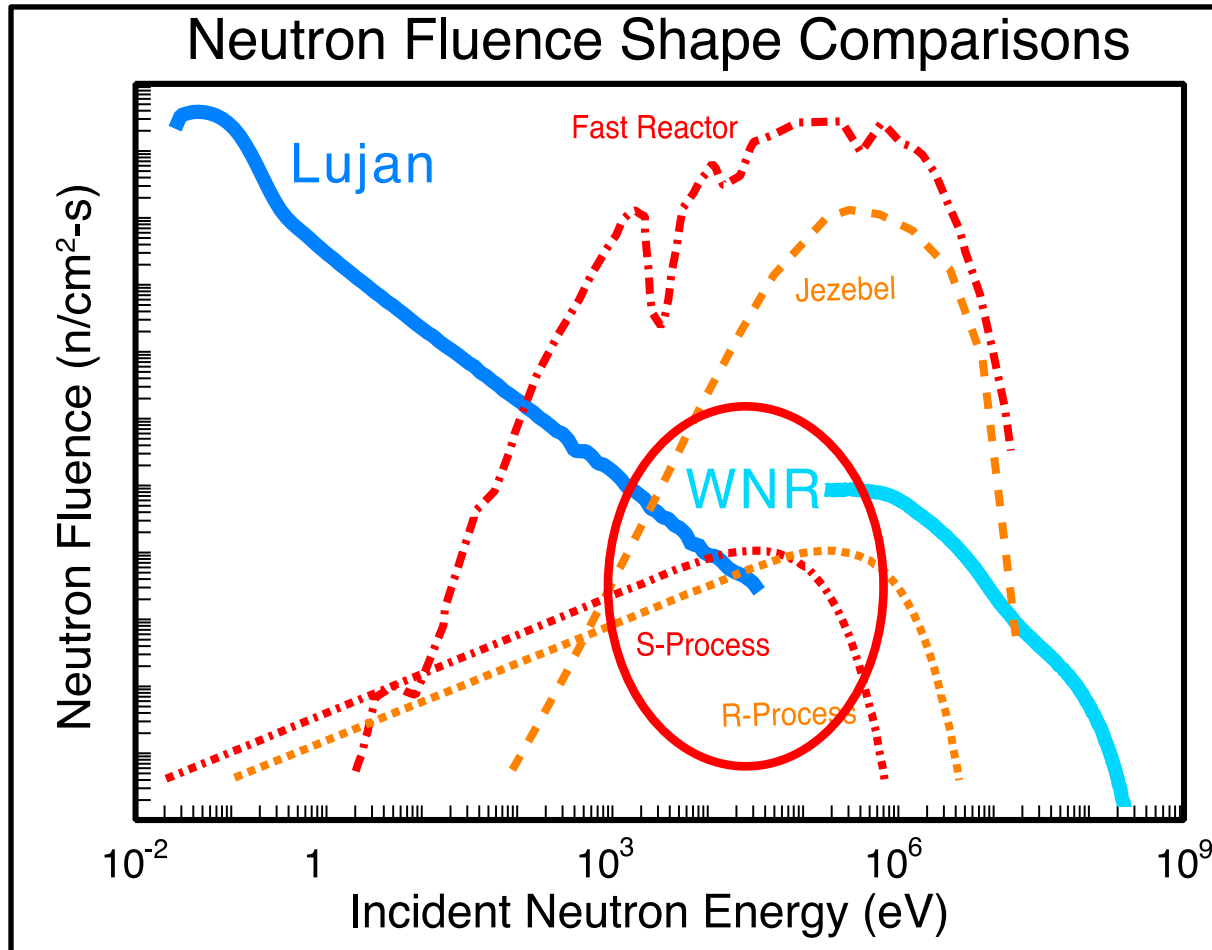
✦The Los Alamos Neutron Science Center



The issue:

- DOE/Office of Science pulled out of Lujan materials research
- Opportunity for LANL to rethink Lujan nuclear science, with a new target

Gaps in our understanding of intermediate energy nuclear reactions (~1 keV – 500 keV)



Redesign spallation moderation target to increase fluence in intermediate energy region

Optimization of a New Target at Lujan for Nuclear Science

- An opportunity now exists to optimize the present Lujan Center neutron spectrum to better cover the important intermediate neutron energy range between 100 eV to 2 MeV.
- Optimizations include:
 - Installation of a faster moderator which will enhance the neutron flux and energy resolution in this intermediate energy region
 - Changes to the pulse structure of the proton beam which includes producing a narrower proton pulse for better energy resolution and increasing the pulse repetition rate
 - Developing pulse stacking in the Proton Storage ring to increase the proton current. Initially, such a pulse-stacked pulse may be approximately 30 ns wide separated by 25 ms. If we store 4 pulses in the ring, the intensity will be approximately 95 uA with a pulse repetition rate of 160 Hz.

Objectives

Improved nuclear data for intermediate energy 1 keV – 500 keV neutrons, for:

- higher-fidelity neutronics simulations
- astrophysical applications
- fast reactor data needs

Historically we model intermediate energy criticality benchmarks more poorly than simple fast benchmarks

- They involve more scattering, a more complex transport, and are more sensitive to inelastic, elastic, reactions
- Neutron-incident reactions in the 1-500 keV region are often less well understood, e.g. actinide capture reactions, inelastic scattering
- Reaction rates (fission, n2n, ...) modeled more poorly here too

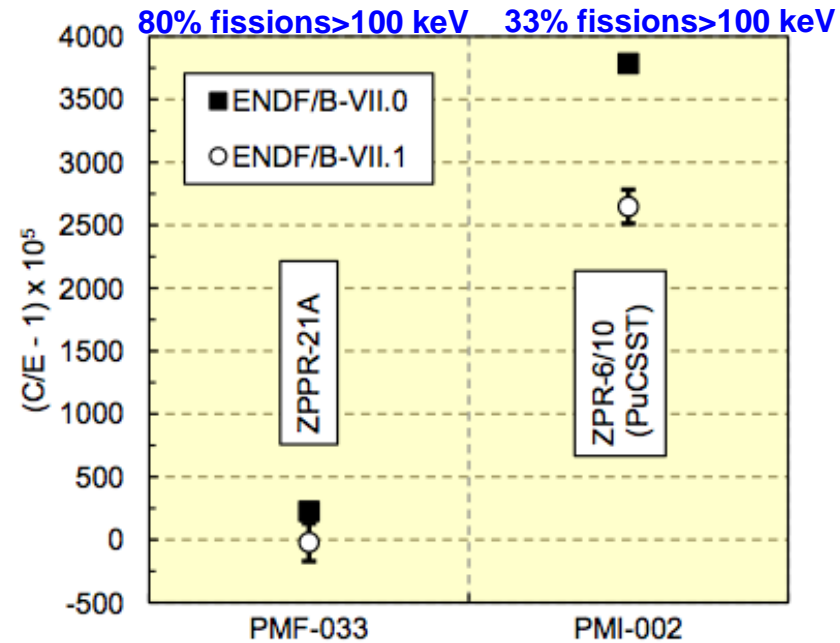


FIG. 24: MCNP Calculations with As-Built models for Pu metal FAST and INTER ZPR/ZPPR Assemblies.

ENDF performs less well in intermediate energy spectra than in fast spectra, for reaction rates Flattop critical assembly

⇒ Fission reaction rates, including threshold fissioners, measured by LANL radiochemists

TABLE XVI: Measured and Calculated Fission Rate Ratios for Selected Actinides in Flattop-25 by Barr *et al.* [15]. Data for the uranium isotopes and ^{239}Pu are ratioed to $^{235}\text{U}(n,f)$, the remaining results are ratioed to $^{239}\text{Pu}(n,f)$. The measurement location for those data given in the top half of the Table are near the center of the assembly ($r=1.11$ cm), data given in the bottom half of the Table are from the tamper region ($r=13.97$ cm). As these data have not been published previously, we also include the measured spectral indices in the second column of this Table. A generic 5% uncertainty is judged appropriate for these data, but the values tabulated are given to the precision used in internal LANL documents.

Reaction	Measured Spectral Index	ENDF/B-VI.0 C/E	ENDF/B-VII.1 C/E
$^{235}\text{U}(n,f)$	0.3155	0.921(46)	0.922(46)
$^{237}\text{U}(n,f)$	0.537	0.832(42)	0.892(45)
$^{238}\text{U}(n,f)$	0.1397	1.029(51)	1.030(51)
$^{239}\text{Pu}(n,f)$	1.307	1.039(52)	1.039(52)
$^{238}\text{Pu}(n,f)$	1.002	0.967(48)	0.950(47)
$^{240}\text{Pu}(n,f)$	0.549	1.043(52)	1.026(51)
$^{241}\text{Pu}(n,f)$	1.073	0.911(46)	0.911(46)
$^{242}\text{Pu}(n,f)$	0.482	0.961(48)	0.984(49)
$^{241}\text{Am}(n,f)$	0.577	0.918(46)	0.914(46)
$^{235}\text{U}(n,f)$	0.08	0.669(33)	0.672(34)
$^{237}\text{U}(n,f)$	0.391	1.018(51)	0.973(49)
$^{238}\text{U}(n,f)$	0.02487	0.832(42)	0.832(42)
$^{239}\text{Pu}(n,f)$	1.145	0.985(49)	0.985(49)
$^{238}\text{Pu}(n,f)$	0.708	0.968(48)	0.946(47)
$^{240}\text{Pu}(n,f)$	0.26	0.899(45)	0.870(43)
$^{241}\text{Pu}(n,f)$	1.251	0.954(48)	0.953(48)
$^{242}\text{Pu}(n,f)$	0.19	0.845(42)	0.871(44)
$^{241}\text{Am}(n,f)$	0.184	0.793(40)	0.784(39)

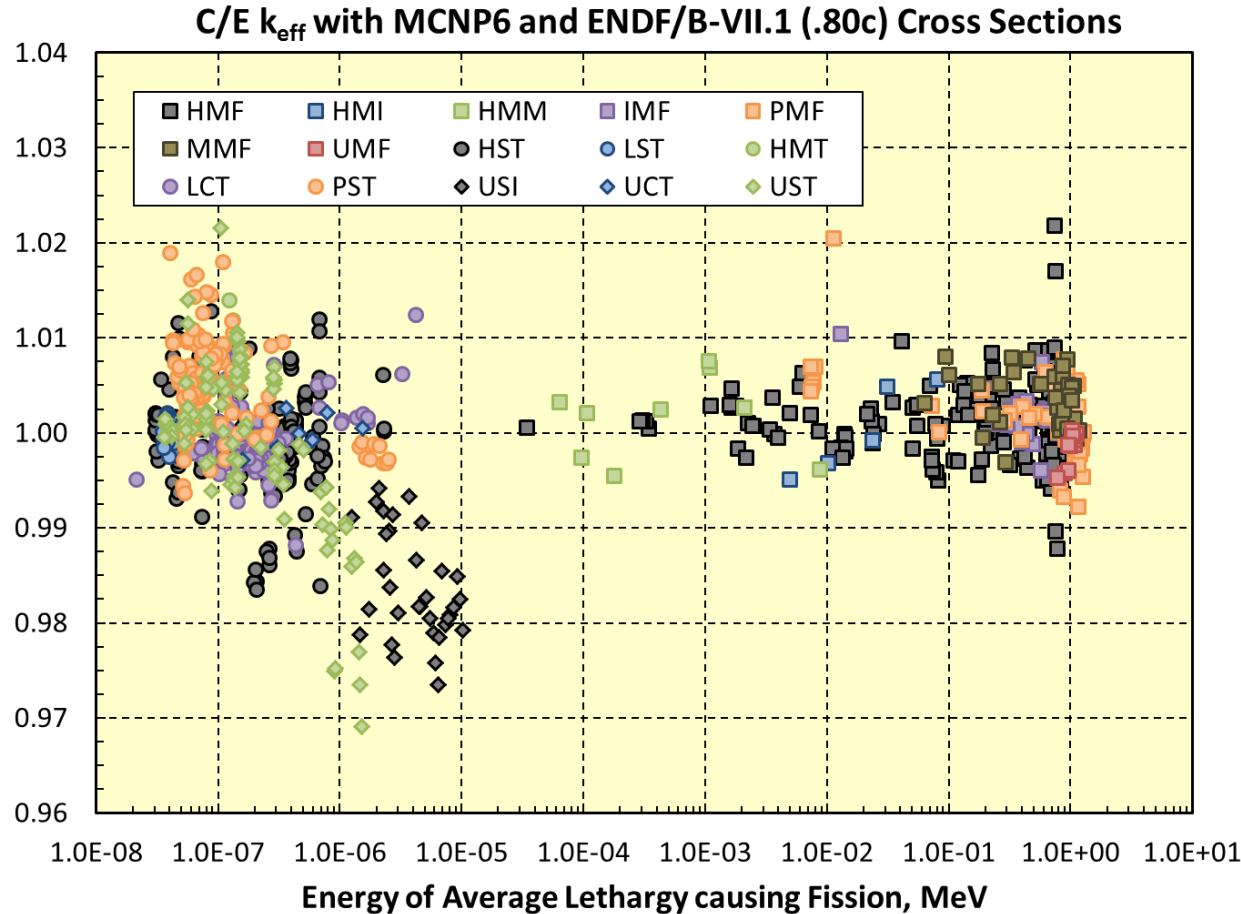
⇒ Fast discrepancy ~ 6%

Center region
(fast)

⇒ Intermediate discrepancy ~ 12%

Outer region
(intermediate)

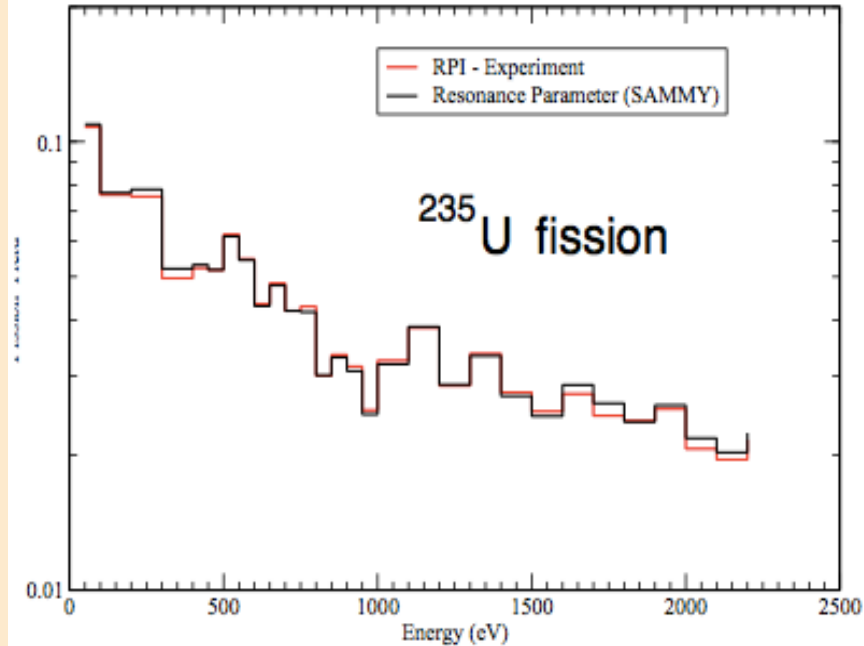
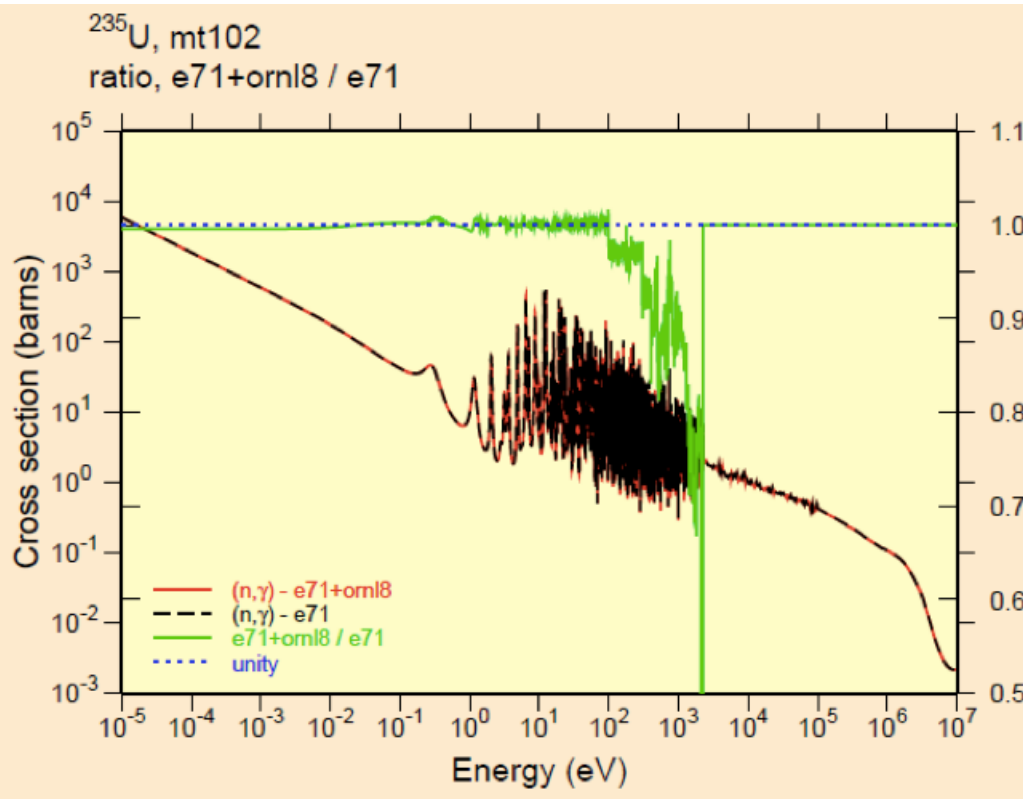
Survey of ICSBEP eigenvalue calculations (Kahler, LANL)



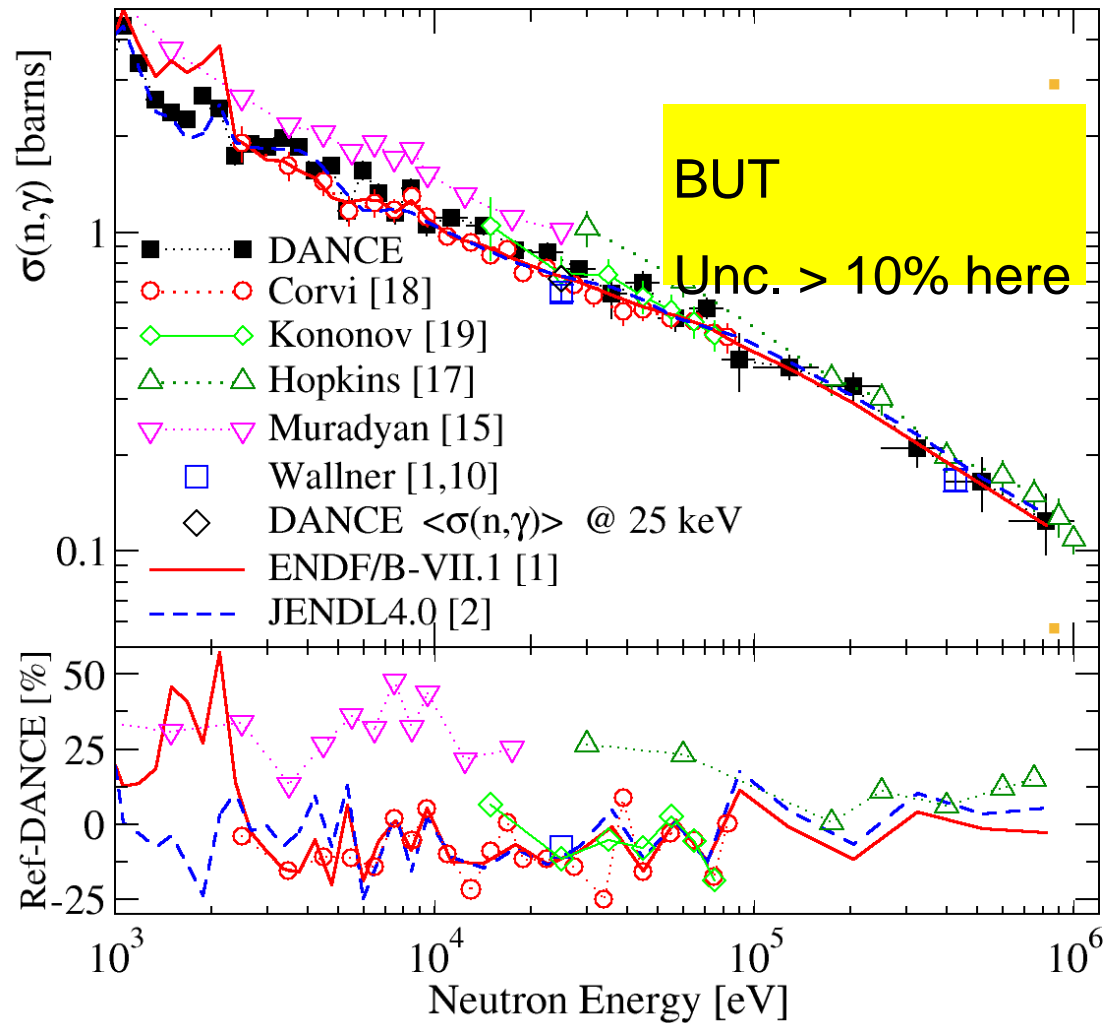
Energy of Average Lethargy values below $1.0\text{E}-6$ are largely “THERM” systems; values above $1.0\text{E}-1$ are largely “FAST” systems.

Fewer data exist the “INTERmediate” energy range.

^{235}U capture: DANCE & RPI data solved the 0.5-2.5 MeV region questions (But questions above 2.5 keV still)



^{235}U capture: we need more accurate data in the 2.5 keV - MeV region



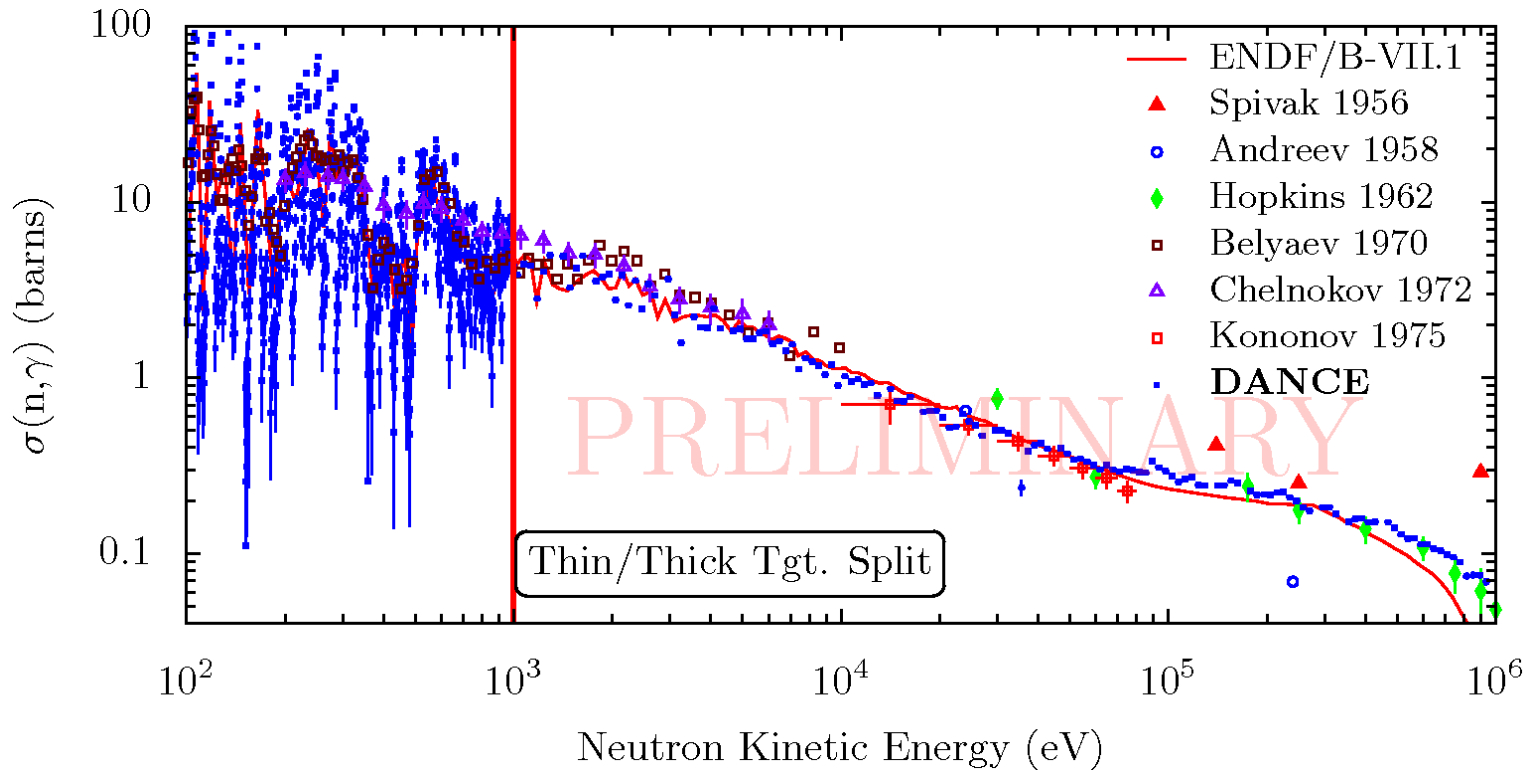
Jandel's ratio method helped

Precision <3% was achieved using simultaneous rate determination;

- Rates of U5(ng) and U5(nf)
- The same target \rightarrow same n flux for both reactions

Being implemented for ^{239}Pu (S. Mosby et al.)

New Plutonium-239 Capture Cross Section Data. But More Accurate Data are Needed > 1 keV

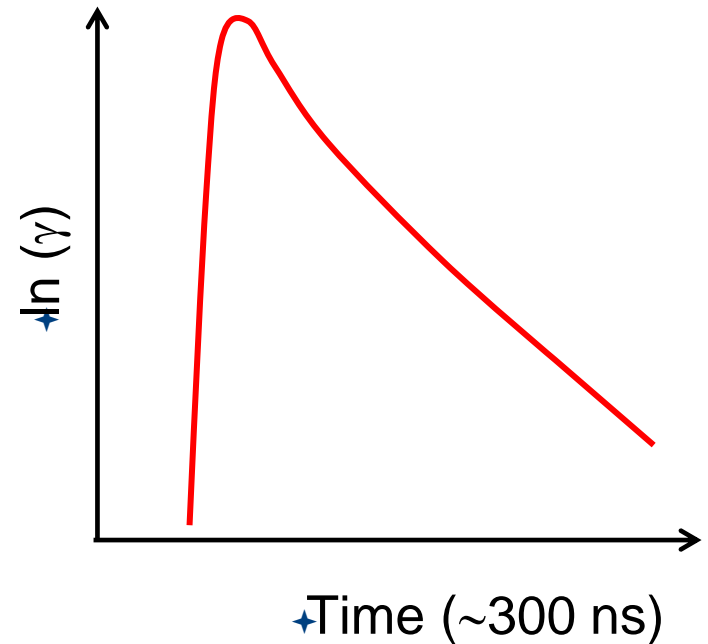


- Results up to 1 keV are published
- Experiment with thick target completed, analysis in final stages

Fission Decay Chain Measurements Motivate Prompt Fission Gamma-Ray Data at LANSCE/Lujan

Traditional approaches

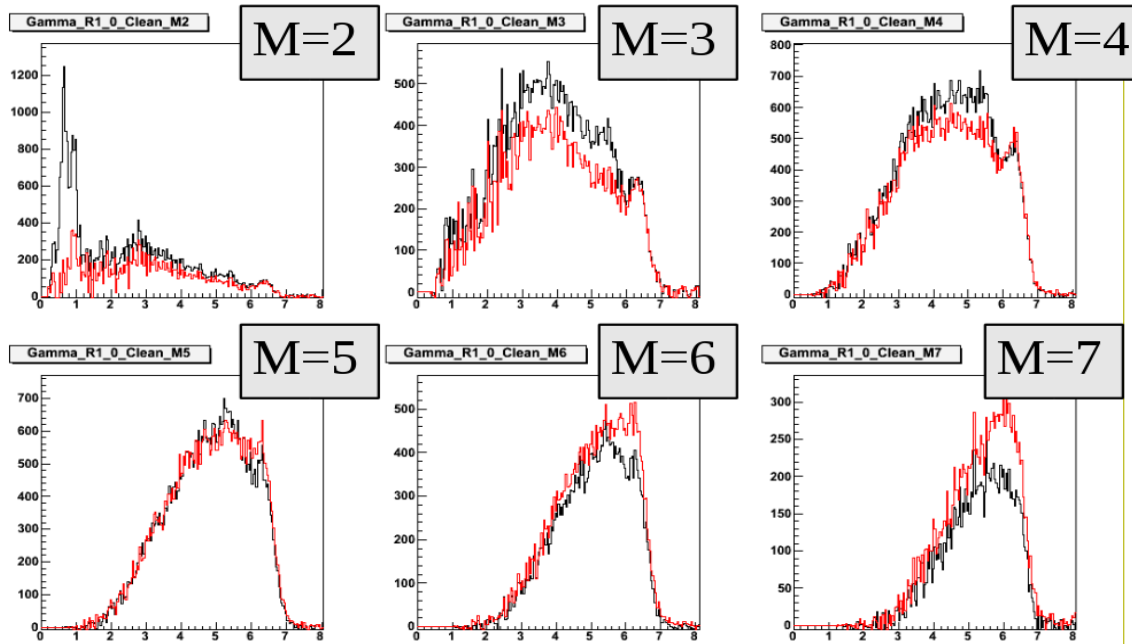
- 1/m plots (count rate v. control rod position, to identify asymptote & critical)
- Feynman variance of counts (doubles ...) to infer multiplication and k-eff
- Decay of fission chain via fission-gamma-rays



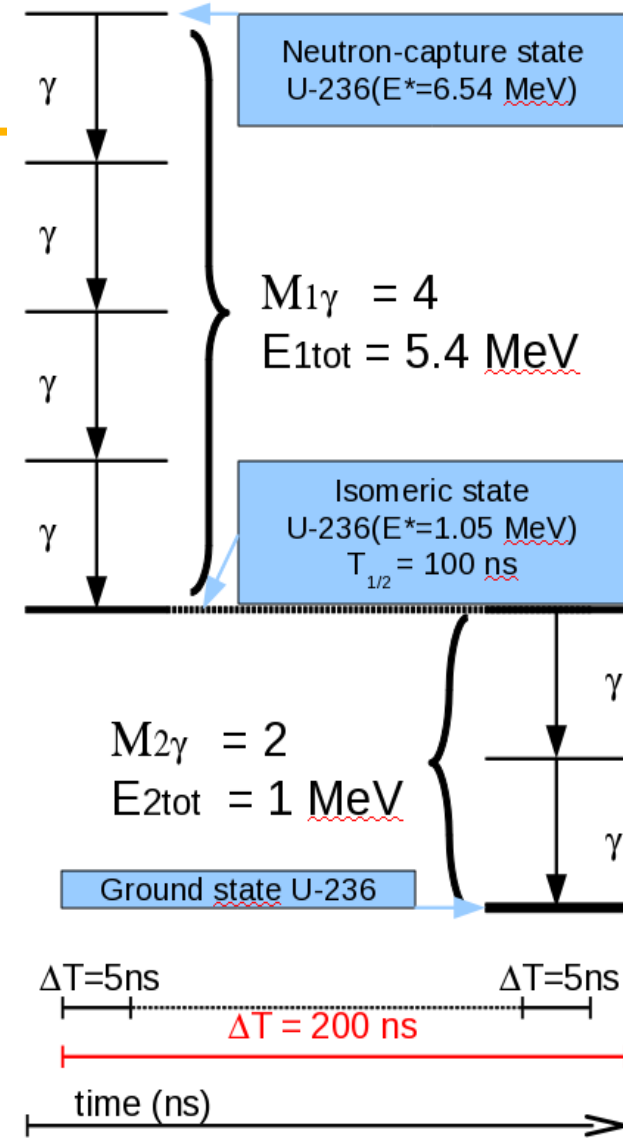
Short-Lived Isomeric states after U235+n

- During analysis of $^{235}\text{U}(n, \square)$ cross section we have found structure in the total gamma-ray energy E_{tot} spectra

M. Jandel et al., Phys Rev Lett 109, (2012)



E^*

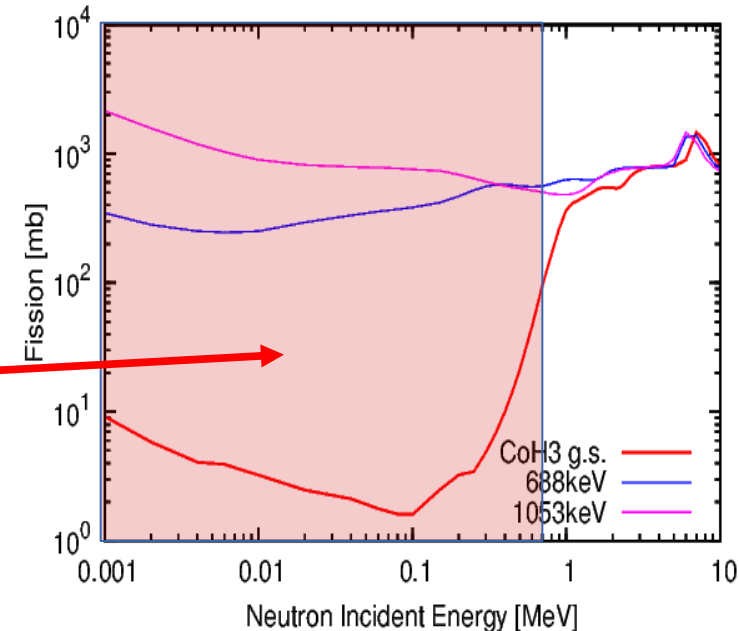
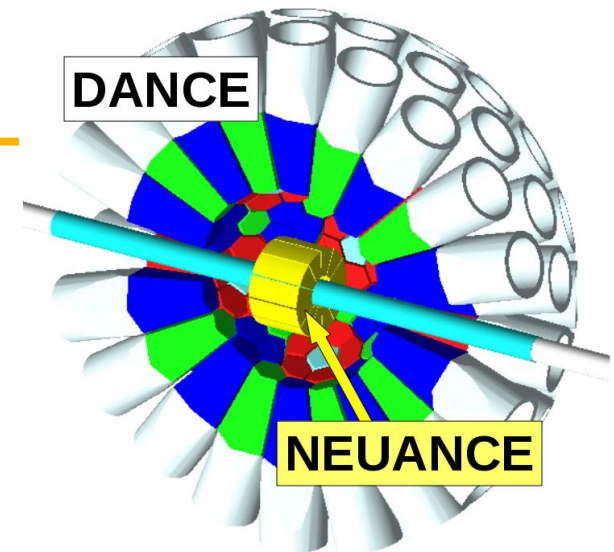


Isomeric states after U-235+n

- In high neutron fluence the secondary reactions can occur
- $^{236}\text{U}^*$: 1024 keV (4-) $T_{1/2} = 100$ ns
- $^{236}\text{U}^*$: 678 keV (1-) $T_{1/2} = 3.7$ ns
- Current work addresses resonance region

- What is the population of these states after $^{235}\text{U}+n$?
- What are the n-reaction cross sections on these states ?

A. Future – unresolved region $E_n > 1\text{keV}$



Conclusions

Future upgrades at Lujan are planned to address 1-500 keV advances:

- Precise capture & fission measurements
- Other reactions – inelastic, elastic scattering, e.g. with RPI staff
- Prompt fission gamma-ray data, isomers