Current activities and future plans for nuclear data measurements at J-PARC

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Content

- Introduction
- Our Facility, "ANNRI"
- Example of Measurements

 Experiment of ²⁴⁴ and ²⁴⁶Cm
 (MAs)
 - Experiment of ¹¹²Sn
 (stable isotope)
- Summary

Introduction(1) ~Motivation~

- To construct new types of reactors,
- To reduce High-Level radioactive Waste (Transmutation),
- To evaluate source terms from the Fukushima reactors, Neutron Capture Cross Sections of
 - minor actinides (MAs) and
 - long-lived fission products (LLFPs)
 - are very important.



However, Experimental data of MAs and LLFPs is **not sufficient** both in quality and in quantity. For example.....

Introduction(2)

Present status of nuclear data (1)



Resonance Region : Cross sections show sharp peaks and valleys due to resonances. **Unresolved Resonance Region** : At higher energies, the resonances get so close together that they can no longer be resolved into separate lines.

Introduction(3)

Present status of nuclear data (2)



Introduction(3)

Present status of nuclear data (2)



Introduction(4)

Present status of experimental data for MAs and LLFPs is not sufficient both in quality and in quantity.

This is because it is not easy to prepare enough amount of sample with a high purity.

Moreover, some MAs are high radio activities.

To overcome these problems

- 1) Intense neutron source
 - \rightarrow Small amount of sample can be used
 - \rightarrow Influence due to decay γ -rays can be reduced.
- 2) High energy resolution and high efficiency g-ray detector system was applied to the TOF measurement (For LLFPs)
 - \rightarrow Background due to impurities can be removed.



Red: Already Published. Green: Published with preliminary results. Blue: Already Measured. Black: Future Plan in few years (got Budget).

Introduction(6)

In this talk, as examples of our experiments, measurements for

- Cm-244, Cm-246, (MAs)
- Sn-112 (Stable) will be demonstrated.

Cm-244,246

A. Kimura, et. al. , J,NST,49,(7),708 (2012)

Sn-112

A. Kimura, et al., Nuclear Data Sheets., 119, (2014) 150.

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Our Facility, "ANNRI" ~ J-PARC~



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Our Facility, "ANNRI" ~ J-PARC~



BL04: ANNRI

(Accurate Neutron-Nucleus Reaction measurement Instrument)



top view and side view of the ANNRI.



top view and side view of the ANNRI.





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Our Facility, "ANNRI" ~Detector system~



Our spectrometer has

- 2 cluster-Ge detectors
 - (7 Ge crystals are installed in the detector)
- 8 coaxial-Ge detectors
- Compton suppressing BGO detectors ⇒22 Ge Crystals.

Energy resolution for 1.33MeV γ-rays:

5.8keV (for 200kevents/s),

2.4keV (for 20kevents/s) [1]

Peak efficiency for 1.33MeV γ-rays:

 $3.64 \pm 0.11 \%$

[1] T. Kin et. al., the 2009 NSS-MIC Conf. Rec., N24-2, (2009).

~ Nal(Tl) detectors~



Nal scintillator
 plastic scintillator
 photo-multiplier tube
 lead
 lithium hydride
 borated polyethylene
 borated rubber

cadmium sheet

The Nal spectrometer consists of two Nal detectors. Size: $330 \text{ mm}\phi$ 203 mmL 90° 203 mm ϕ 203 mmL 125°

Surrounded by plastic scintillator for cosmic-ray and shields.

The measurement with the NaI(TI) has two objectives.

- To perform the complementary measurement. By comparing the results for the same sample, the validation can be performed.
- The other is for measurements of the fast-neutron capture reaction.

The upper limit was 100~keV

Our Facility, "ANNRI" ~ Beam Intensity ~



By Dr. Kino

In the epithermal energy region, the neutron intensity is more than 10 times as high as the values of the other instruments.

Moreover, under the future 1-MW operation, these intensities are expected to increase.

K. Kino, *et. Al.*, NIM-A, **626**, 58 -66 (2011).

energy-integrated intensities	1MW (Future)	300kW		
1.5-25 meV	3.7 × 10 ⁷ n/s/cm ²	$1.1 \times 10^7 \text{ n/s/cm}^2$		
0.9-1.1 keV	$1.0 \times 10^{6} \text{ n/s/cm}^{2}$	$2.9 \times 10^5 \text{ n/s/cm}^2$		

However, in J-PARC, pulsed protons usually consist of two bunches (called "double-bunch mode"), each with a width of 60 ns, at intervals of 600 ns



Most users in MLF are users of diffractometers, scattering spectrometers and reflectometers. They require "neutron intensity".

~ "Double Bunched" Resolution Function~



At the low neutron energy (a), the time structures of the single and double bunches are almost same. However, the time structure is affected by the double bunch as the neutron energy increases (b, c).



Two-dimensional plots as a function of the time and energy of neutrons at the moderator surface for single bunch (a) and double bunch (b) modes.

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Regulations

Furthermore, because of the regulations of MLF,

- Un-sealed samples (Fission chambers are recognized as unsealed by radiation safety team.)
- Pu and U
- can not be handled.
- -> To do fission experiments are very difficult!! (Is there a merit to do fission experiments in ANNRI?)

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Experiments of ^{244,246}Cm ~Samples and Measurement conditions~



Outside 9mmФ 1.5mmt

Inside 5mmΦ 0.5mmt

Table 1 The isotopic composition of the244Cm sample or the 246Cm sample.[1]

	²⁴⁴ Cm sample	²⁴⁶ Cm sample
	TIMS (mole%)	TIMS (mole%)
²⁴⁴ Cm	90.1±1.7	27.5±0.5
²⁴⁵ Cm	2.71±0.34	1.06±0.28
²⁴⁶ Cm	7.22±0.34	59.4±1.3
²⁴⁷ Cm	N.D.	2.9±0.4
²⁴⁸ Cm	N.D.	9.10±0.24

Samples:

Cm-244 ($T_{1/2}$ =18.1y: MA) Net weight = 0.6 mg Activity = 1.8 GBq Measurement Periods: 64 hours Cm-246 ($T_{1/2}$ =4753y: MA) Net weight = 2.1 mg Activity = 12.1 MBq (²⁴⁴Cm: 1.7GBq) Measurement Periods: 94 hours Both of the samples Chemical form = CmO₂ Container = Al capsule

For the background estimation, a dummy case (Al 278mg) and a blank sample was measured for done for 48 and 44 hours.

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To reduce air scattering, the air in the beam duct was replaced with helium.

Experiments of ^{244,246}Cm ~TOF Spectrum ~



This graph shows TOF spectra of the ²⁴⁴Cm, the ²⁴⁶Cm sample, and the dummy case

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Counts/100ns

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Experiments of ^{244,246}Cm ~ γ-ray Spectrum at the 1st resonance of ²⁴⁴Cm ~

Cm-244



The252.4- and 380.8-keV γ -rays have already been studied in α decay of ²⁴⁹Cf, electron capture decay of ²⁴⁵Bk, and β -decay of ²⁴⁵Am.

The other γ -rays were previously unknown γ rays.

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Experiments of ^{244,246}Cm ~Analysis~

The data were analyzed with the procedure.

- Dead-Time Correction
- Background Estimation and Subtraction
- Self-Shielding and Multiple-Scattering Correction
- Normalization (Using the 1st resonance of ²⁴⁰Pu)
- Evaluation and Subtraction of Influence of Fission Events
- Evaluation and Subtraction of Influence of Impurities

The obtained neutron-capture cross sections are....

Experiments of ^{244,246}Cm ~Cross Section of ²⁴⁴Cm~

Only one neutron-capture cross-section data of ²⁴⁴Cm (n,g) was reported in 1969[1].



^[1]M. S. Moore et.al., Physical Review C, 3, 1656 (1971).

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Experiments of ^{244,246}Cm ~Cross Section of ²⁴⁶Cm~

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Experiments of ^{244,246}Cm ~Cross Section of ²⁴⁶Cm~

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~Experimental Set up For Sn-112~

	Chemical form	Diameter [mm]	Weight [mg]	Time [hr]	
¹¹² Sn	Metal	5	97.3	37.6	Beam Condition
²⁰⁸ Pb	Metal	5	159.7	16.3	Beam Power = 210kW
Blank	Hold	ler and films	only	22.7	Beam Size = 7mmφ Beam Duct = vacuum

Each sample was put in a FEP bag and attached to a PET sample holder.

- For the background estimation, a ²⁰⁸Pb sample (99.57%) and a blank sample were measured.
- For dead-time correction, pulses from a random-pulse generator were input to the pre-amplifier of every Ge crystal.

	Isotopic Distribution[%]									
	¹¹² Sn	¹¹⁴ Sn	¹¹⁵ Sn	¹¹⁶ Sn	¹¹⁷ Sn	¹¹⁸ Sn	¹¹⁹ Sn	¹²⁰ Sn	¹²² Sn	¹²⁴ Sn
¹¹² Sn	99.6	0.3	0.05	< 0.01	< 0.01	< 0.01	< 0.01	< 0.01	< 0.01	<0.01

Data processing and analysis

The data were analyzed with this procedure.

- Dead-Time Correction
- Background Estimation and Subtraction
- Self-Shielding and Multiple-Scattering Correction
- Normalization

For ¹¹²Sn : to thermal cross sections by Krane[1]

The obtained results of neutron-capture cross sections are....

~Cross Section of ¹¹²Sn~



- Resonances at 21, 46, and 251 eV were not observed,
- The resonance at 240 eV is listed in ENDF B-VII but not listed in JEBDL 4.0.

~Cross Section of ¹¹²Sn~



- ➡ Resonances at 21, 46, and 251 eV were not observed,
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~γ-ray Distributions of each resonance~



Many γ-rays are clearly observed.

Are these spectra important??

~γ-ray Distributions of each resonance~



Many γ-rays are clearly observed.

1284 (1284.06->g.s.)

The 6189- and 7242-keV γ -rays have been reported by C. Samour. 8 γ -rays have already been studied in other reactions. ((p,n γ), (p,3n γ), (α ,2n)) 16 or more previously unknown γ -rays are observed.







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Our sample is contaminated with Se(1.33%), Te(830ppm), and Ag(18ppm). However, these impurities are not listed on the certification sheet.

~Cross Section of ¹¹²Sn~

With subtracting the influences of the chemical and isotopic impurities, preliminary cross section for ¹¹²Sn was obtained.





The resonance at 236.6 eV is listed in ENDF B-VII.1 but not listed in JENDL 4.0.



The resonance at 236.6 eV is listed in ENDF B-VII.1 but not listed in JENDL 4.0. The 332-and 489-keV γ-rays are clearly observed in the 240-eV resonance.

The 236.6-eV resonance is one of the ¹¹²Sn resonances.





~Resonance energies for ¹¹²Sn~ ^{/2}

Table Resonance energies for ¹¹²Sn comparison to evaluated values.

This work	JENDL 4.0	ENDF/B VII.1	Confirm*	This work	JENDL 4.0	ENDF/B VII.1	Confirm*
Not Observed.	21.02	21.02		809.2±1.7			0
Not Observed.	40.38	40.38		879.3 ± 1.9	877	877	0
64.61 ± 0.06	64.66	64.66	0	903.3±2.0			× **
72.24 ± 0.08	72.26	72.26	0	1086.0 ± 2.5			0
94.76±0.11	94.8	94.8	0	1156.0 ± 2.8			0
104.65 ± 0.12	104.7	104.7	0	1321 ± 3	1321	1321	0
Not Observed.		166		1419 ± 4	1416	1416	0
236.6 ± 0.3		240	0	1511 ± 4			∆ ***
252.9 ± 0.4	251.1	251.1	0	1539 ± 4			Δ***
514.0 ± 0.9	514	514	0	1639 ± 5			0
577.3 ± 1.1	577	577	0	1687±5			× **
679.7±1.3			× **	1824 ± 5			0
715.6±1.4			0	1982 ± 6			0
771.3±1.6			0				

*: 332-and 498-keV γ-rays

**: The numbers of the events are not enough.

***: Overlapped resonances. (cannot separate clearly).

~Resonance energies for ¹¹²Sn~ ^{/2}

We found 3 miss assigned resonances and 13 new resonances.

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~Advantages of ANNRI~

High intensity pulsed neutron with High speed DAQ.

✓ A small amount (less than 1 mg) sample can be used.



~Advantages of ANNRI~

• High intensity pulsed neutron with High speed DAQ.

✓ A small amount (less than 1 mg) sample can be used.

• High γ-ray energy resolution with Ge detectors.

Prompt γ-ray spectra from resonances can be obtained.
 -> An origin of each resonance can be checked.
 (An impurity nuclide or the target nuclide)
 Spin assignments may be checked.

Sufficient beam time.

 In next 2013 JFY, ANNRI will be used in "user program" for 85 days.

Everybody can apply this program.

~Future plan~

- To measure neutron capture cross sections of ^{155,157}Gd, ¹¹⁶Sn, ^{133,135,137}Cs and so on.
 In order to deduce resonance parameters precisely,
- To prepare neutron total cross section measurements.
- To develop an analysis method from "doublebunched" results using the REFIT-code and the "double-bunched" resolution function.

We have built a new experimental instrument (ANNRI), and obtained preliminary neutron-capture cross sections of MAs and LLFPs.

•Neutron-capture cross sections are deduced using a small amount (less than 1 mg) of a high radioactive sample.

Even a stable isotope, there are many miss-assigned resonances in the evaluated values.
 However, using the array of Ge detectors at the ANNRI and by checking prompt γ-ray spectra, γ rays from impurities can be discriminated.

Thank you for your kind attention!!

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