



# Transmutation of Actinides in CANDU Reactors

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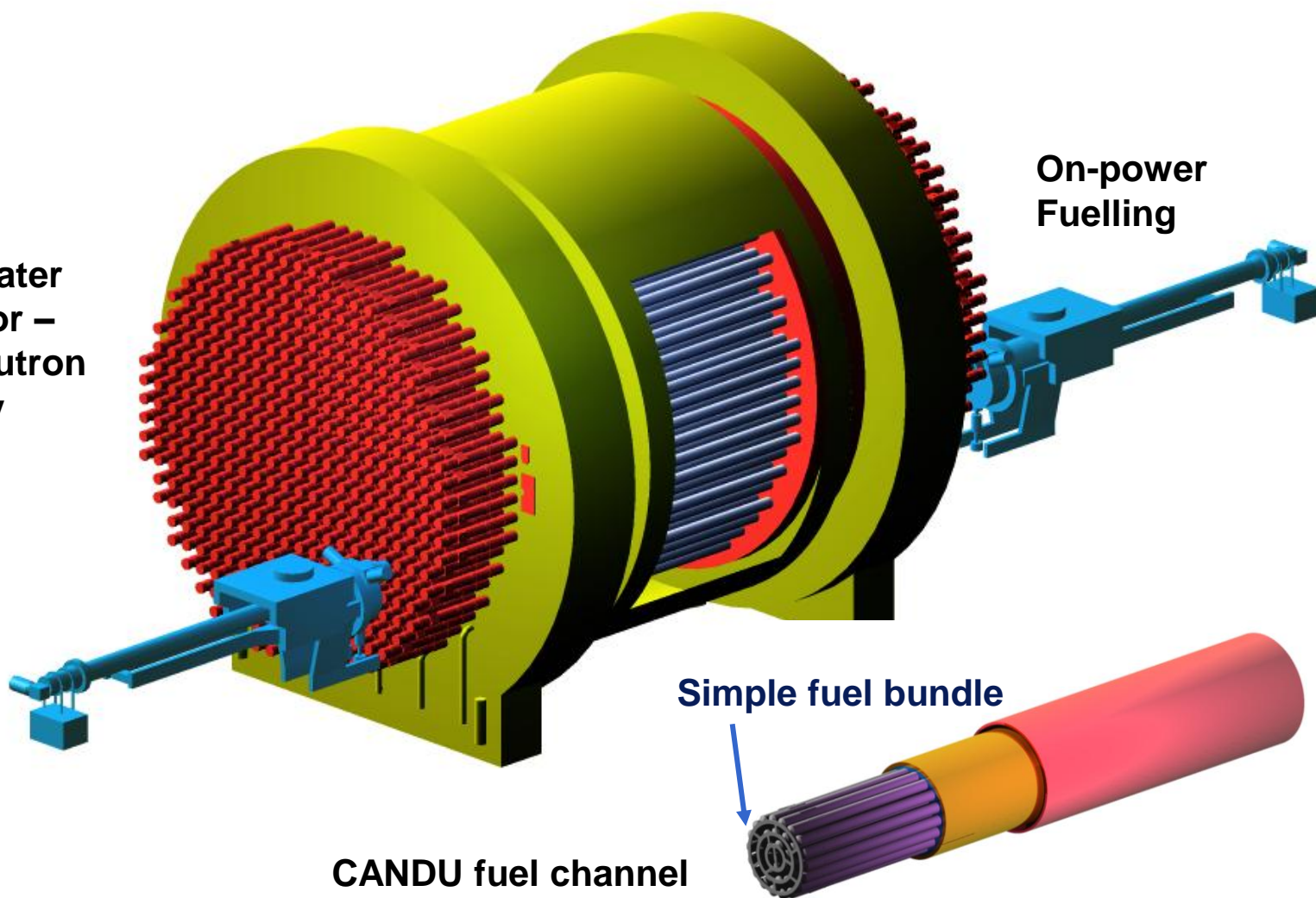
# Outline

- **Introduction to CANDU reactors**
- **Motivation for transmutation of actinides**
- **Transmutation of actinides in CANDU**
  - **Group-extracted TRU in MOX**
  - **Separated Am/Cm in targets**



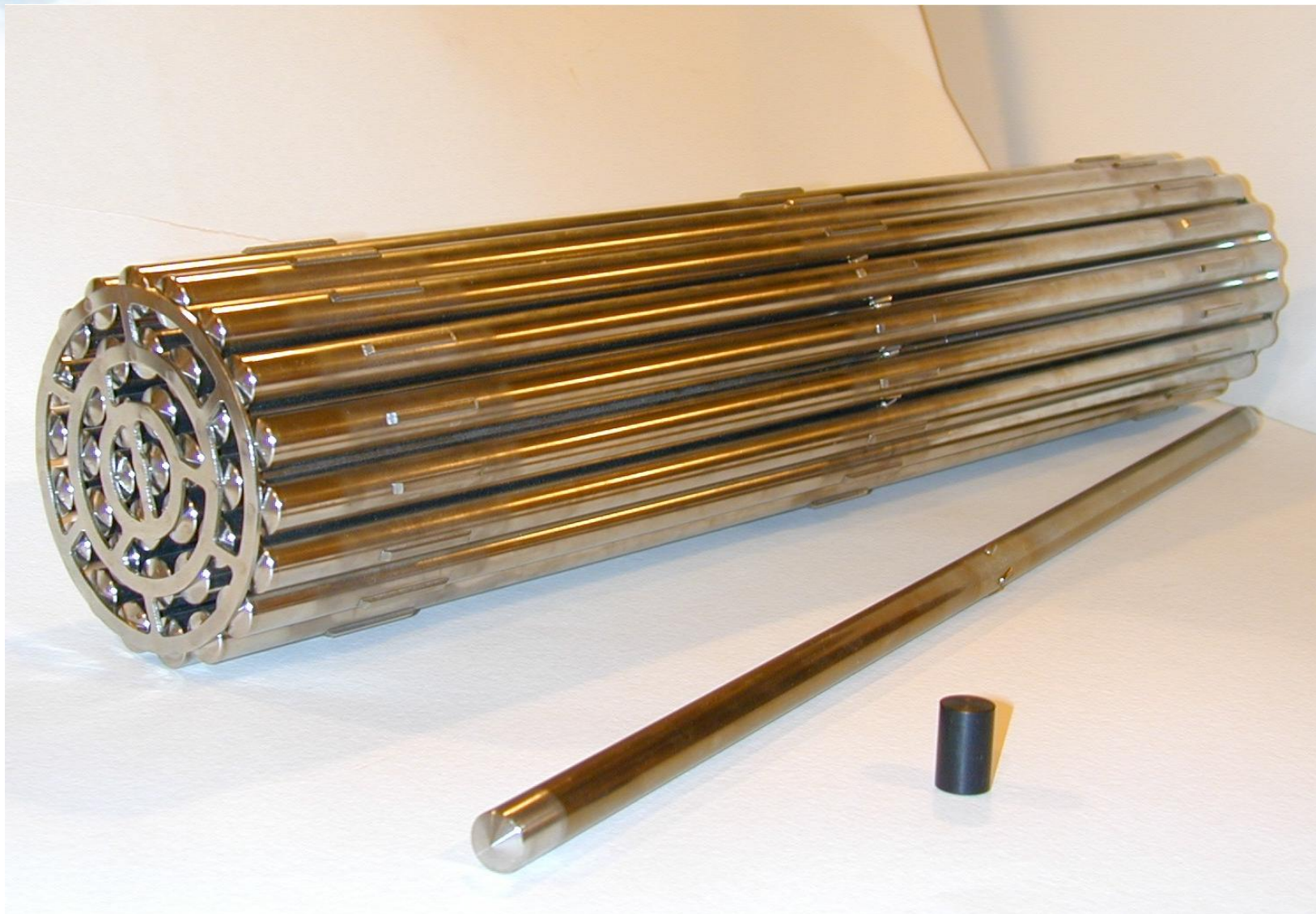
# The CANDU Reactor

Heavy Water Moderator –  
Good neutron economy





# 37-element bundle







# Motivation

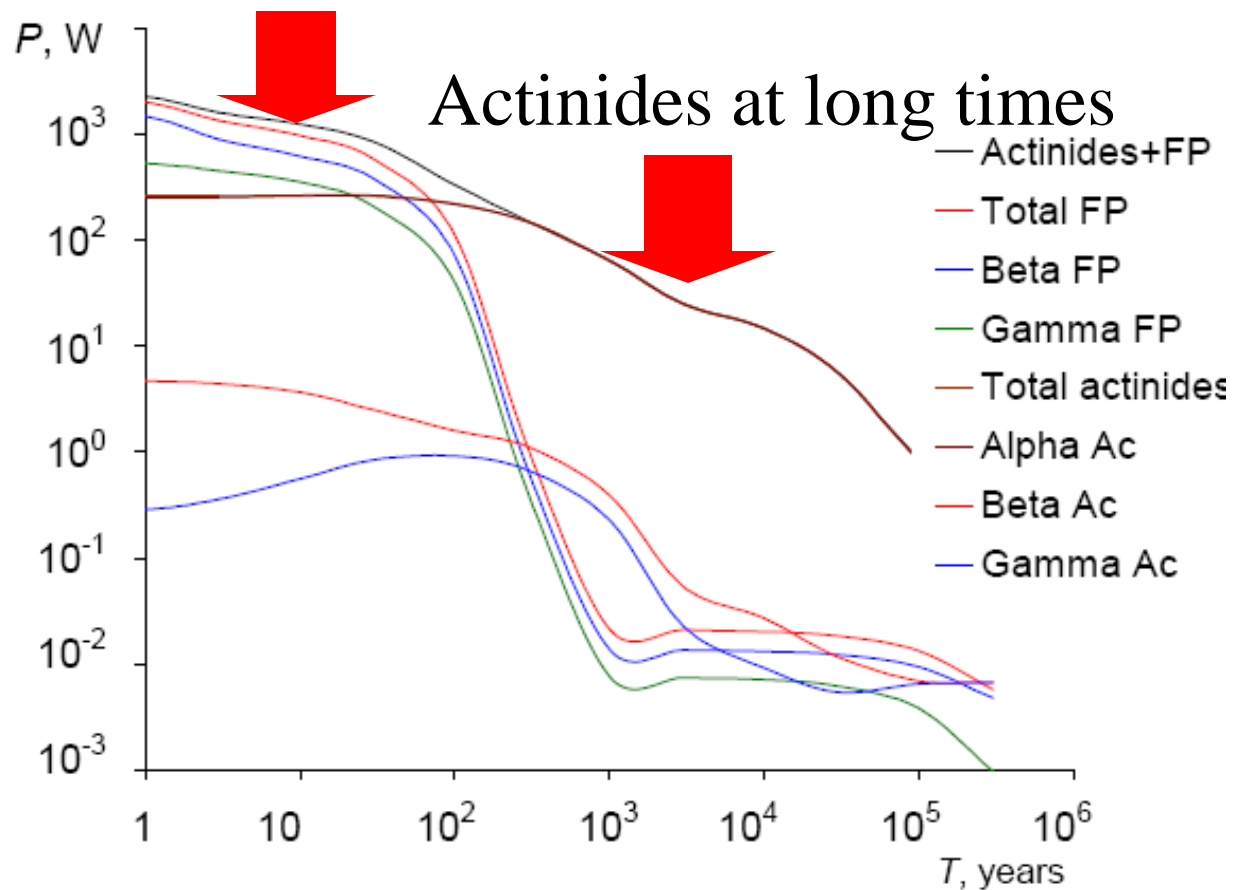
- Increase capacity of long-term geological disposal
- YM final, total cost: **\$96 billion**
- Technical capacity limited by decay heat load





# What's Contributing to the Heat Load?

FP's at short times



\*Data for Russian VVER

ICRS-10, 10-14 may 2004

**B.R. Bergelson, A.S. Gerasimov, and G.V. Tikhomirov**



# Transmutation Scenarios

- **Two transmutation scenarios were examined**
- **Group-extracted TRU in MOX**
- **Separated Am/Cm in targets**

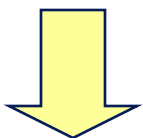


# TRU MOX Scenario

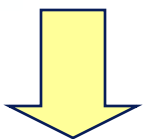
**LWR, 45 MWd/kg**



Cool 30 years



Group extraction



MOX

- WIMS-AECL lattice cell calculations

- RFSP full-core calculations

- 45 MWd/kg exit burnup for MOX





# 30 year cooled SNF

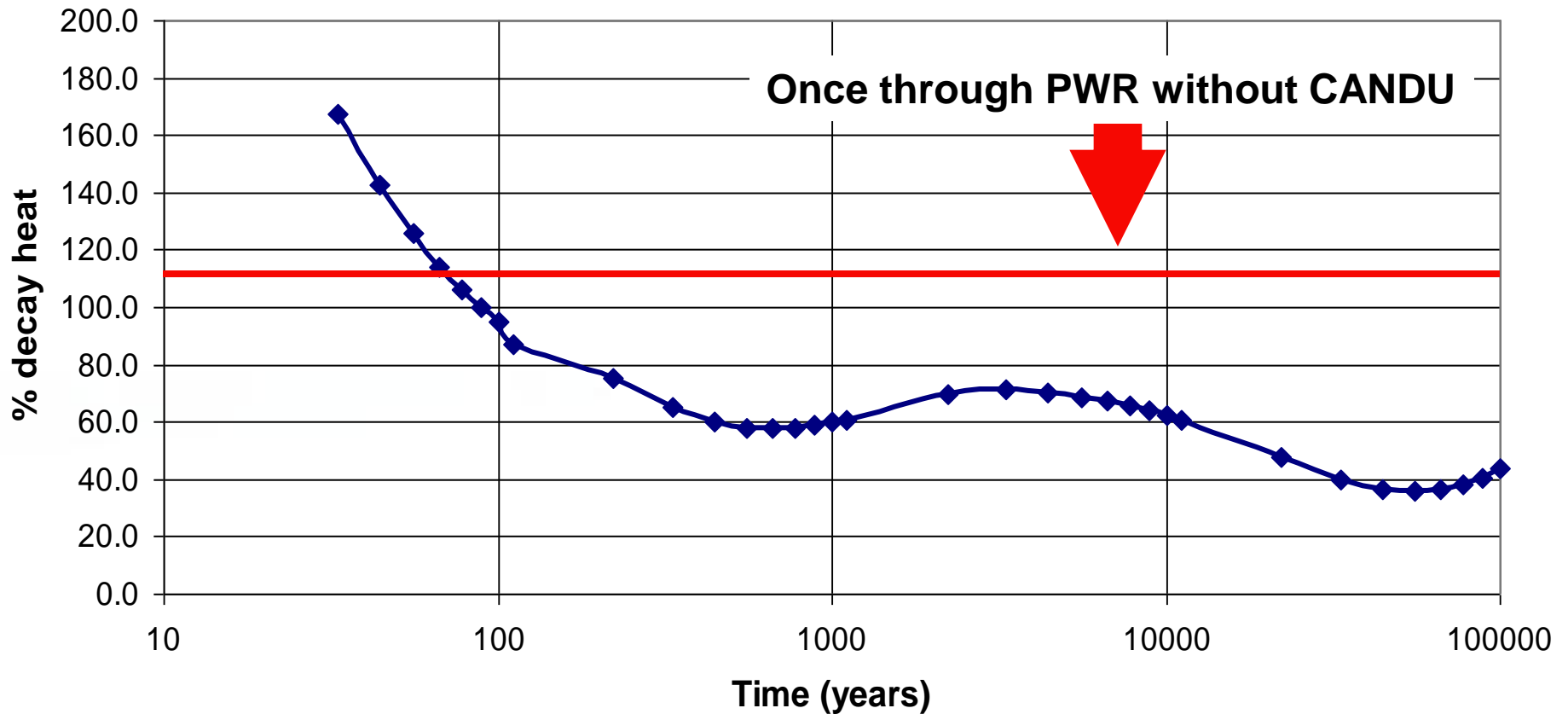
	<b>MOX</b>	<b>Initial TRU Composition, g/kg initial TRU</b>
<b>Initial TRU Content, g/bundle</b>	<b>653</b>	
<b>Initial TRU Content, % by volume</b>	<b>3.3%</b>	
<b>Pu-238</b>	<b>+163</b>	<b>13</b>
<b>Pu-239</b>	<b>-77</b>	<b>563</b>
<b>Pu-240</b>	<b>-1.8</b>	<b>201</b>
<b>Pu-241</b>	<b>+65</b>	<b>30</b>
<b>Pu-242</b>	<b>+176</b>	<b>38</b>
<b>Pu Total</b>	<b>-39</b>	<b>845</b>
<b>Np Total</b>	<b>-52</b>	<b>47</b>
<b>Am-241</b>	<b>-90</b>	<b>100</b>
<b>Am total</b>	<b>-64</b>	<b>108</b>
<b>Cm total</b>	<b>+3700</b>	<b>0.6</b>
<b>Total MA</b>	<b>-45</b>	<b>155</b>
<b>Total TRU</b>	<b>-40</b>	<b>1000</b>

Change in actinide composition (%) at discharge burnup



# Decay Heat from Actinides

MOX





# Nuclide Contribution to Heat Load

Nuclide	Time Frame of Main Contribution to Heat Load	% Difference
Pu-238	Less than 100 years	+163
Cm-244	Less than 100 years	+2641
Am-241	Less than 1000 years	-90
Pu-239	1000-100,000 years	-77
Pu-240	1000-100,000 years	-1.8



# Full-Core Calculation

Input fuel composition



Lattice cell calculation: WIMS



Cross-sections, Depletion



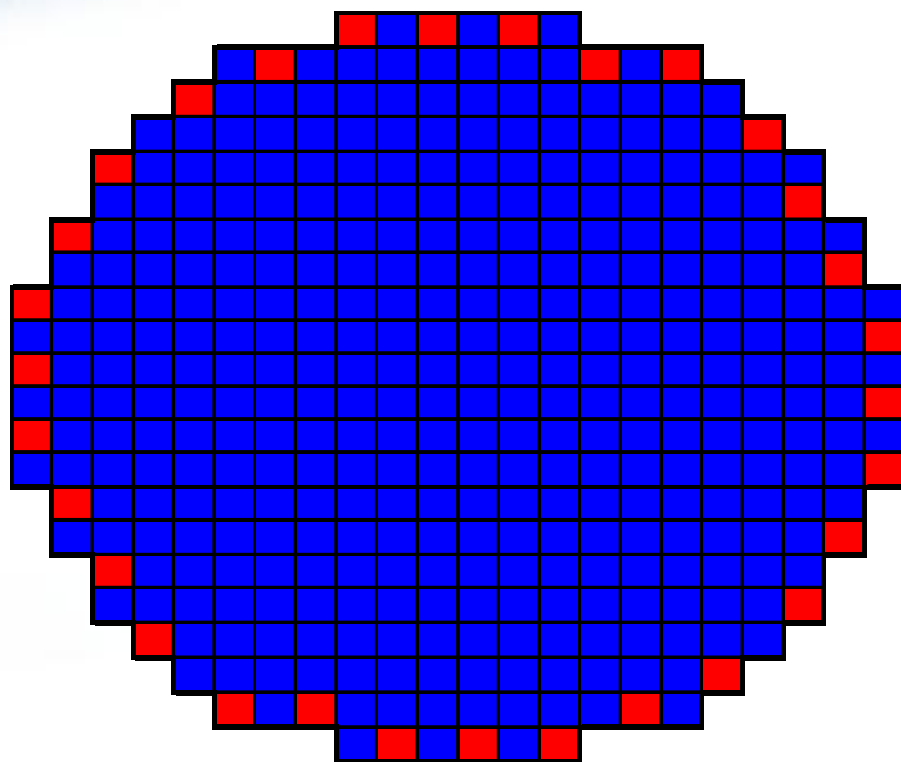
Full-Core Calculation: RFSP



Full-core Parameters, Dwell Time, Burnup



# Am and Cm Target Channels



- **30 target channels**

 **Am and Cm in IMF**

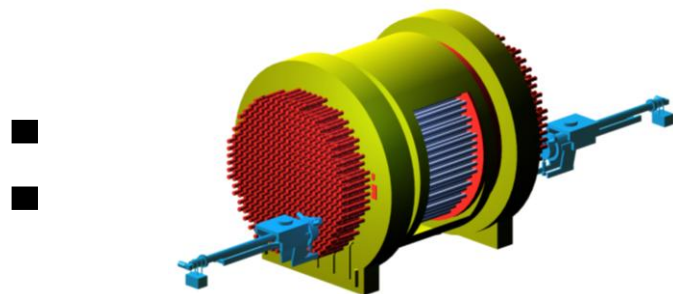
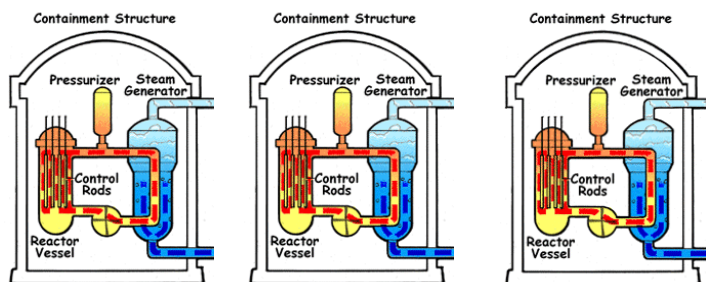
 **0.9% Fissile RU**





# Important Criteria

- **Support ratio**

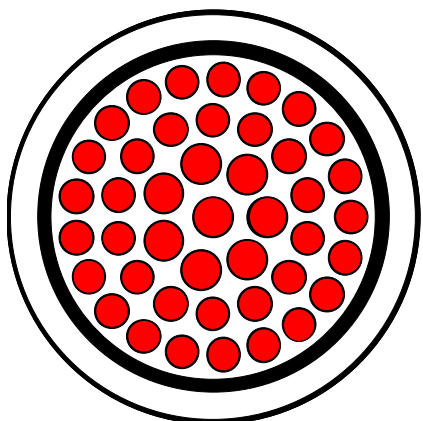


- **% transmutation**

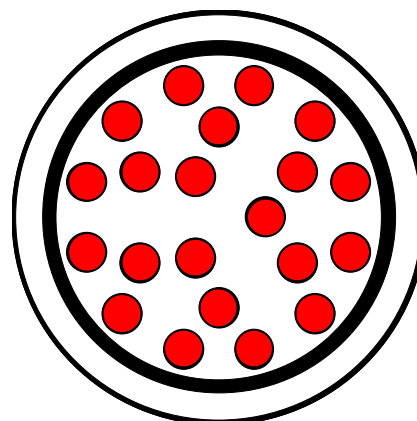
- **Residence time**



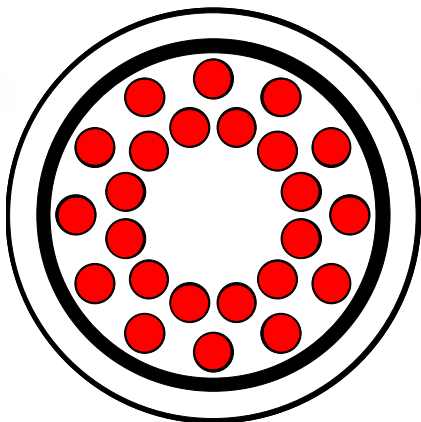
# Fuel Bundle Designs



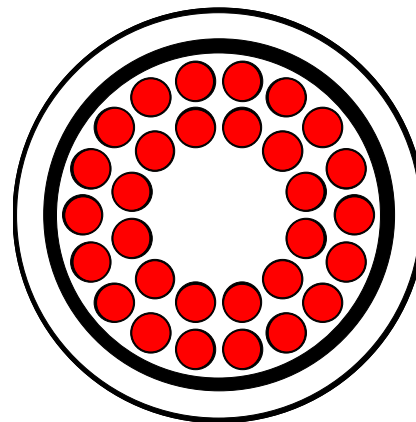
**CANFLEX 43 elements**



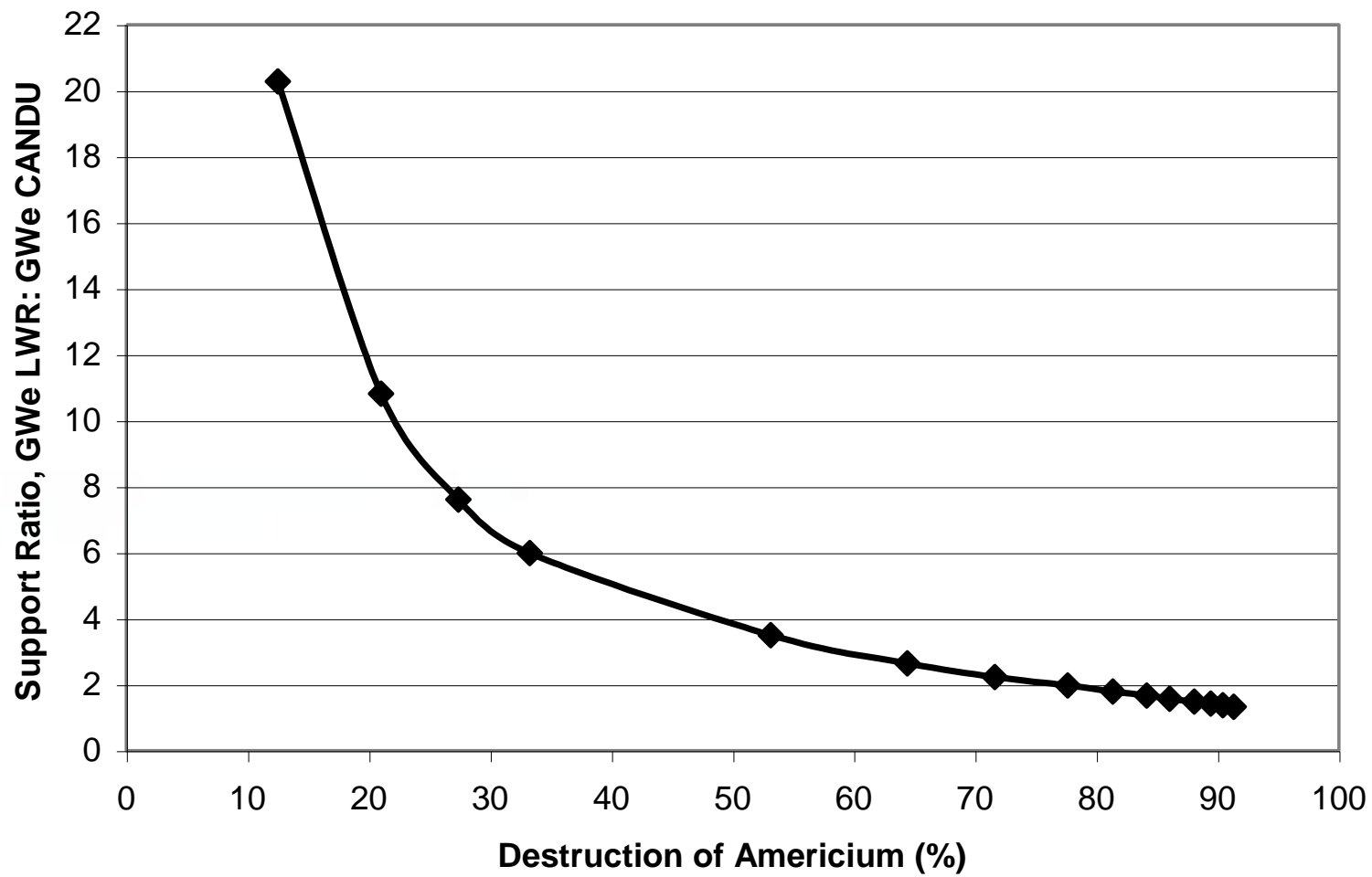
**21 elements**

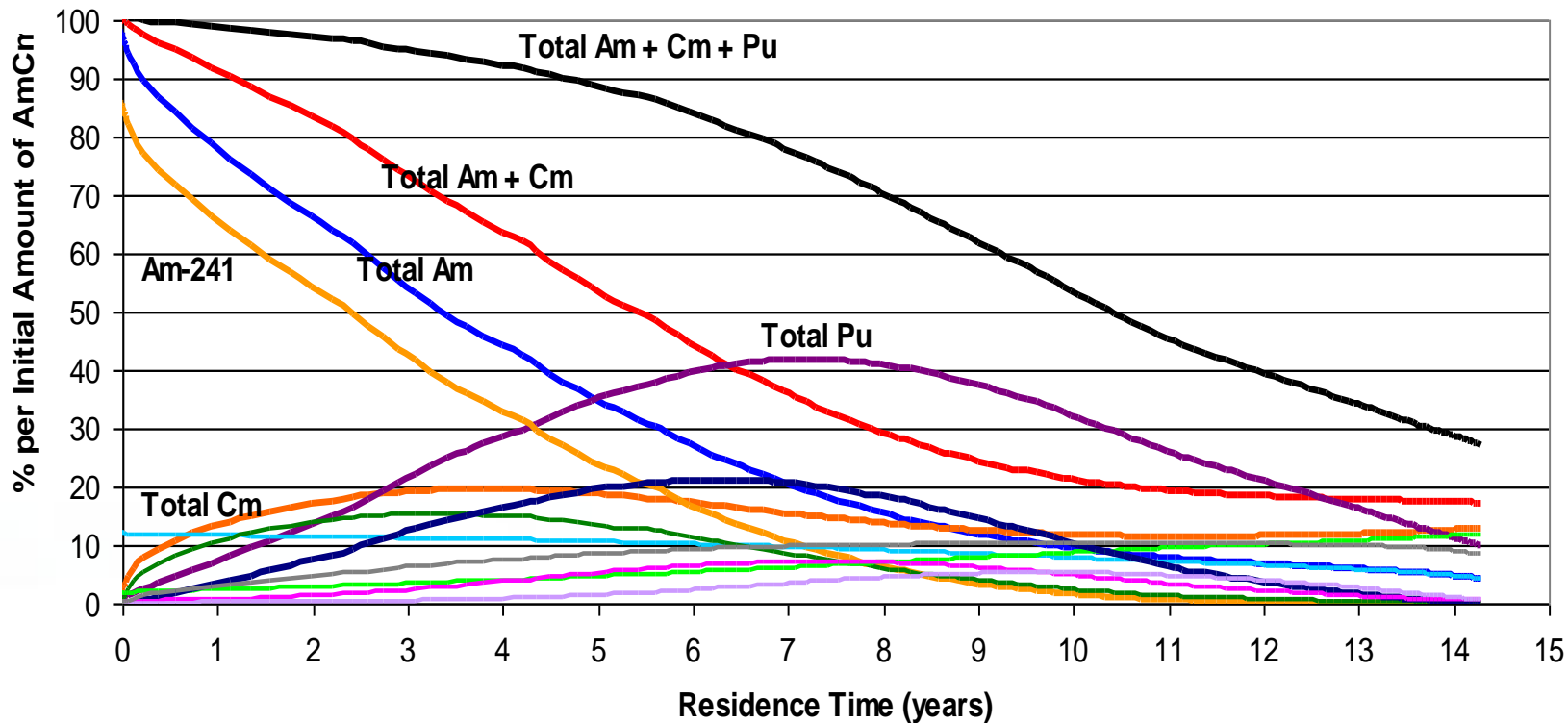


**24 elements**



**30 elements**







# Results

	<b>Input kg/CANDU</b>	<b>Exit kg/CANDU</b>	<b>% Change</b>
<b>Am</b>	<b>373</b>	<b>112</b>	<b>-70</b>
<b>Cm</b>	<b>9</b>	<b>68</b>	<b>+700</b>
<b>Total Am + Cm</b>	<b>382</b>	<b>180</b>	<b>-53</b>

- **21-element bundle**
- **26% initial concentration**
- **Support ratio 2.5 GWe LWR : 1 GWe CANDU**
- **Residence time for AmCm = 5.7 years**





# Full-Core Calculation

Input fuel composition



Lattice cell calculation: WIMS\*



Cross-sections, Depletion



Full-Core Calculation: RFSP



Full-core Parameters, Dwell Time, Burnup

\* Calculations done with a developmental version of WIMS-AECL



# Summary

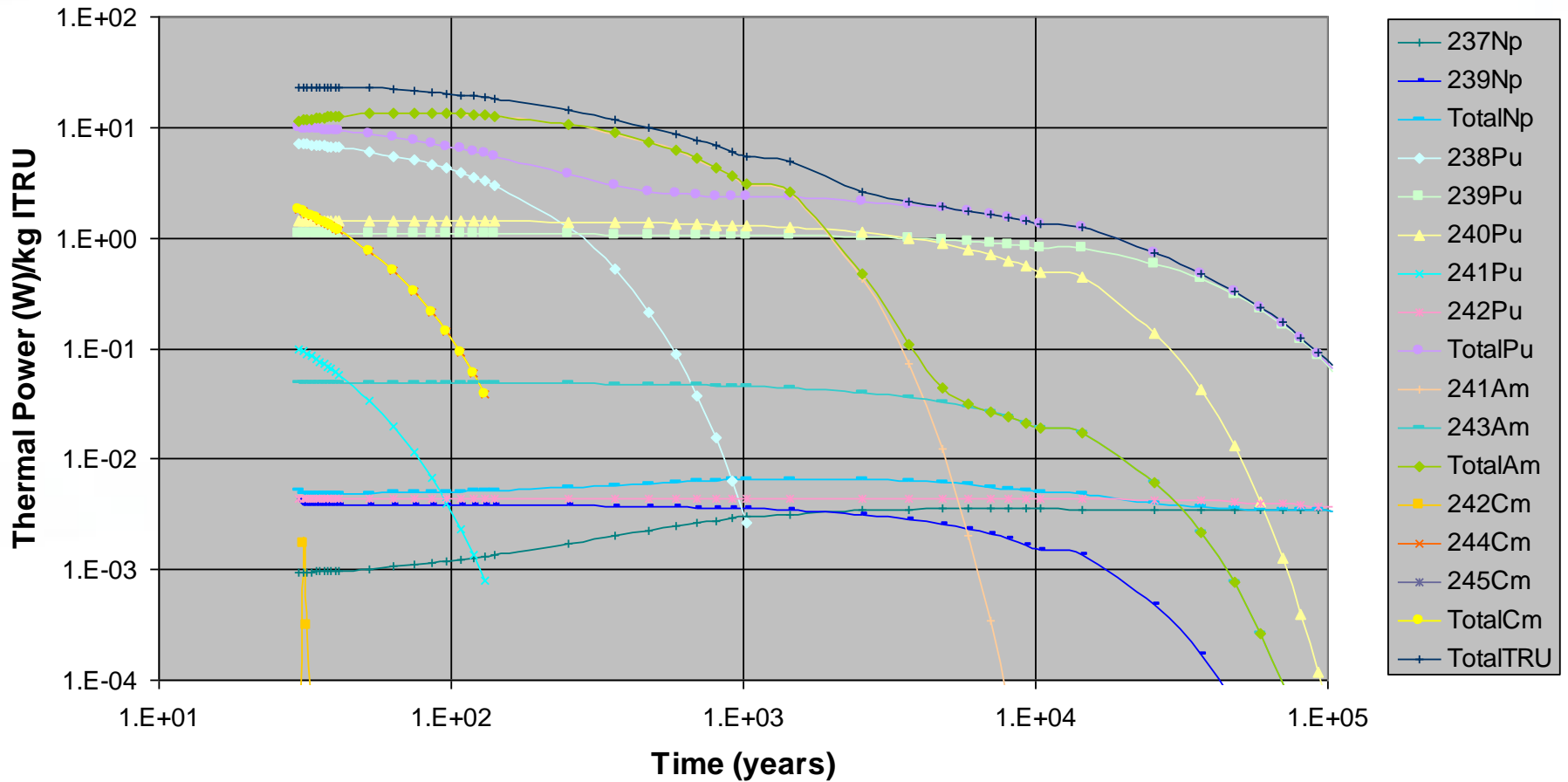
- **CANDU reactors have unique features which allow them to effectively transmute transuranics**
- **TRU in MOX**
  - We can burn 40% of TRU
  - Reduce heat load by 40% at 1000 y
- **Am/Cm targets**
  - We burn 70% of Am (53% or Am+Cm)
  - Reduce heat load by 70% at 1000 y
- **Provide a significant increase in geological repository capacity.**
- **Full-core calculations indicate that both fuel cycles are feasible**

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**AECL**  
**EACL**

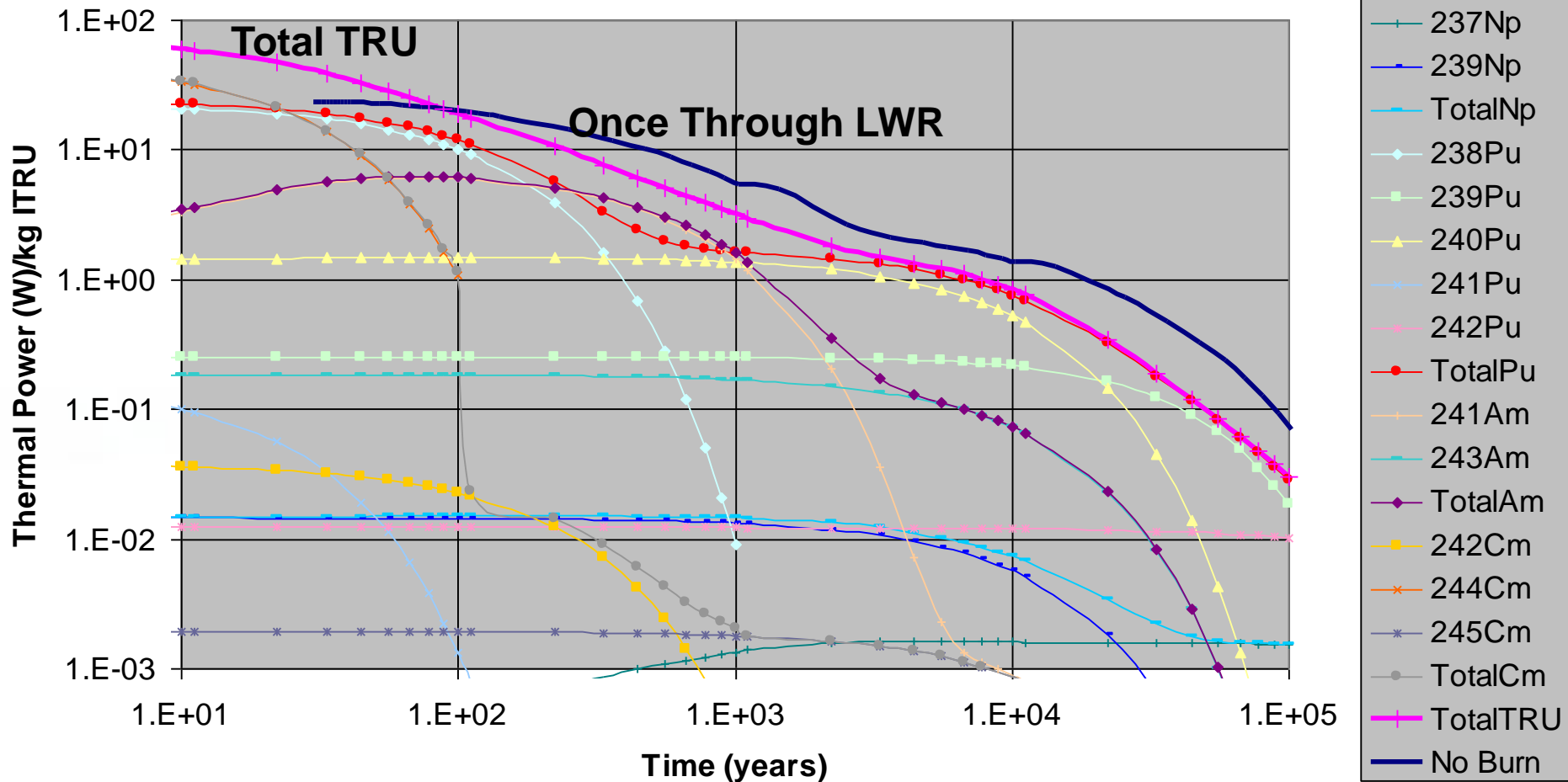


# 30 year cooled—No Burn





# Decay Heat from Actinides, MOX





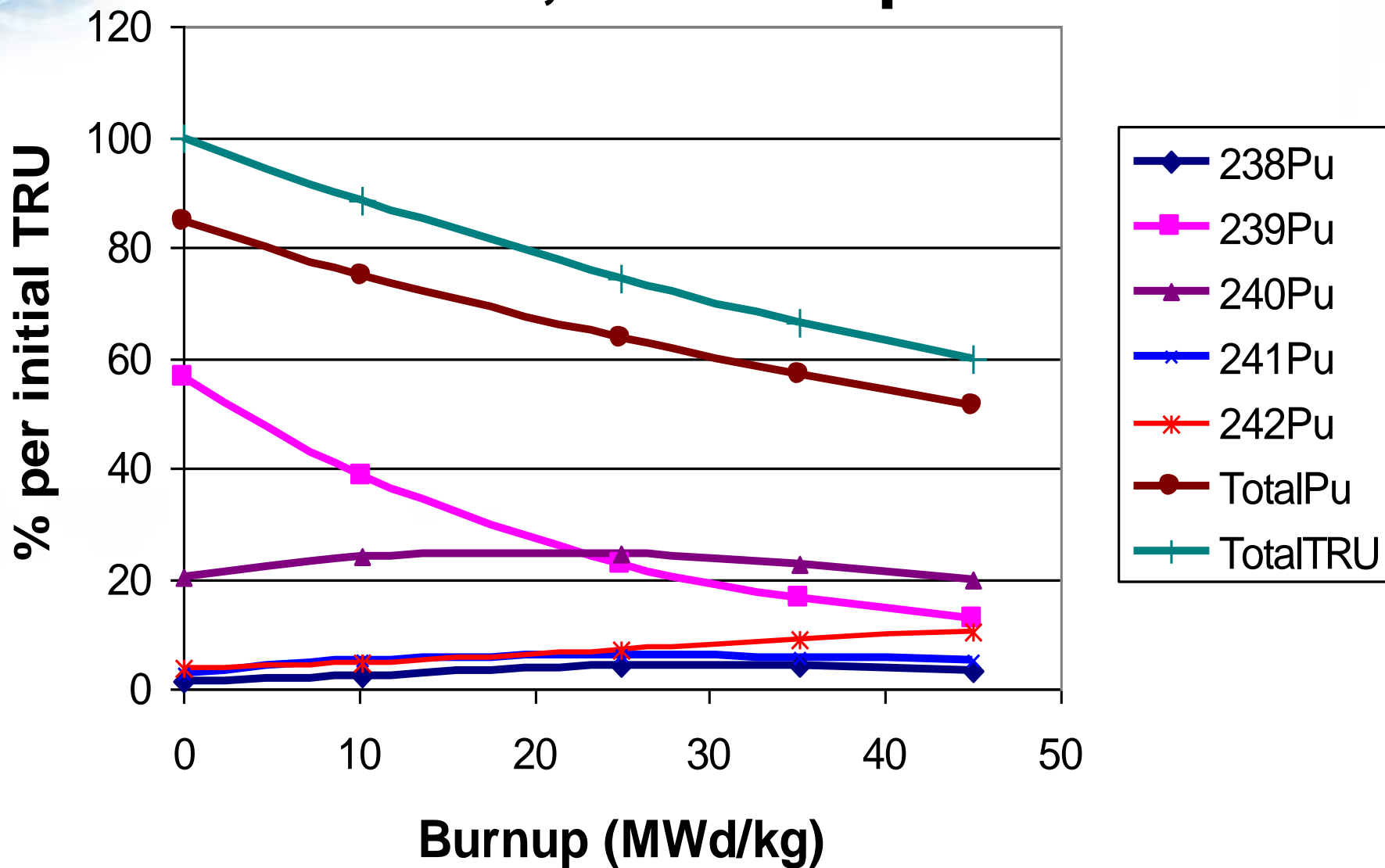


# The Calculation

- **WIMS-AECL used to calculate neutron fluxes**
- **ORIGEN-S used for the depletion calculation**
- **MOX: burned to 45 GWd/t**
- **Assumed 3% neutron leakage**

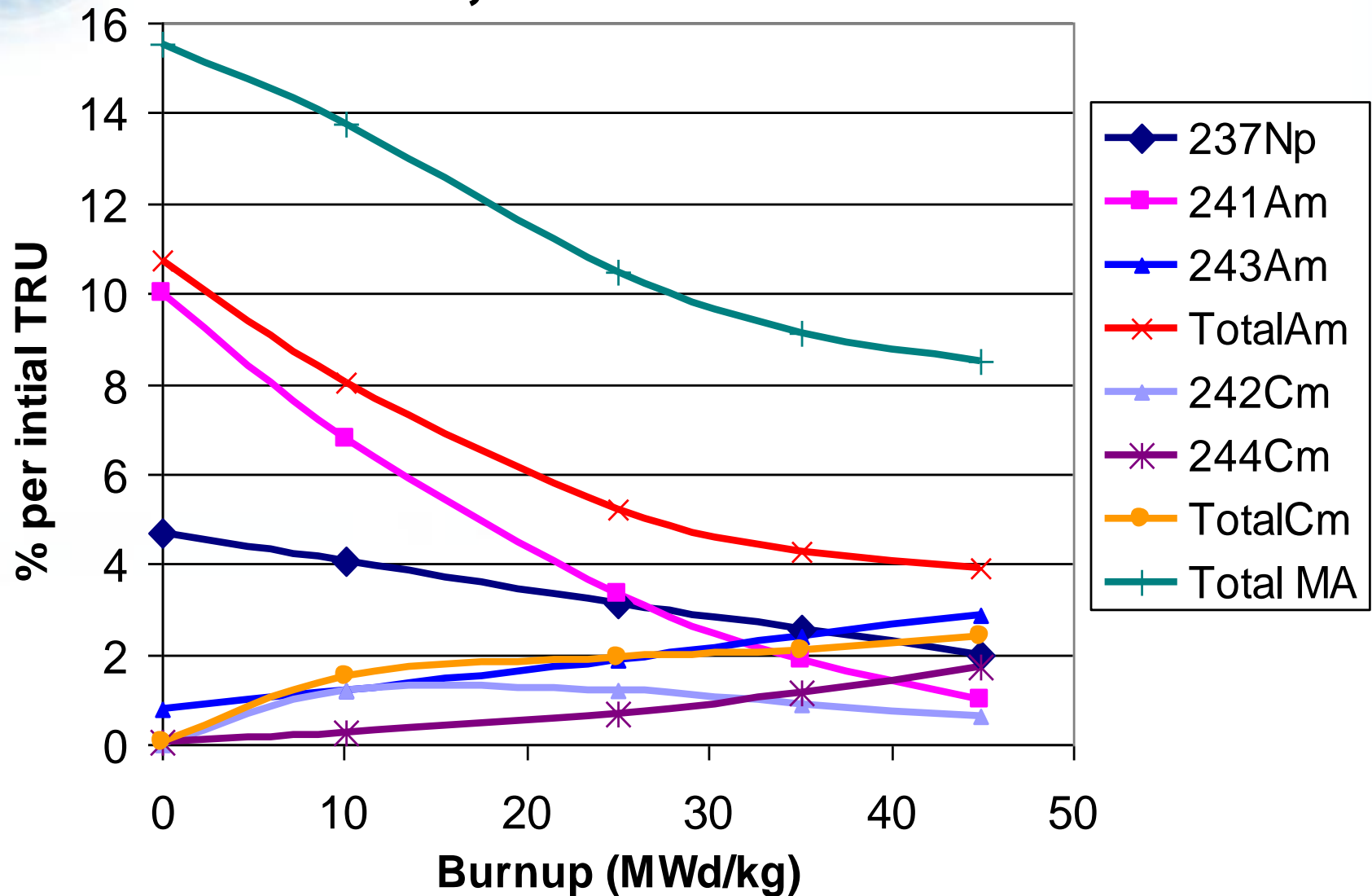


# MOX, Pu Isotopes



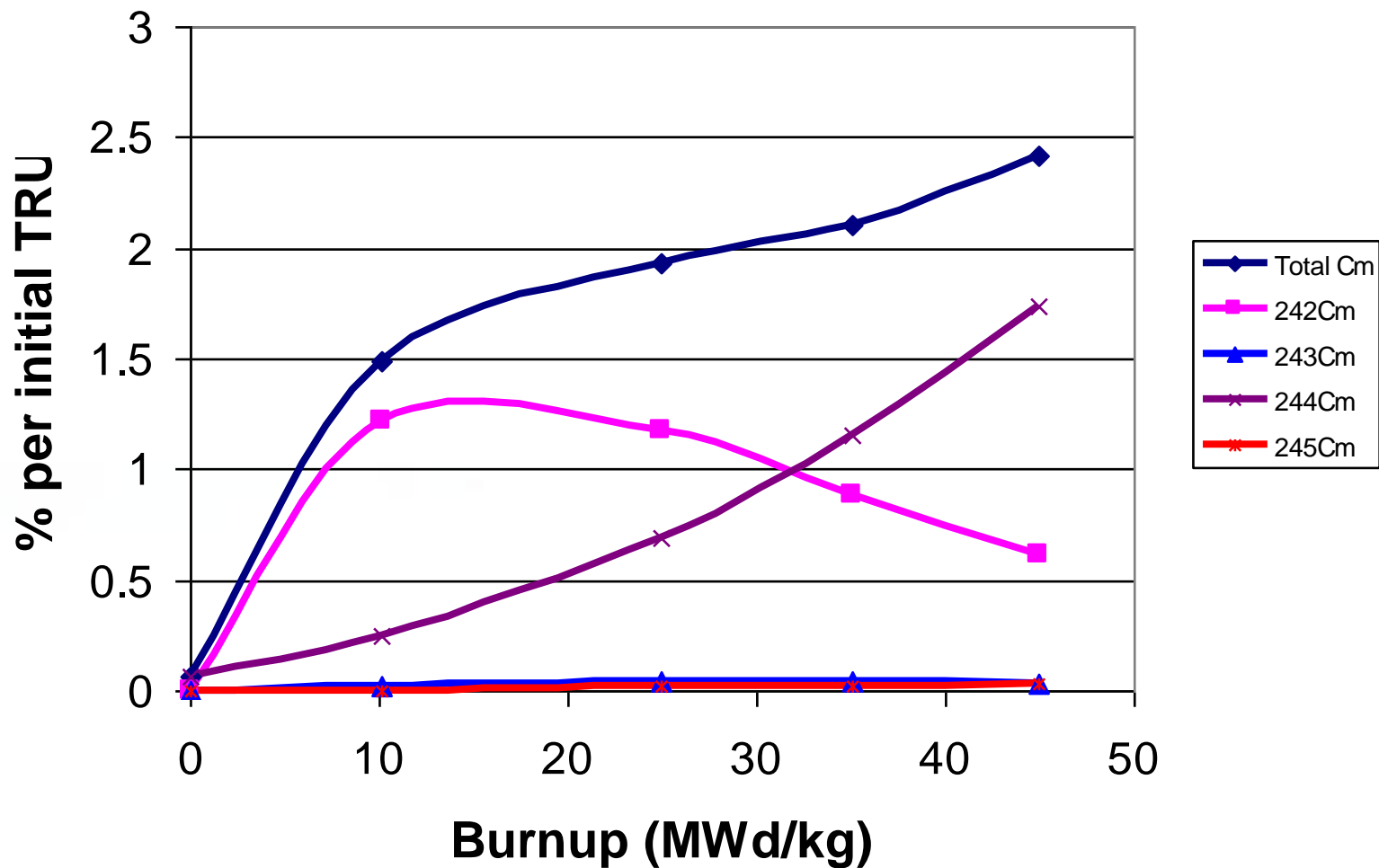


# MOX, Minor Actinides





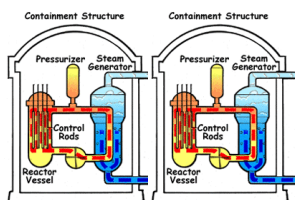
# Group Extracted TRU MOX, Cm



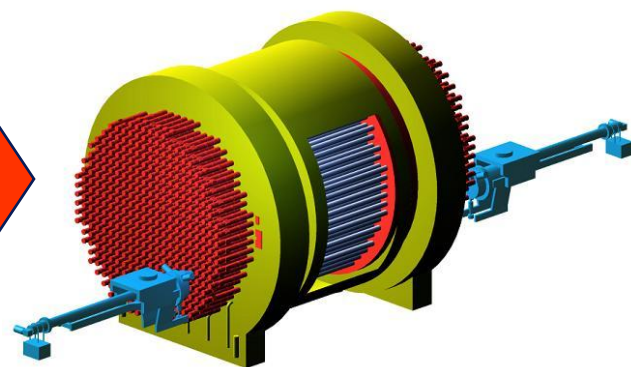
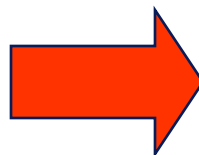
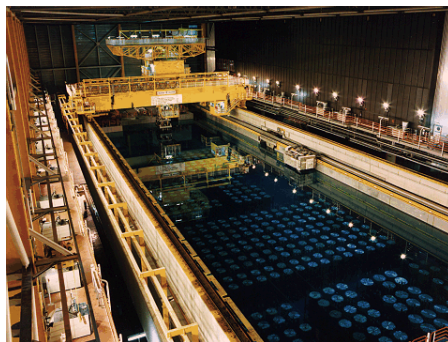
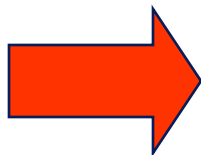
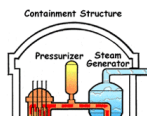


# Am Mass Flow

## 2.5 GWe LWR



**Decay for 30 years  
Separate Am, Cm**



**21 kg/year**

**90 kg/year**

**27 kg/year**

**63 kg/year destroyed**



# Full-Core Results

	<b>Time-Average</b>	<b>Refueling Ripple</b>	<b>NU Fuel</b>
<b>Max Channel Power (kW)</b>	<b>6600</b>	<b>7100</b>	<b>7300</b>
<b>Max. Bundle Power (kW)</b>	<b>790</b>	<b>845</b>	<b>935</b>

- **Avg. burnup for RU is 12.2 MWd/kg**
- **3.7 channels/day, 11 bundles/day**



# For 30 year cooled TRU in MOX At Mid-Burn

