

# *Los Alamos Transmutation Research*



- Heavy Liquid Metal Coolant  
Technology & Accelerator-  
Driven Materials Test Station

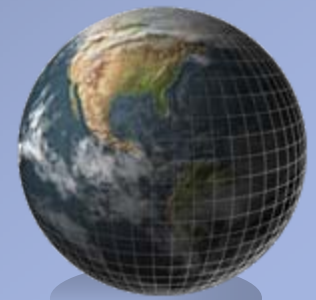
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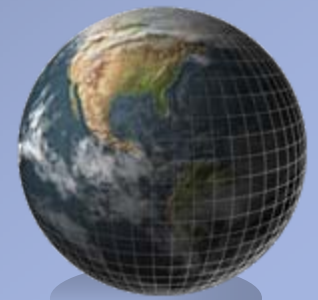
Sept. 28, 2006

# Outline

- Heavy Liquid Metal Coolant Technology
  - Facilities, materials testing and coolant technology development
  - Corrosion modeling and database
- Materials Test Station (MTS)
  - Mission
  - Design
  - Outlook



# Heavy Liquid Metal Coolant Technology

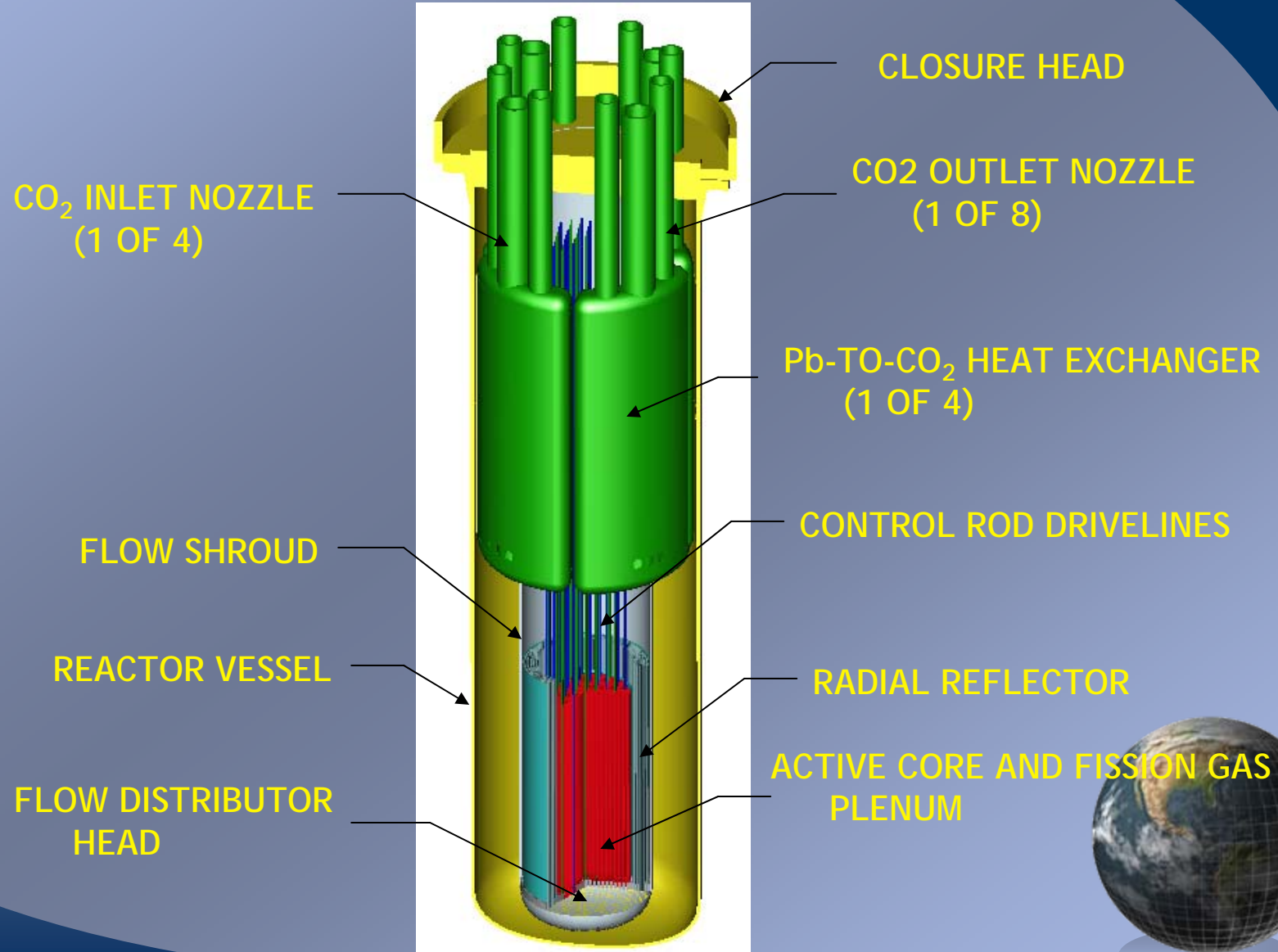


# *Supporting AFCI and Gen IV Goals*

- AFCI (Advanced Fuel Cycle Initiative):
  - Original goal: ADS spallation target and transmutation assembly - substantially met;
  - Extended goal: reactor-based systems and Gen IV
- Gen IV LFR (Generation IV Nuclear Energy Systems: Lead Fast Reactor):
  - Near to mid-term: long-life LBE/Pb-cooled cores - long-term corrosion performance under testing, modified materials and/or surface treatment necessary
  - Long-term: high temperature Pb-cooled systems - new classes of materials and coolant technology necessary
- GNEP (Global Nuclear Energy Partnership)
  - Sodium as coolant for Advanced Burner Reactor (ABR)
  - Will SMR (small and modular reactors) carry LFR?



# Gen IV LFR: 20 MWe SSTAR Concept



# DELTA Loop (DEvelopment of Lead-alloy Technology and Applications)

- Design in 2000
  - 316L construction
  - ~100 kW power
  - Up to 15 m<sup>3</sup>/h forced circulation with a 20 HP pump, normally operated below 2 m/s LBE flow in test section
  - Oxygen controlled
- Loop construction finished in 2002
- Operation and testing since 2003

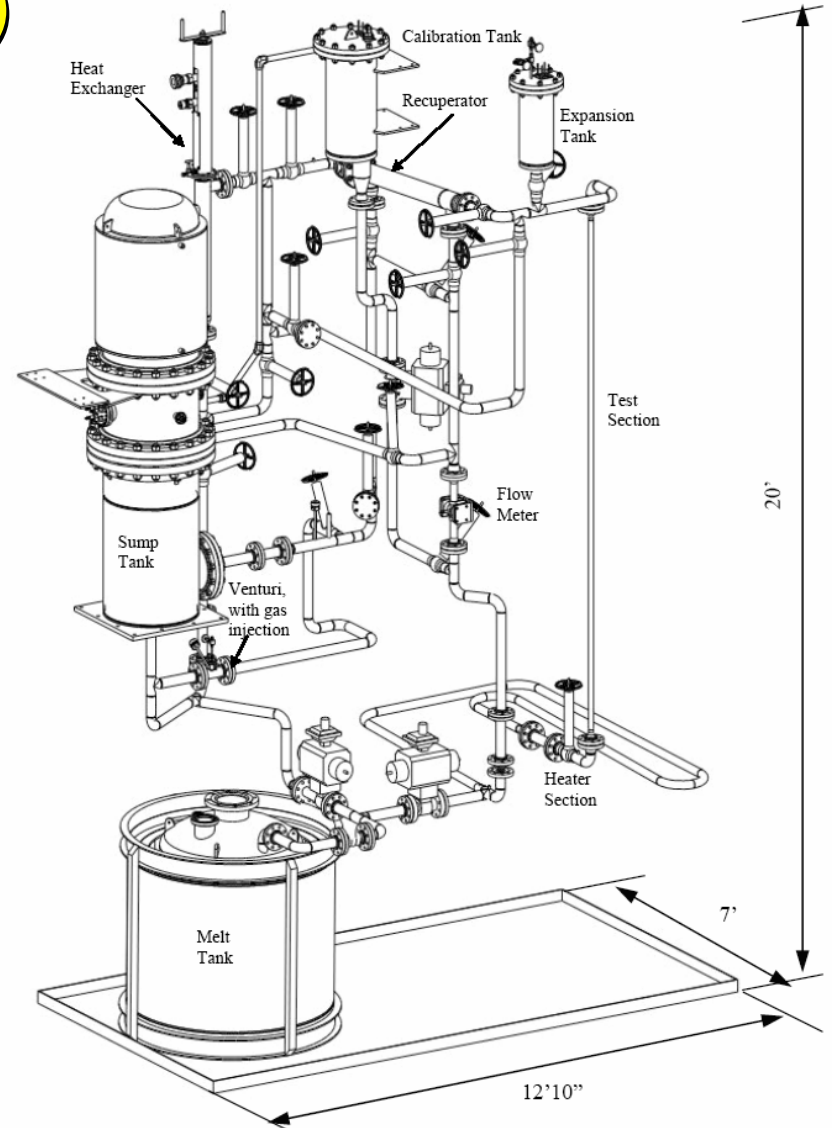
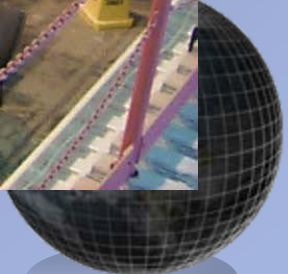


Fig. 1. Illustration of the DELTA Loop.

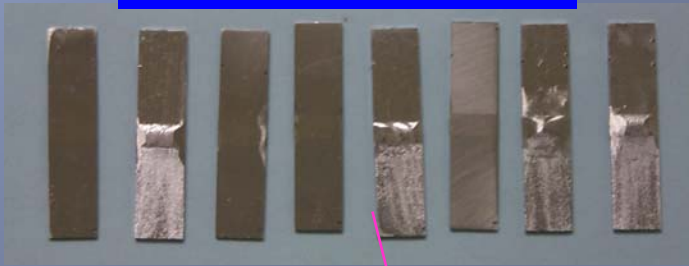
# *DELTA Loop in MPF-18, TA-53, LANL*



# DELTA Loop Test Program

- Large test matrix of materials, surface treatment, and alloy modifications
- Materials from other national labs, universities and international partners

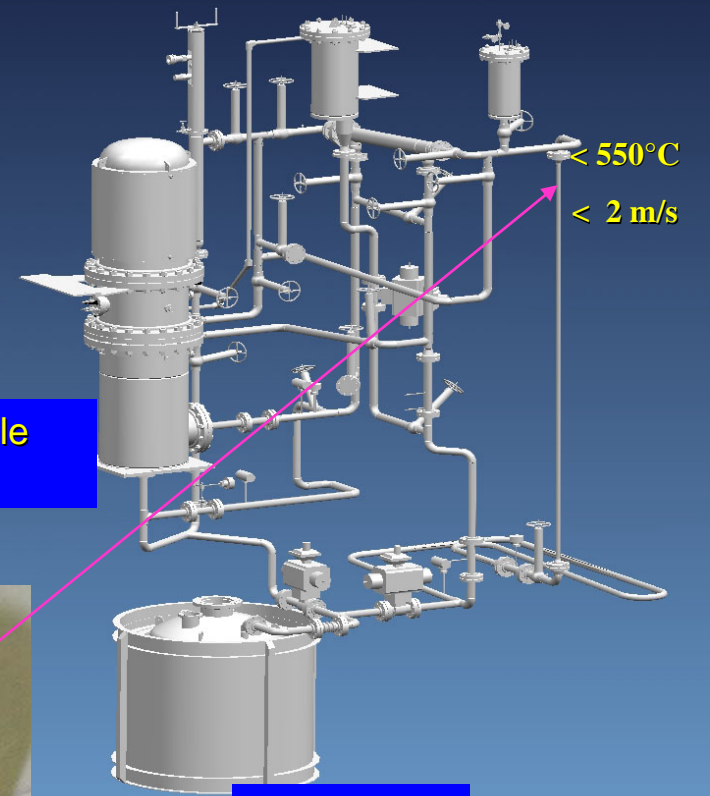
Corrosion test specimens



Sample holders (slotted channels)



Assembled sample holders



After testing

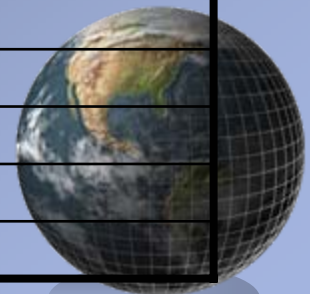




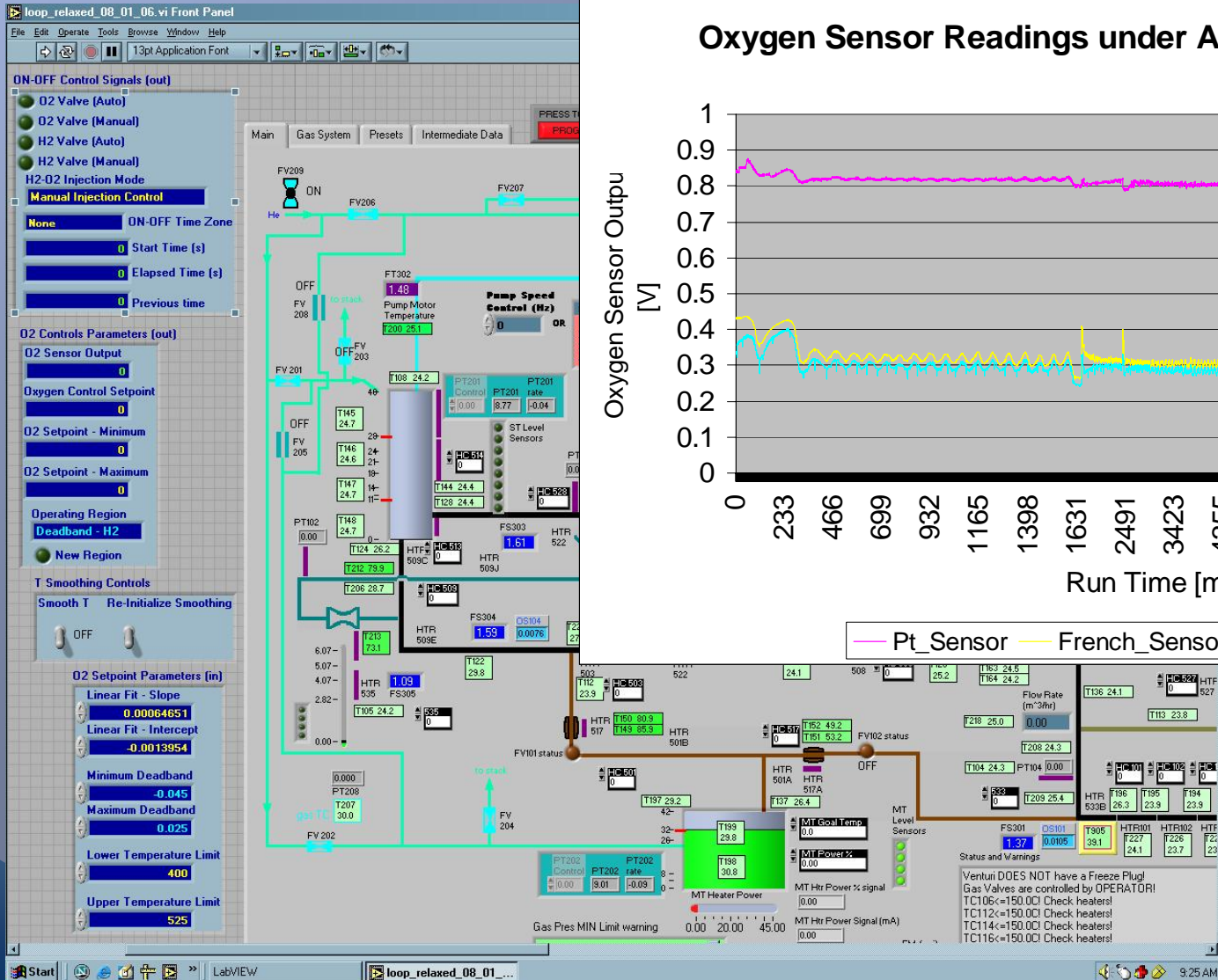
# 2005 DELTA Test Campaign

- $T=540^{\circ}\text{C}$ ,  $C_{\text{O}}=10^{-6}\text{wt}\%$ ,  $v=1\text{m/s}$ ,  $t=200, 400, 600\text{hrs}$
- Stress Corrosion Cracking specimens(HT-9, EP823 U-bends and C-rings)
- Corrosion specimens:

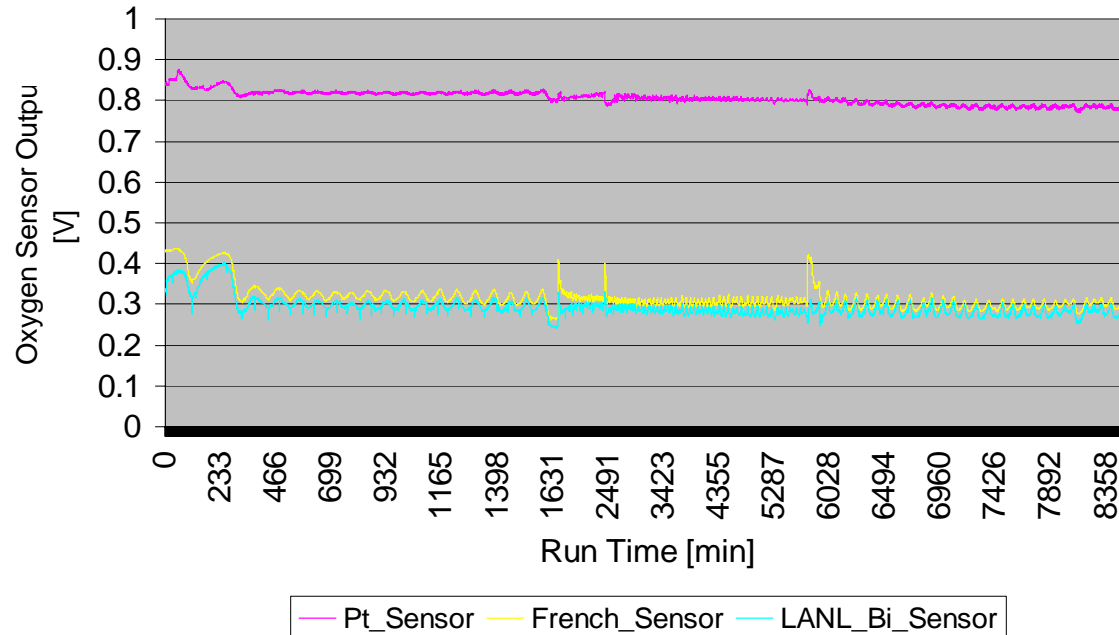
ODS steels	MA957	NF616, sputtered and implanted	NF616
	MA956		Si + Xe <sup>+</sup>
	12YWT		Y + Xe <sup>+</sup>
	14YWT		O <sup>+</sup>
	PM2000		Ta + Xe <sup>+</sup>
	(JNC)		HCM12A, sputtered and implanted
Amorphous coating on 316L	SAM40	Laser-peened	Y + Xe <sup>+</sup>
	SAM40X3		O <sup>+</sup>
	HVOF		Ta + Xe <sup>+</sup>
	C22 HVOF		316L
Mo, W coating	MA957	High Cr	EP823
	EP823		HT-9
	HT-9		T91
	9Cr-1Mo		18-20%Cr



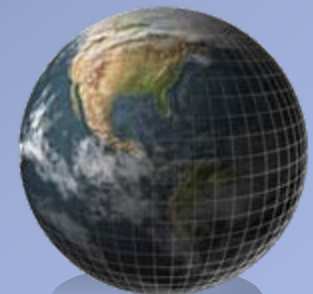
# Stable Oxygen Control in DELTA via Automated Direct Gas Injection



## Oxygen Sensor Readings under Automatic Control



# *A Bypass in DELTA with a PbO Mass Exchanger to Control Oxygen Concentration in LBE*



# *Investigate Effects of Oxygen Concentration in LBE on Heat Transfer and Wetting in DELTA*



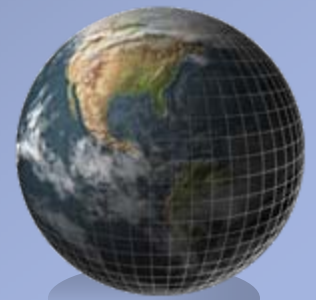
Fig. 4(a). Interior of the pipe after the first run at low oxygen concentration.



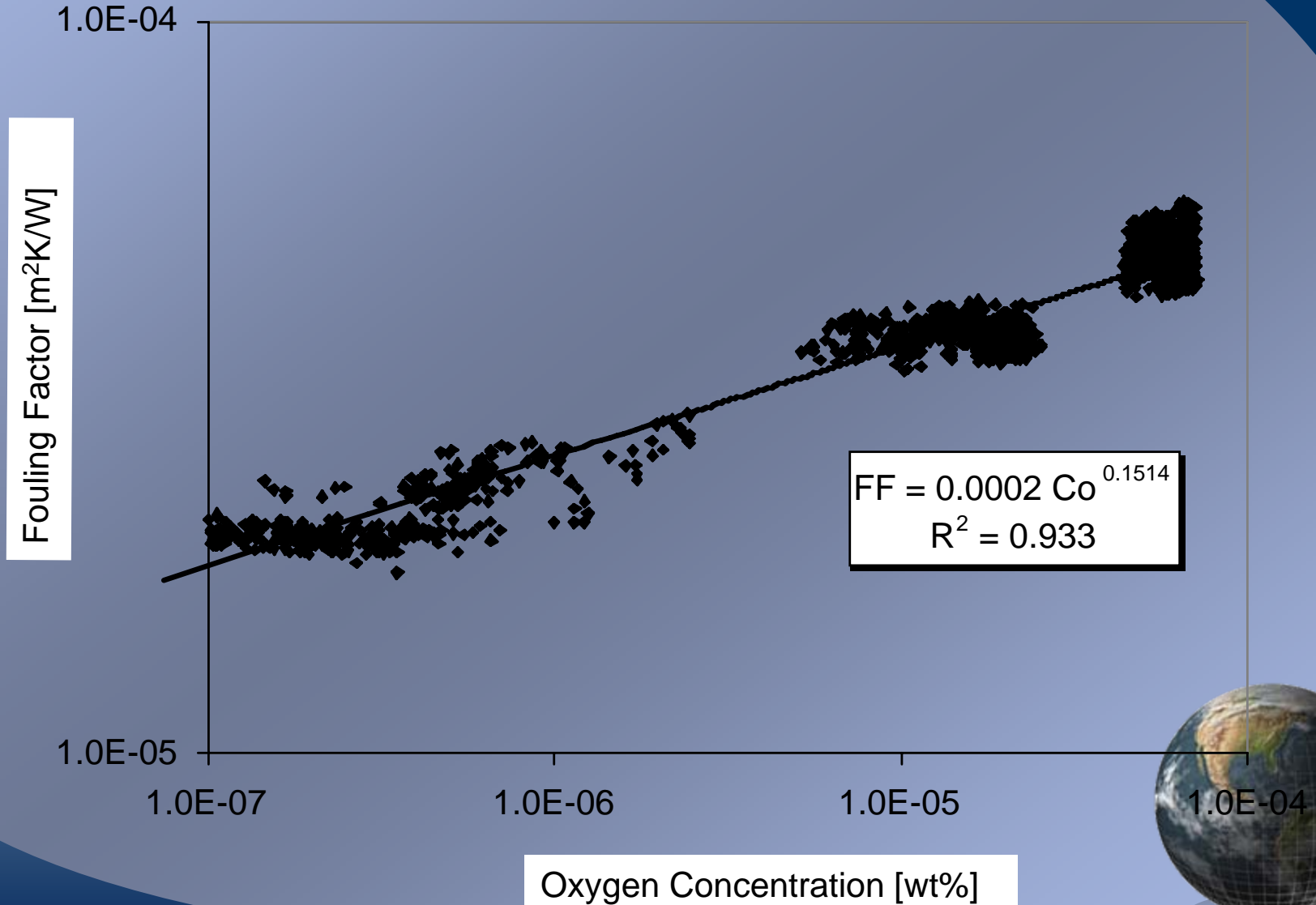
Fig. 4(b). Interior of the pipe after the second run at a higher oxygen concentration.



Fig. 4(c). Interior of the pipe after the third run at the highest oxygen concentration, showing significant oxidation and evidence of non-wetting.

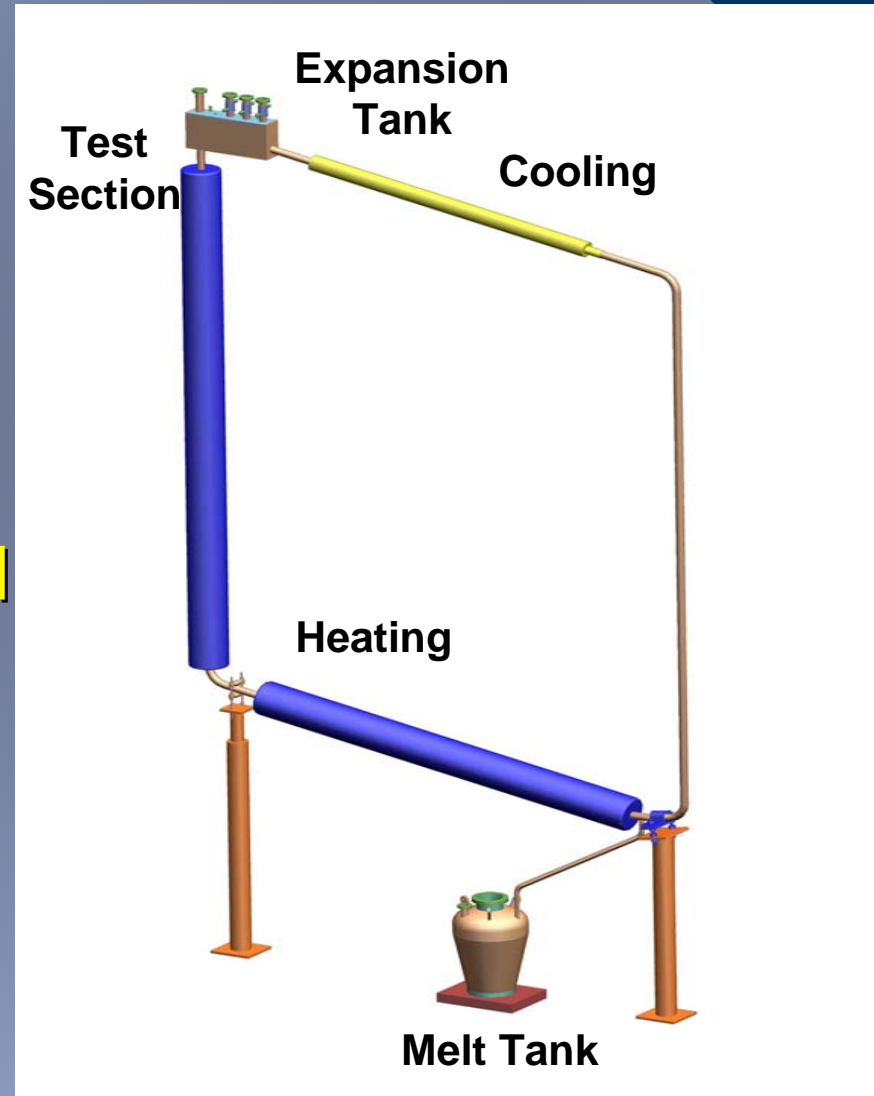


# Fouling Factor: Increasing Oxygen Concentration Reduces Heat Transfer

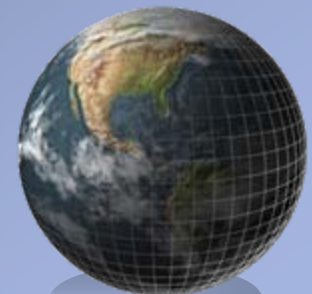
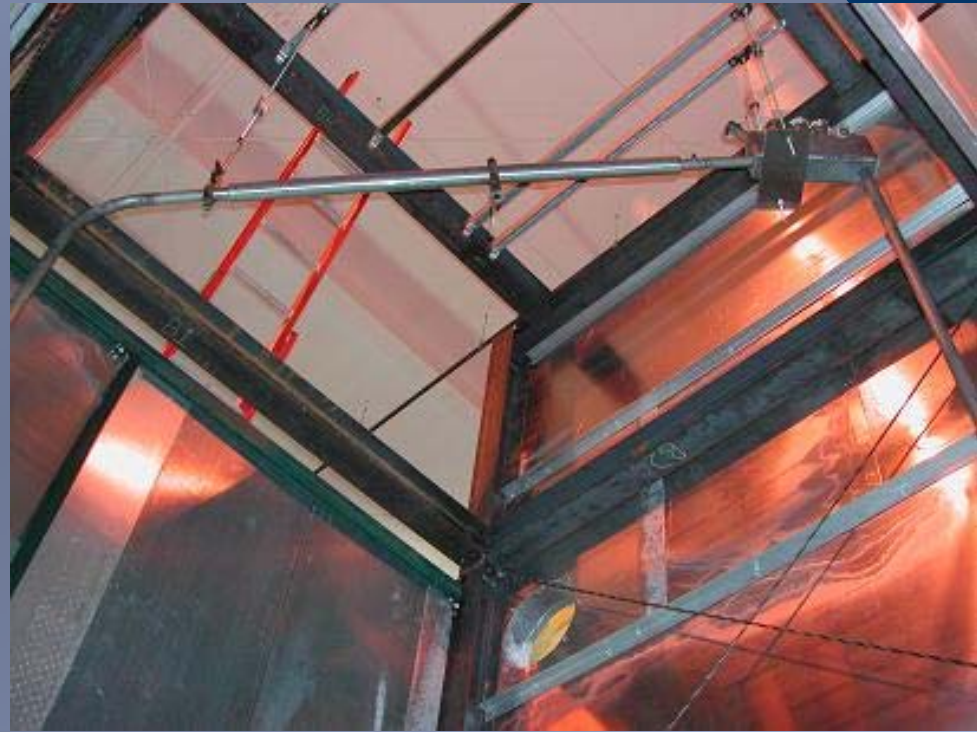


# Lead Correlation Stand to Test Materials in Pb for Temperatures up to 650-700°C

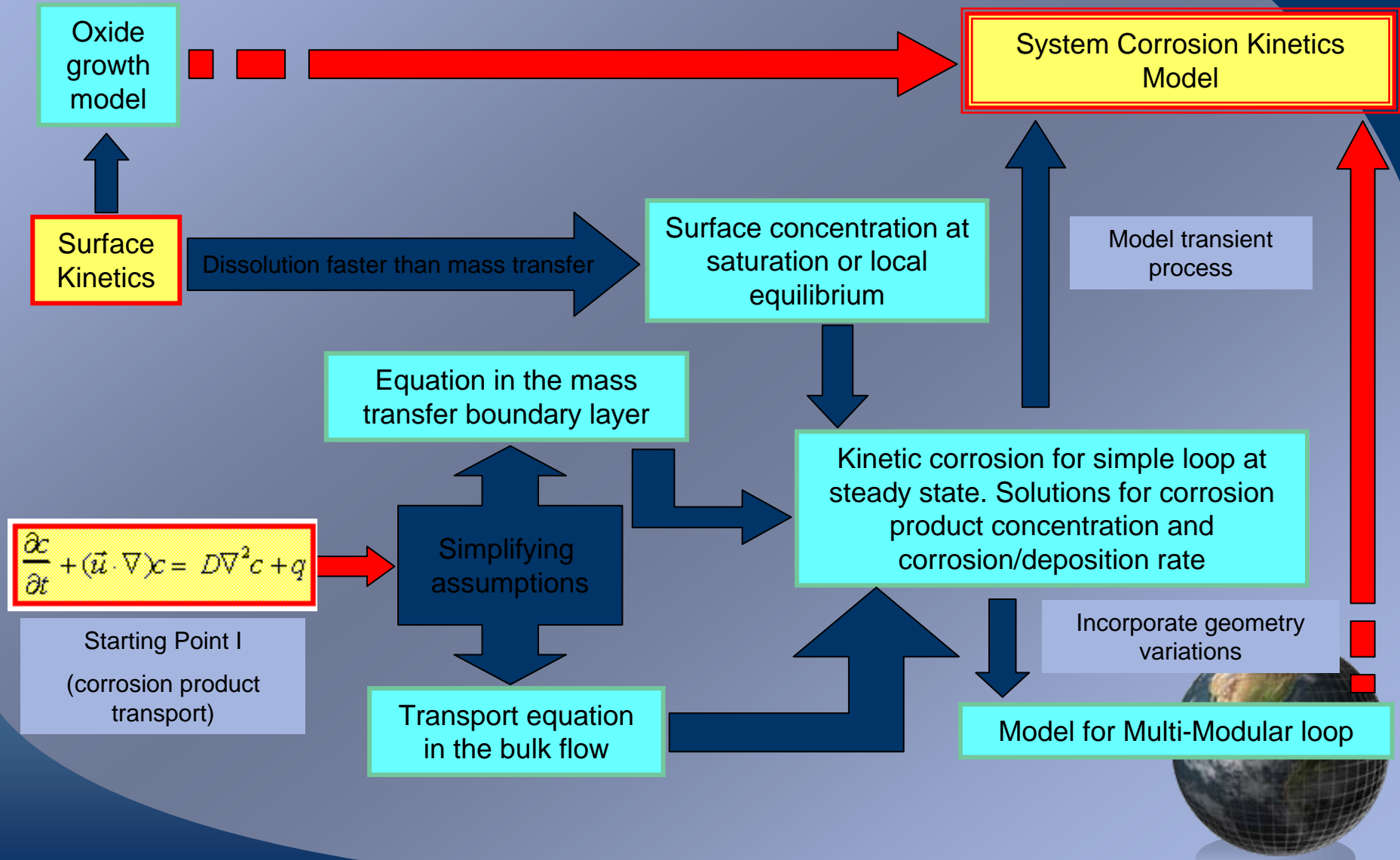
- Design
  - ODS steel (MA956 – 4.5%Al) construction
  - 0.25m/s natural circulation
  - Oxygen controlled
- Loop construction finished
- Auxiliary systems to be added



# *Lead Correlation Stand (LCS) in Support and Containment Structure (Next to DELTA)*



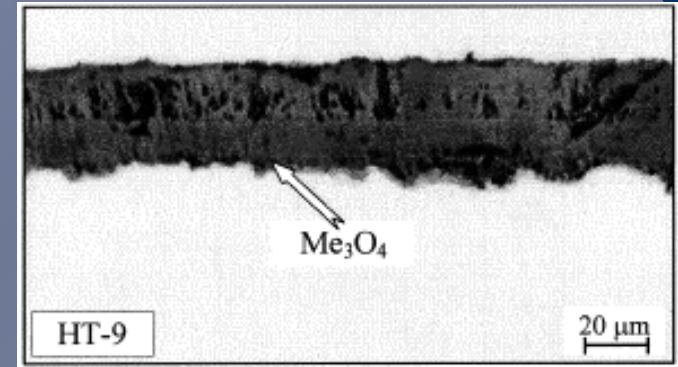
# A Framework to Systematically Model Oxidation and Corrosion of Steels in Lead-Alloys



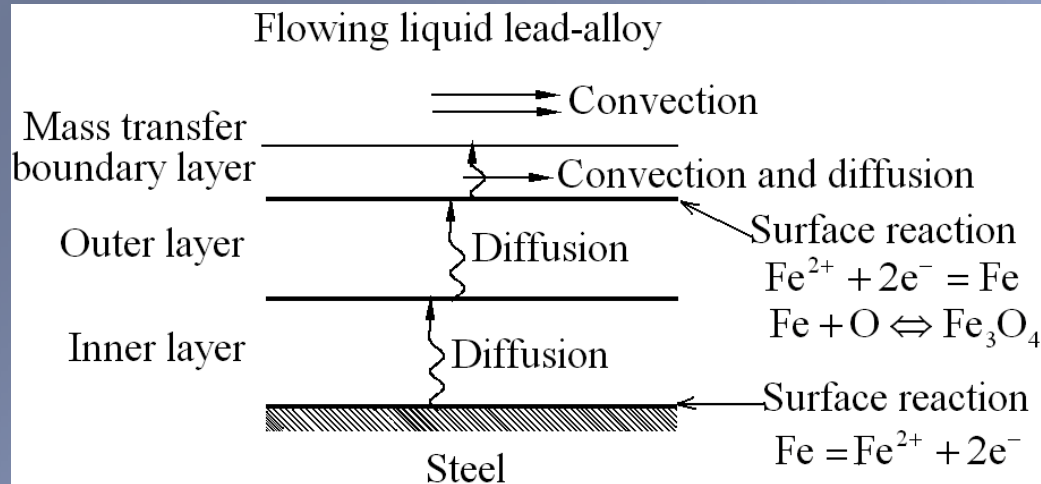


# Modeling Oxidation/Corrosion Based on First Principles and Experimental Findings

- Oxide scale:  
experimentally observed structures

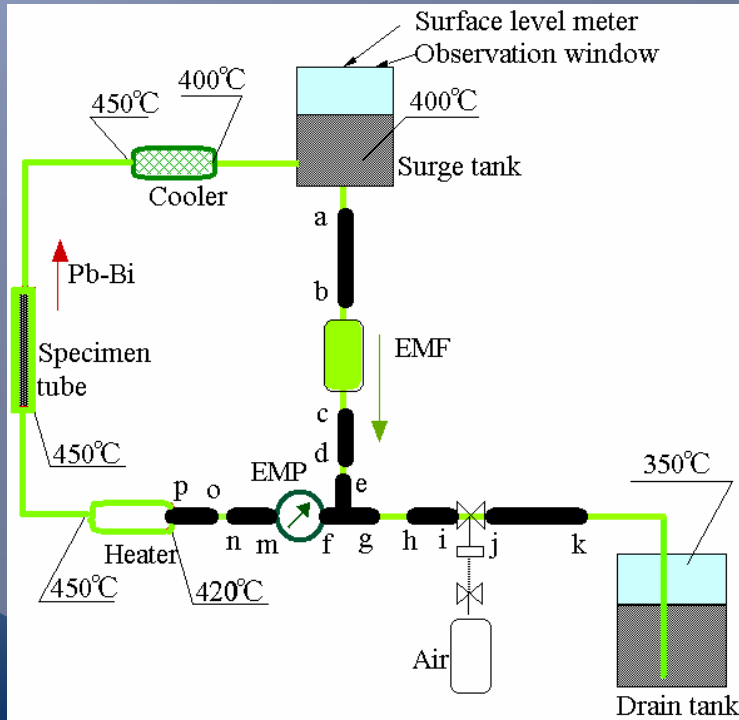
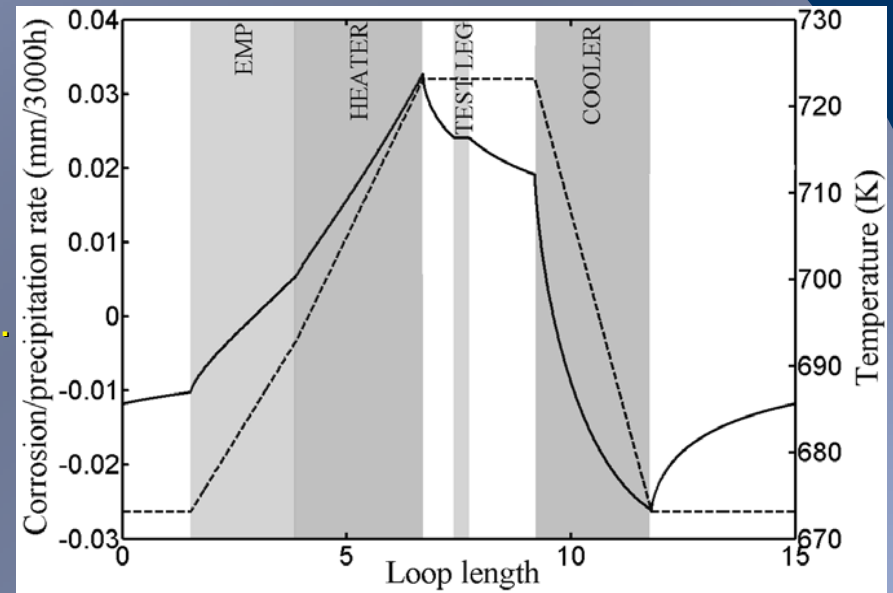


- Mass transfer in liquid metals:  
turbulence model

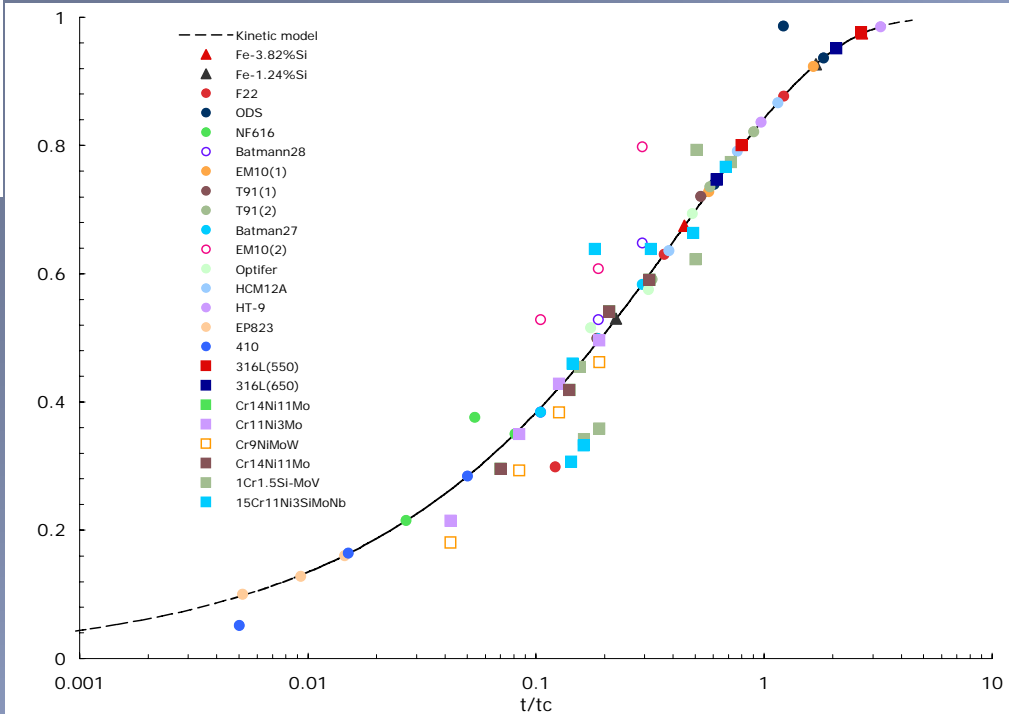
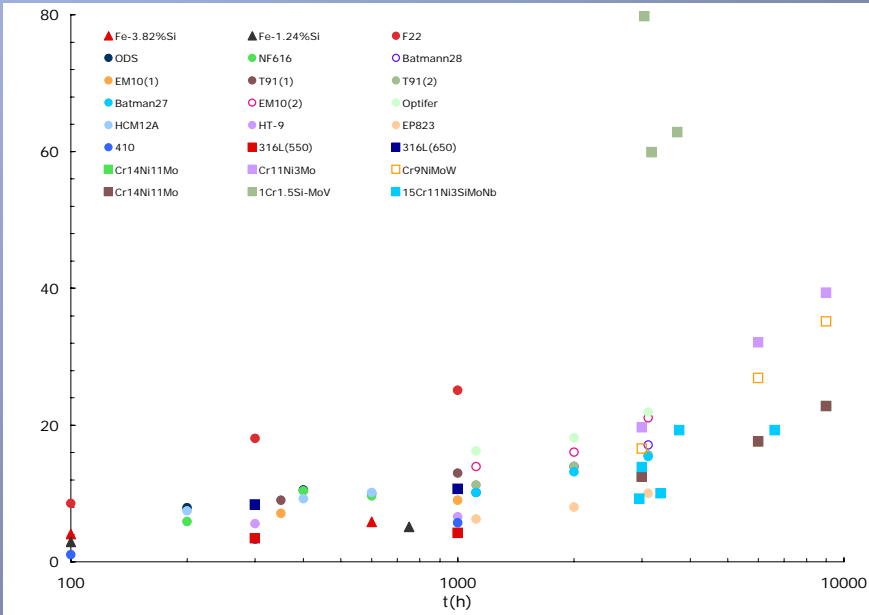


# Benchmarking of Corrosion Model against JLBL-1 (JAERI Lead-Bismuth Loop) Experiment

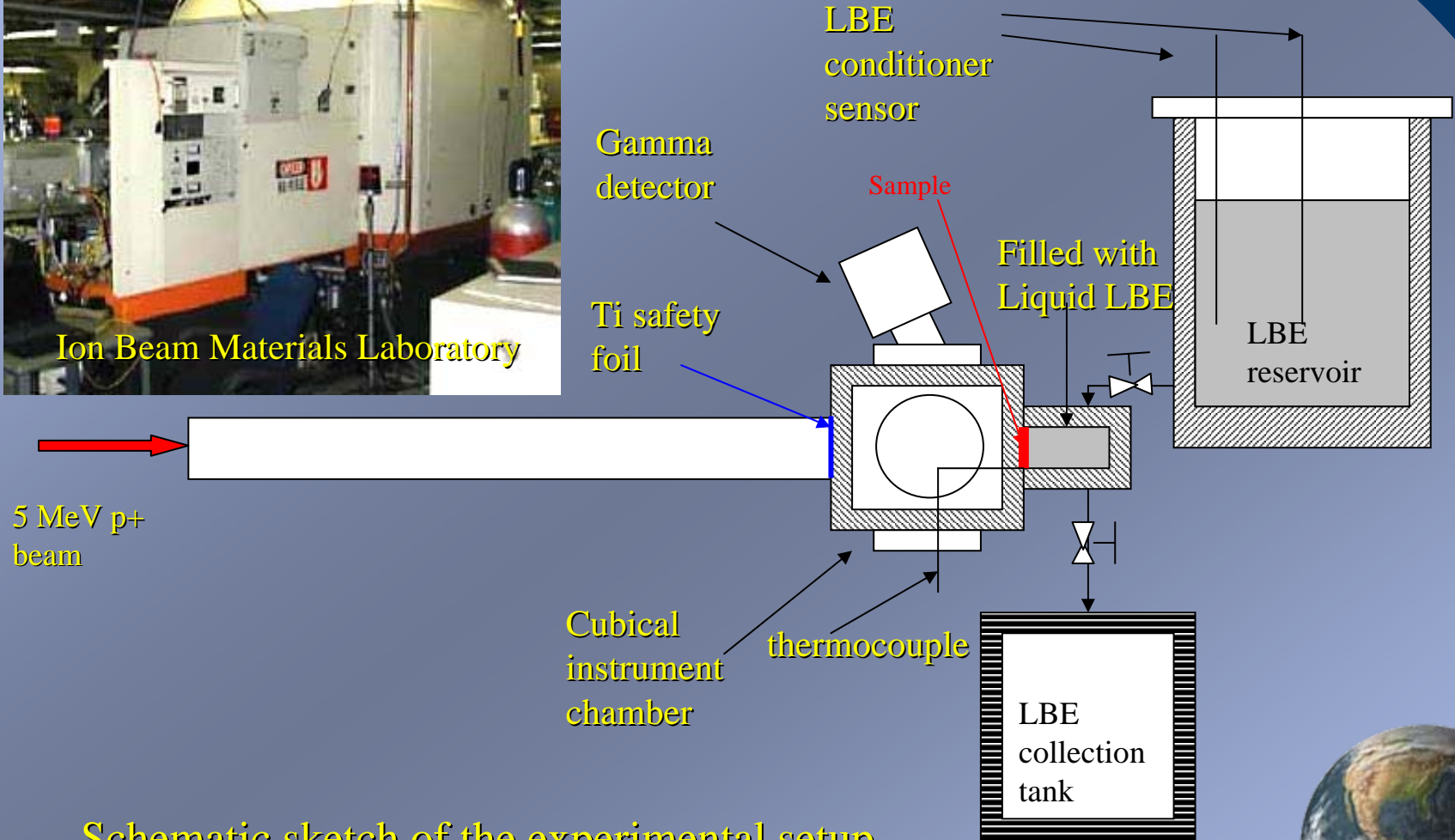
- Calculated dissolution/precipitation rate for iron (solid line) and the temperature profile (dashed line) for JLBL-1 loop.
- Deposition zone (thick back line) in JLBL-1 experiment. The corrosion rate is between 0.03-0.1 mm at the highest temperature leg.



# Universal Scaling of Reported LBE Loop Test Data based on the System Kinetic Model



# Irradiation and Corrosion Experiment (ICE)



Schematic sketch of the experimental setup.



# ICE Device and Shielded Platform



*LBE reservoir*



*Window/sample*



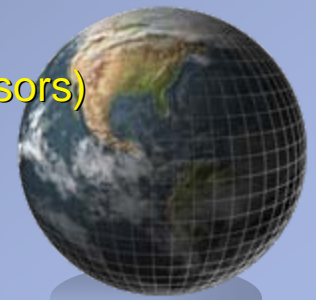
*ICE chamber*



*Shielded Platform*

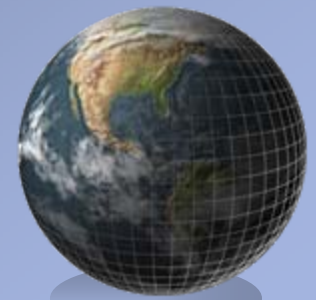
# *Extensive Collaborations*

- University
  - UNLV (TC-1, materials, corrosion testing, sensors, etc)
  - MIT (Si effects, oxygen sensor seals)
  - U Wisconsin (testing fusion alloys and modifications)
  - Idaho Accelerator Center (oxygen sensor irradiation)
  - U Illinois U-C (corrosion probes)
  - U Michigan (irradiation effects on corrosion)
  - TAMU (nano-engineered coatings and materials)
  - Penn State U (oxide microstructure)
  - UC Santa Barbara (ODS steels)
- International
  - OECD/NEA HLMC Handbook project
  - EU (DEMETRA, MEGAPIE)
  - CEA (coolant technology, materials testing, oxygen sensors)
  - SNU/KAERI (fuel cladding materials, oxygen sensors)
  - JAERI (ADS), JNC/TITech (FR)
  - IPPE (ISTC projects on TC-1, PbO-based oxygen control, oxygen sensors)

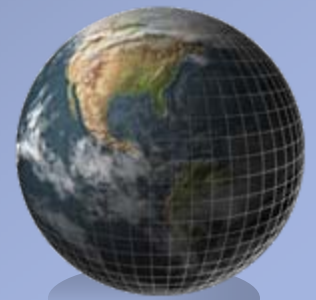


# *Lead-Alloy Coolant Technology and Materials Readiness Level Assessment (2005)*

- For peak temperature  $< 550^{\circ}\text{C}$ , and for short to medium lifetime at high temperatures, technology readiness level (TRL) is at early stages of “Proof-of-Performance”.
- For long-life core (20-30 years), and for high performance and higher temperature (up to  $750^{\circ}\text{C}$ ), TRL is at “Proof-of-Principle”.
- For temperature higher than  $800^{\circ}\text{C}$  (e.g. for hydrogen production), TRL is at “Proof-of-Concept”.



# Accelerator-Driven Materials Test Station (MTS)





# *The MTS is a fast spectrum irradiation facility supporting fuels and materials development for the Global Nuclear Energy Partnership*

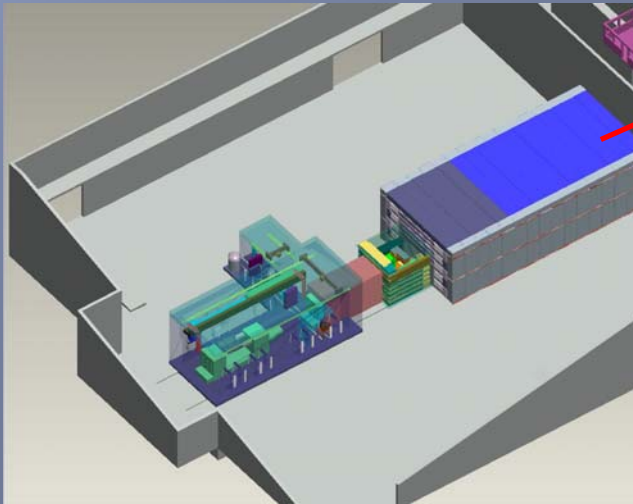
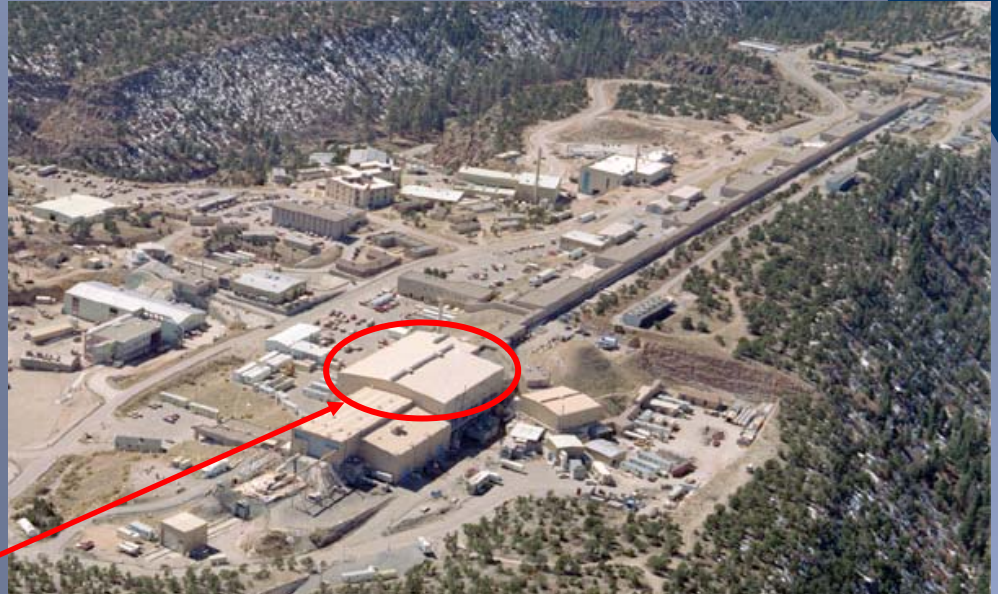
- The U.S.A. has no fast neutron spectrum irradiation facility
- There are a limited number of viable facilities abroad:
  - PHENIX (France, due to close by end of this decade)
  - JOYO (Japan)
  - BOR-60 (Russia)
- The AFCI program has been successful in securing irradiation services abroad, but the process
  - is time consuming
  - has risks associated with approvals outside of U.S. control
- The MTS will provide a near term fast spectrum domestic irradiation capability until the Advanced Burner Test Reactor is available circa 2019



# *MTS will be located in the 3,000 m<sup>2</sup> LANSCE “Area A” experiment hall*

## **Existing assets include:**

- 800-MeV, 1.35-mA proton linac
- 30-T crane
- 6 MW secondary cooling
- Ktons of shield blocks
- 12 MW electrical service



***The LANSCE accelerator and Area A have a replacement cost of \$1B. Utilization of this unique resource significantly reduces MTS capital costs.***

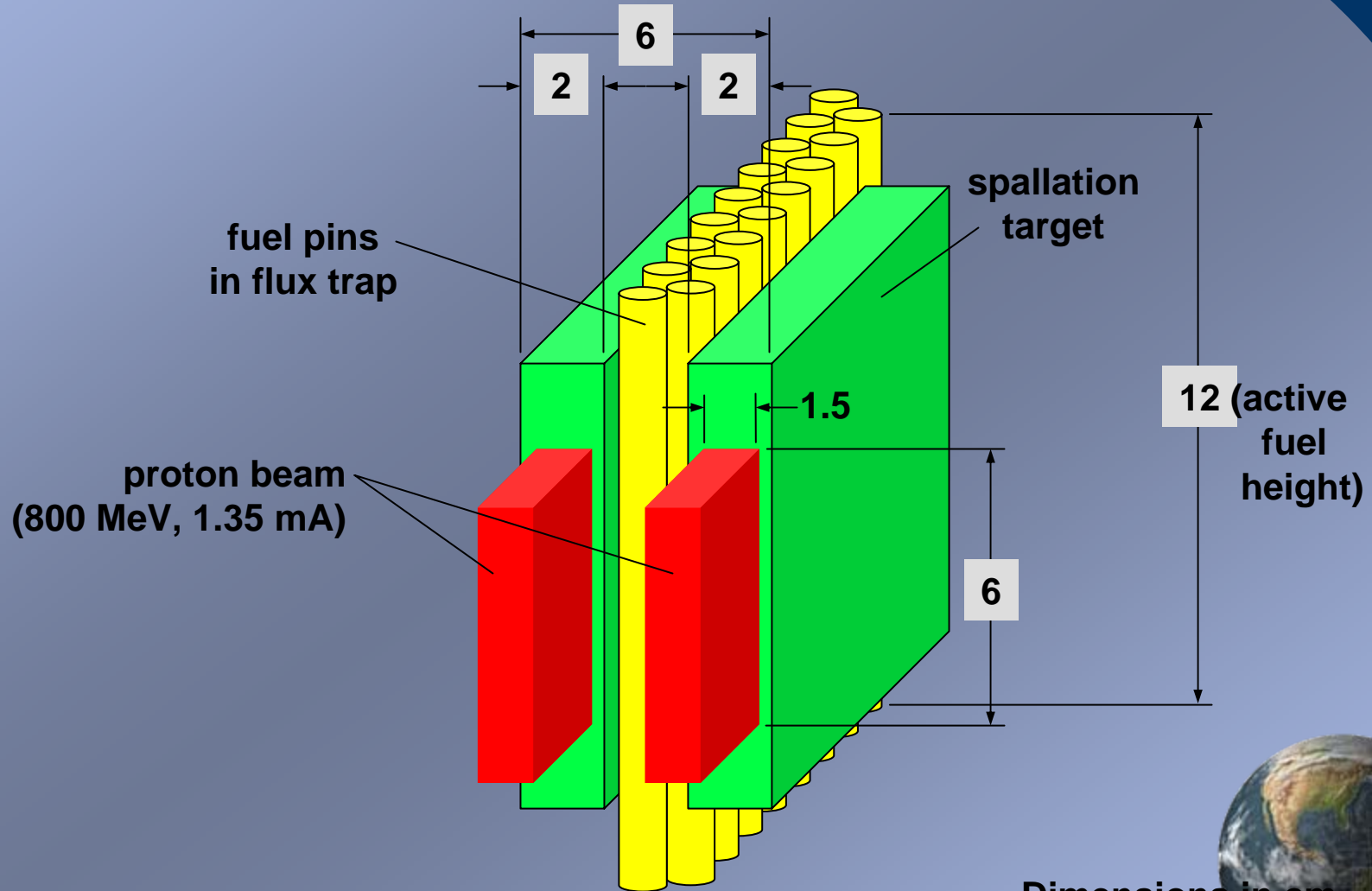


# *The MTS irradiation environment meets programmatic objectives*

Criterion	Design Requirement	Current Design
Neutron spectrum	Similar to that of a fast reactor	Meets requirement
Peak fast (>0.1 MeV) neutron flux	$\geq 1 \times 10^{15} \text{ n.cm}^{-2}.\text{s}^{-1}$	$1.3 \times 10^{15} \text{ n.cm}^{-2}.\text{s}^{-1}$
Irradiation volume	Sufficient to achieve 360 W/cm linear heating in 20 cm of highly enriched fuel	Exceeds requirement by factor of 10
Availability	$\geq 3\%/y$ burnup and $\geq 10 \text{ dpa/y}$ in Fe in the peak flux region	4%/y burnup and 18 dpa/y in Fe
Prototypic fast reactor environment	Ability to accommodate liquid metal coolants	Meets requirement



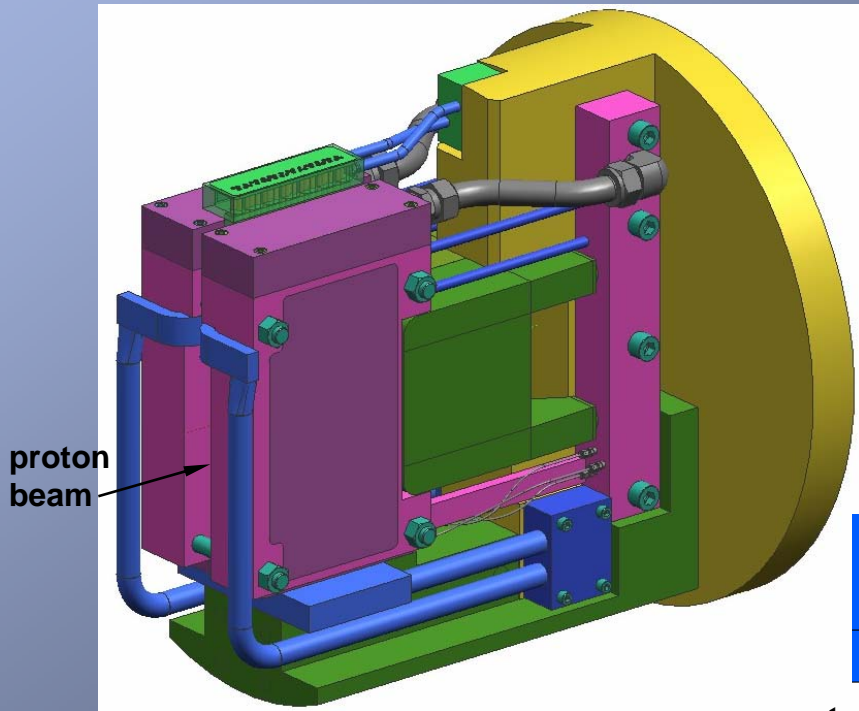
*The MTS spallation target is split, with a “flux trap” irradiation zone located in between*



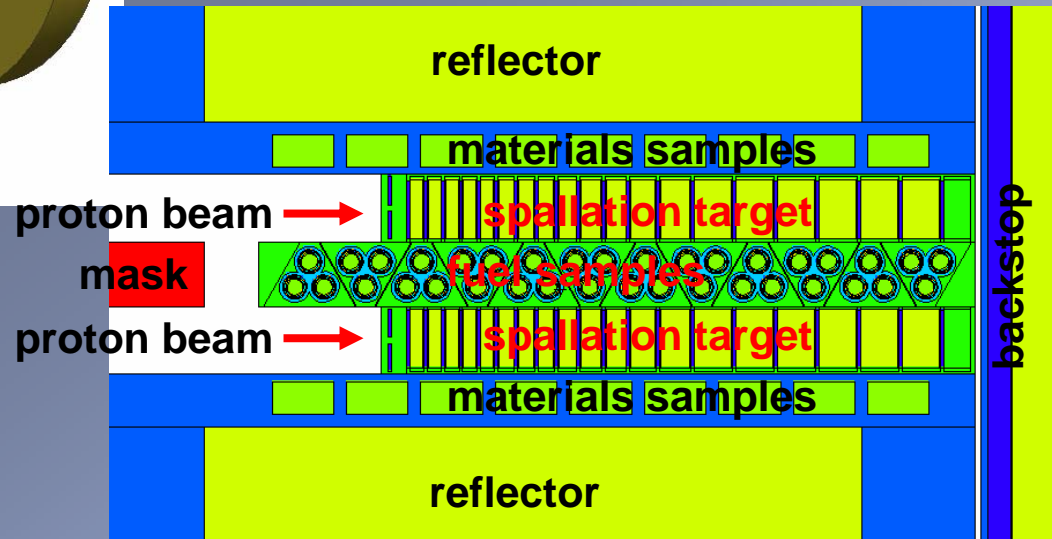
Dimensions in cm.



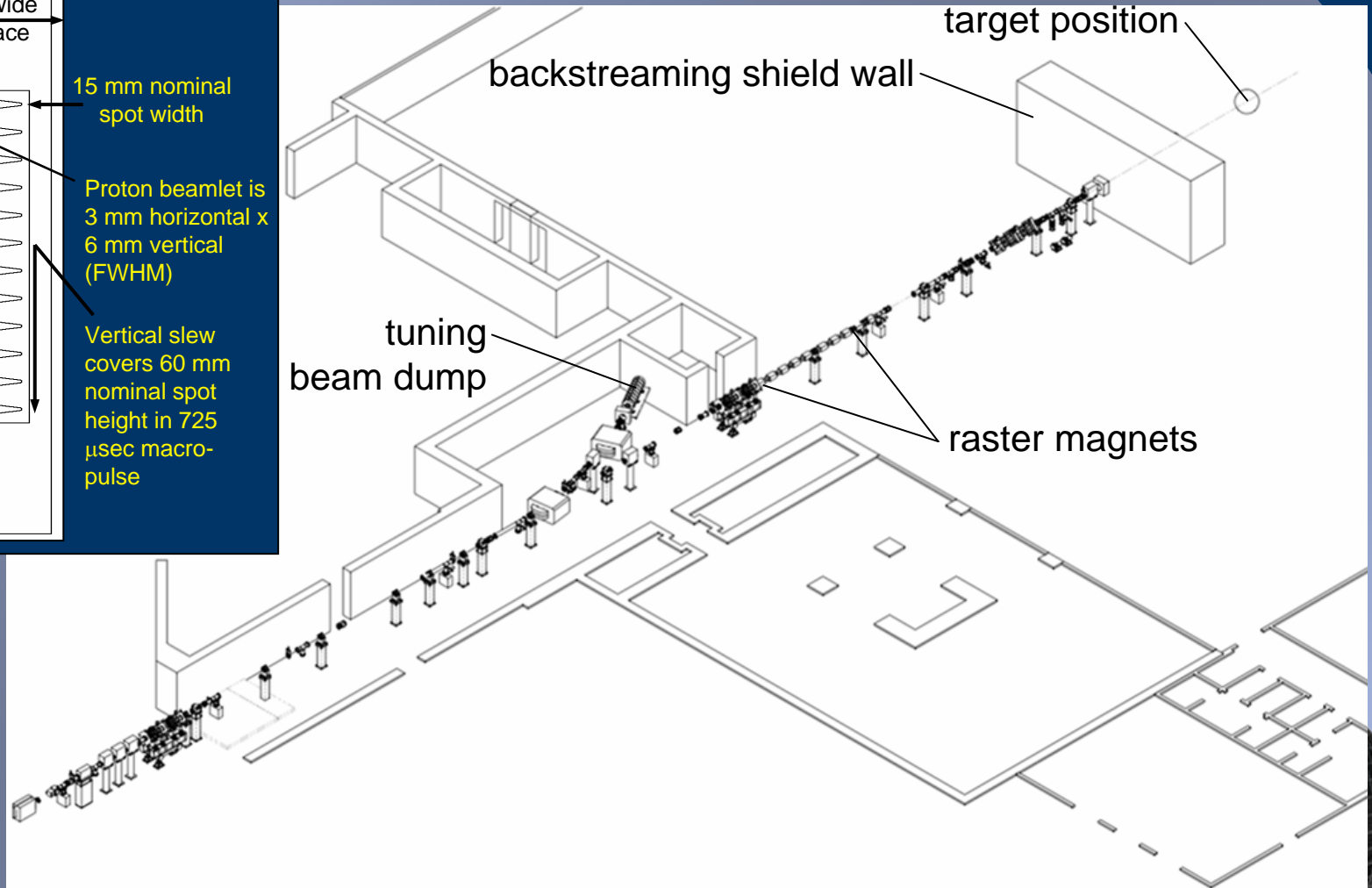
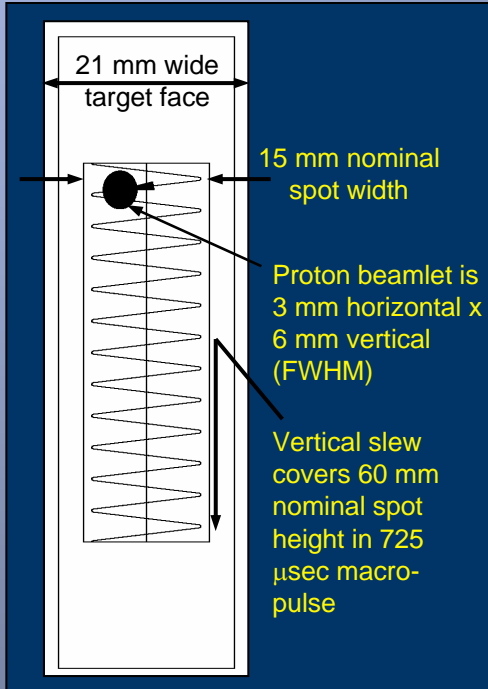
*The target assembly consists of a series of nested inserts*



Horizontal cross section at mid-plane



# The MTS beam line design includes a raster system that produces nearly uniform current density on target



# The spallation target uses well established technology

Target material

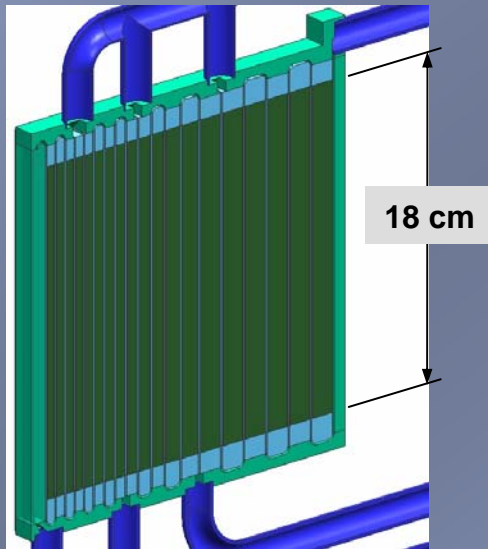
Coolant

Beam energy (MeV)

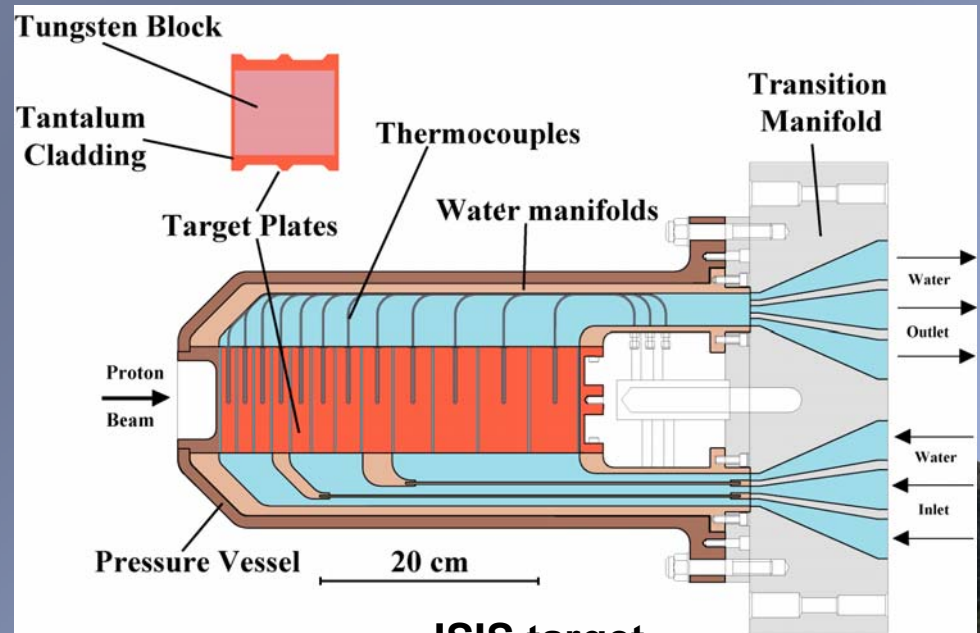
Beam current (mA)

Current density ( $\mu\text{A}/\text{cm}^2$ )

	<u>MTS</u>	<u>ISIS</u>	<u>APT</u>
Target material	Ta-clad W	Ta-clad W	SS-clad W
Coolant	D <sub>2</sub> O	D <sub>2</sub> O	H <sub>2</sub> O
Beam energy (MeV)	800	800	800
Beam current (mA)	1.35	0.2	1
Current density ( $\mu\text{A}/\text{cm}^2$ )	75	10	70

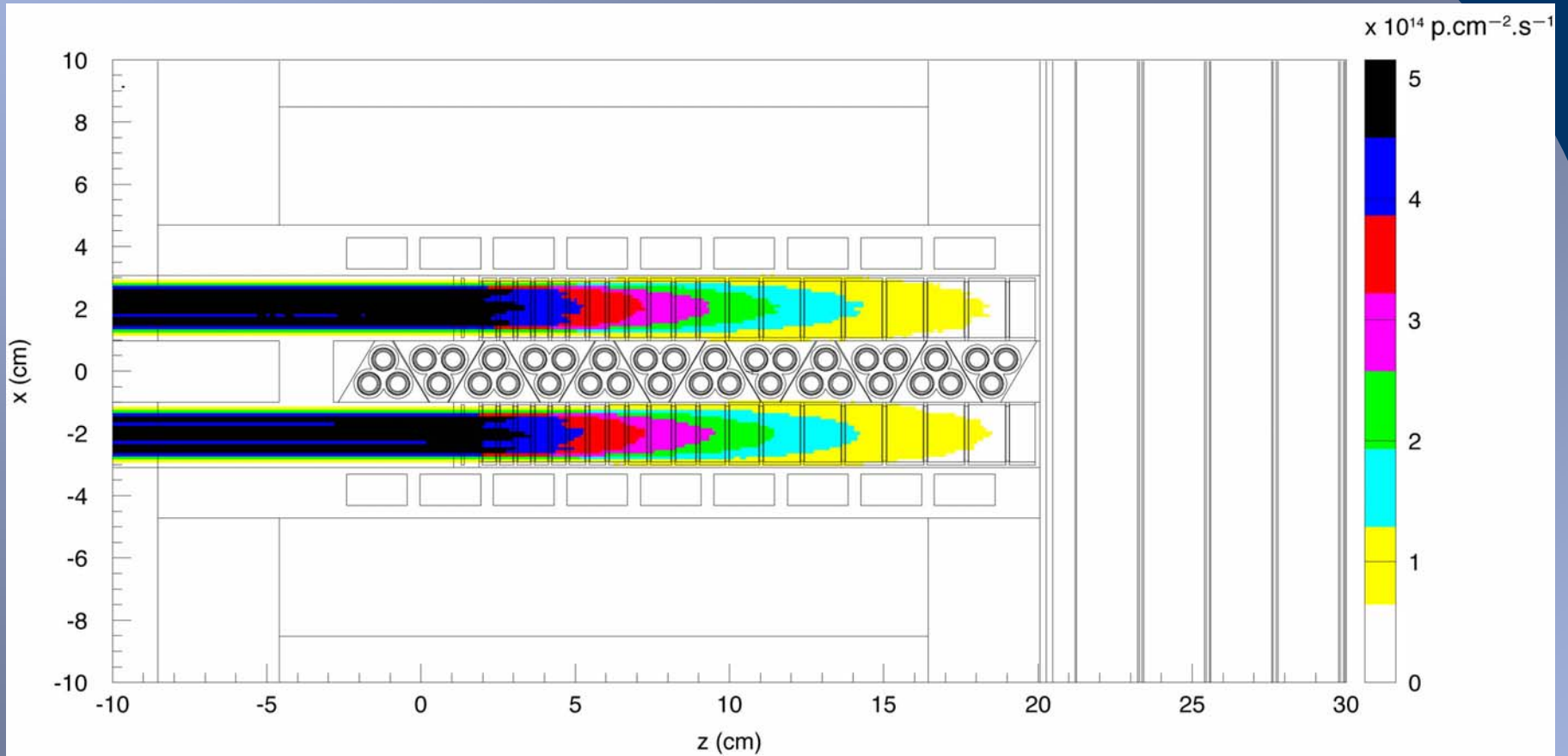


MTS target



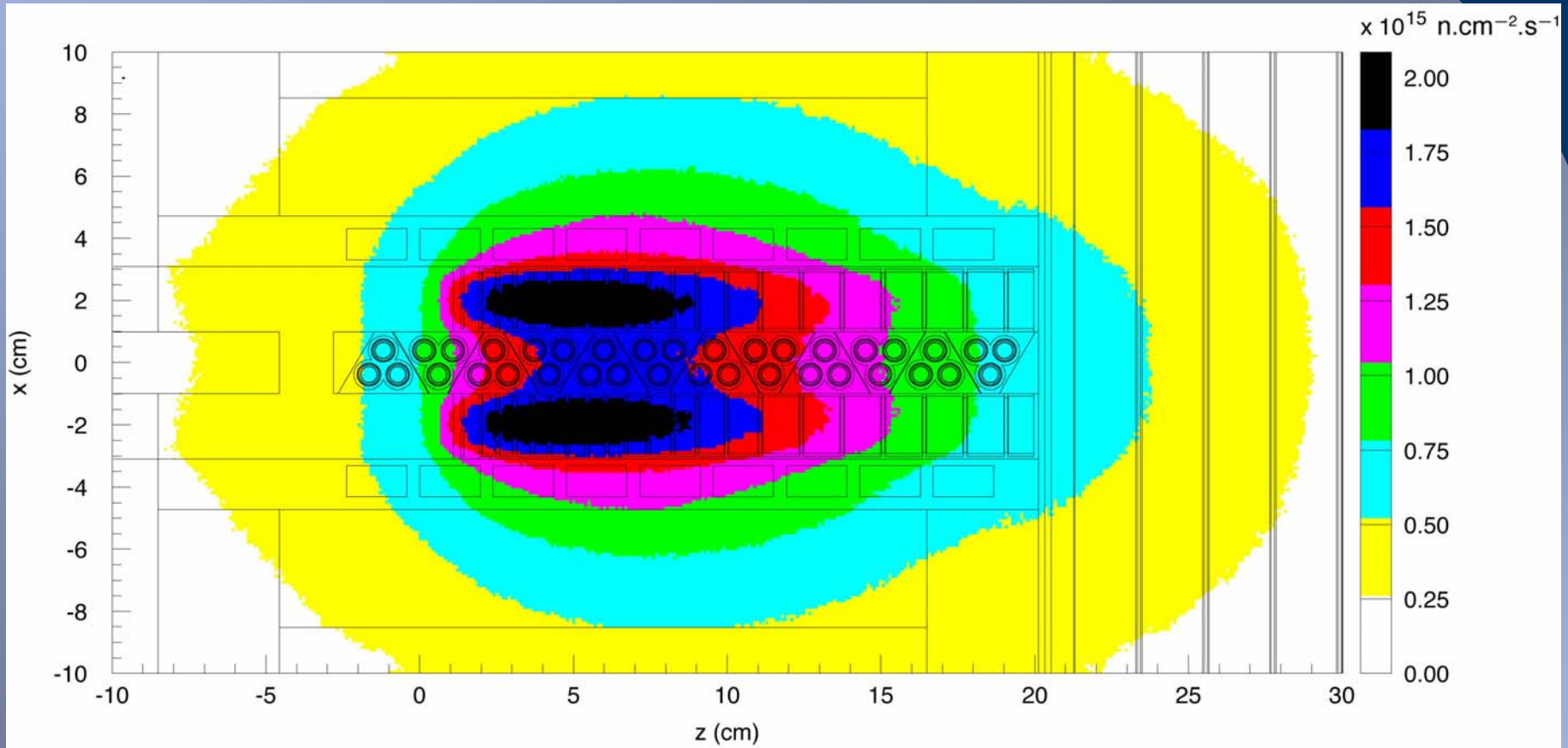
ISIS target

# Proton flux distribution at target mid-plane

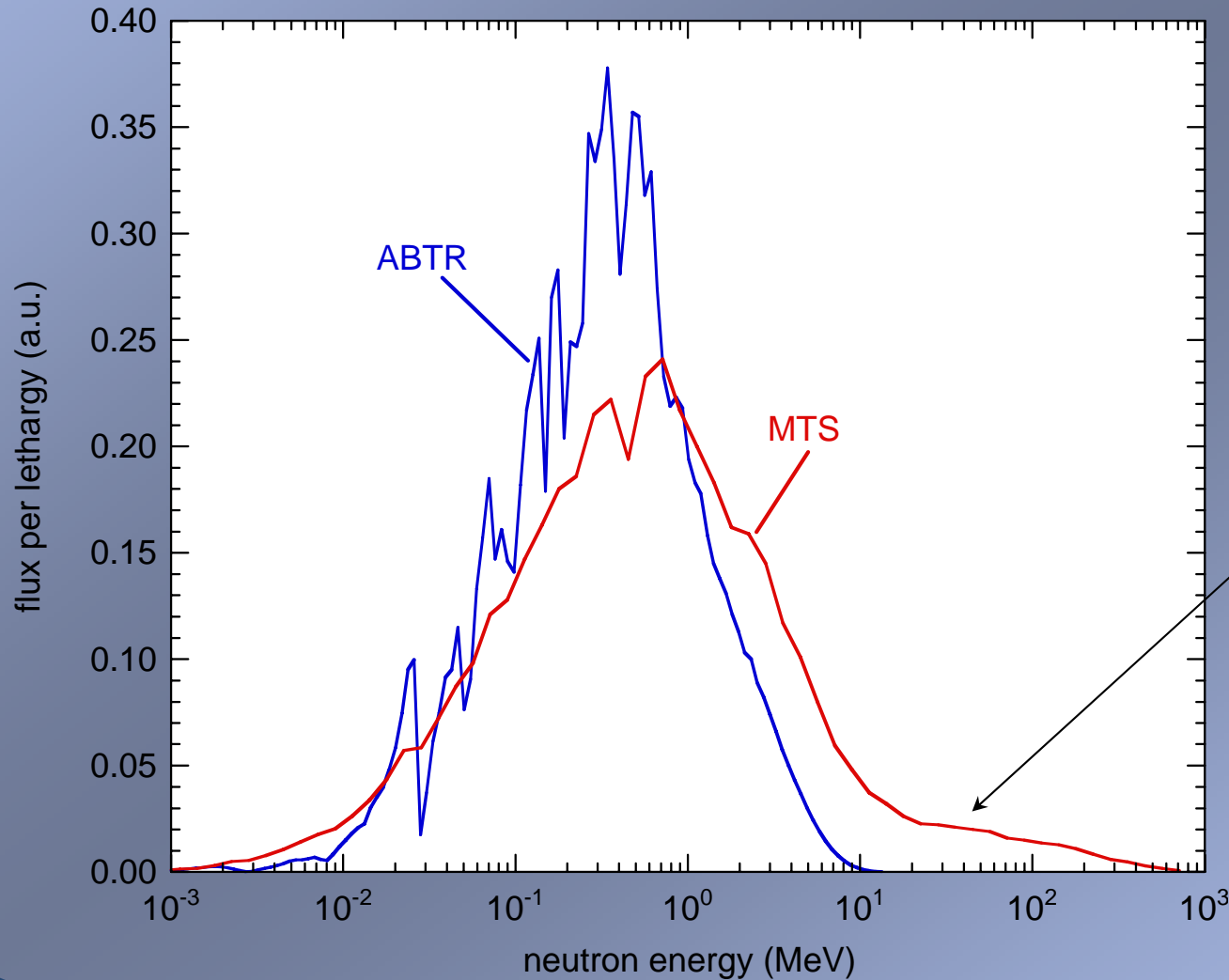




# Neutron flux distribution at target mid-plane



*MTS neutron spectrum is similar to that of a fast reactor with the addition of a high-energy tail*

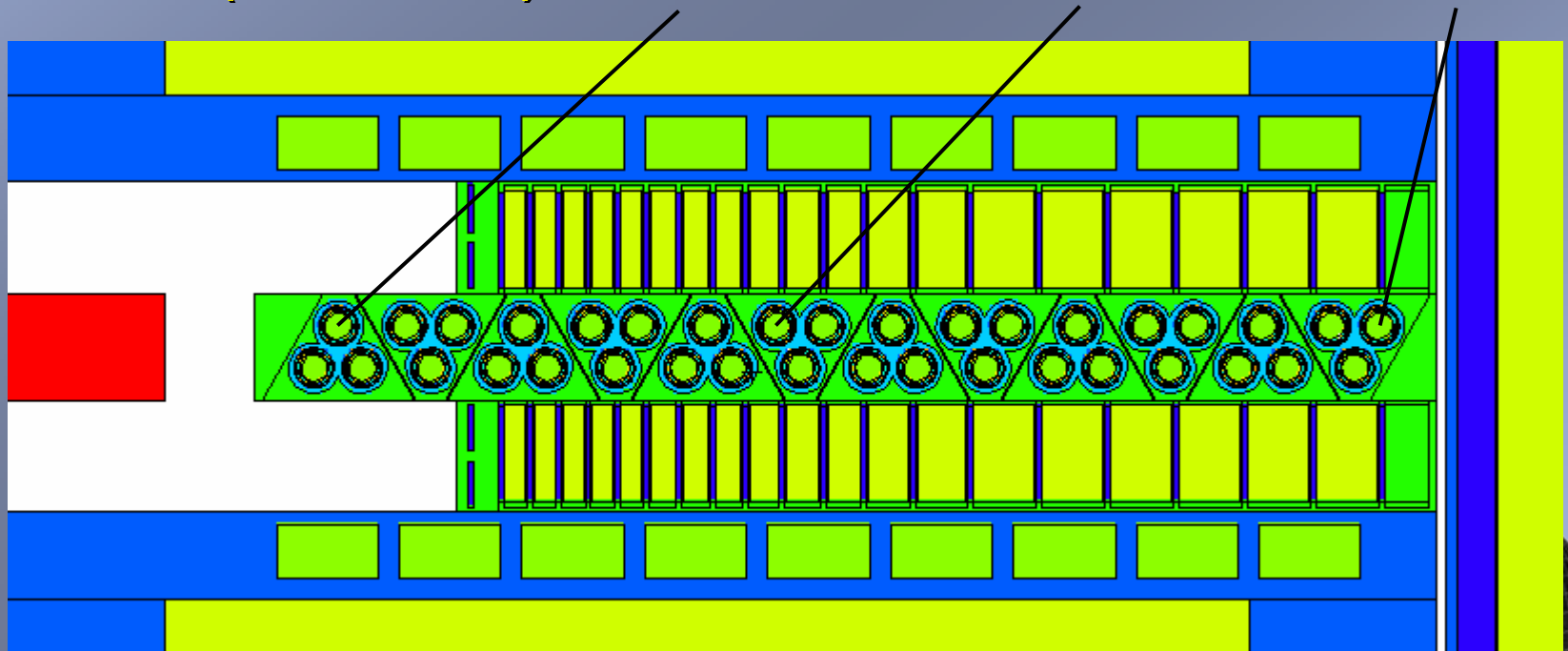


**Only 6% of  
neutron flux  
is >10 MeV**

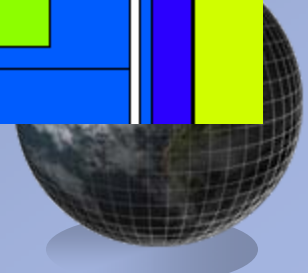


*Compared to a fast reactor, the high-E tail of the MTS spectrum causes greater He production in Fe*

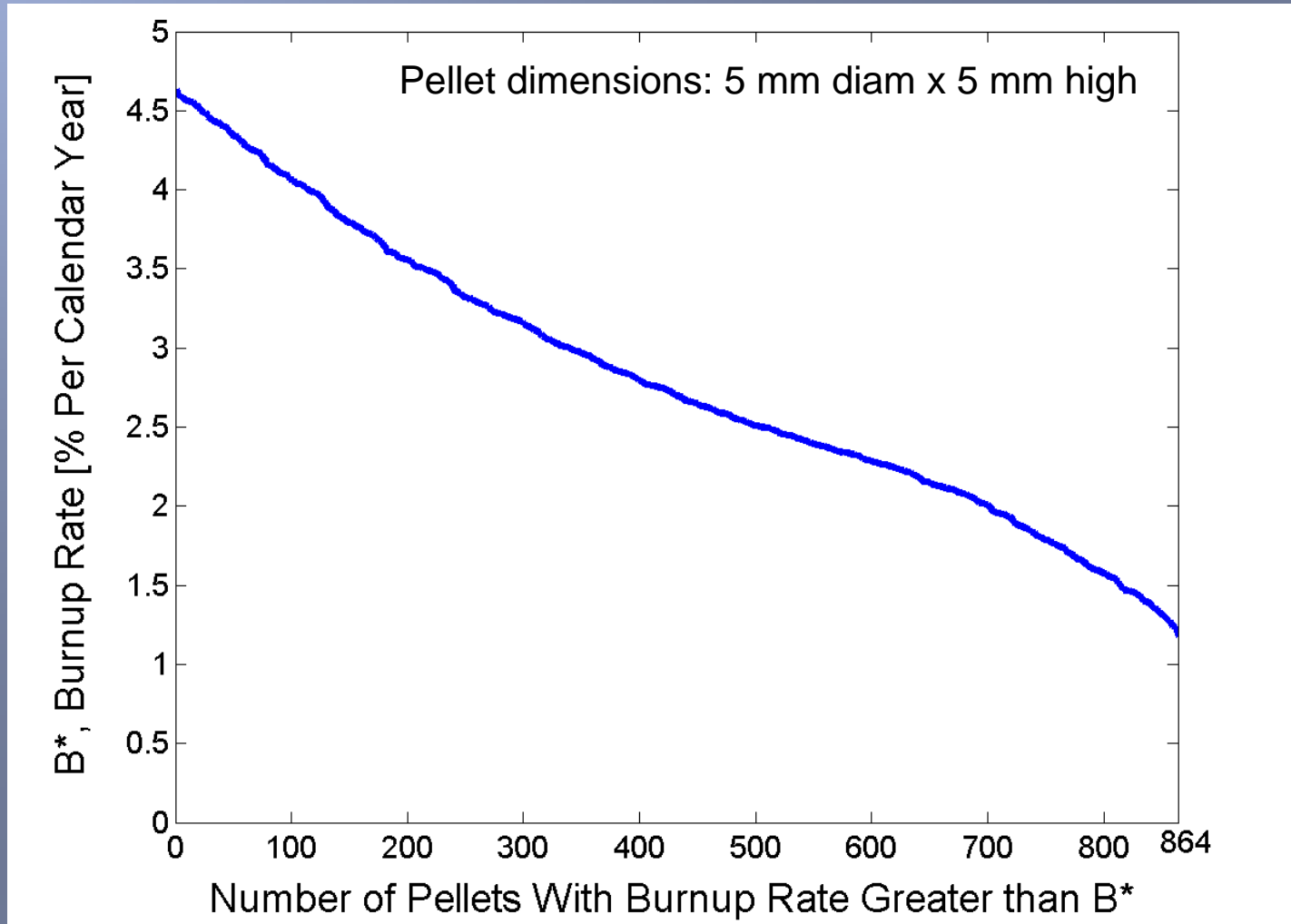
He production (appm/y)*	8.6	226	177
Atomic displacements (dpa/y)*	3.9	17.9	8.1
He/dpa	2.2	12.6	21.7
Burnup (%/y)*	1.4	4.6	2.2
Neutron flux ( $10^{14}$ n/cm <sup>2</sup> /s)	5.1	16.7	8.1



\*Per calendar year (4,400 hours of full-power operation).



*Burnup rate histogram, 864 fuel pellets  
(HEU-WG<sub>2</sub>Pu oxide fuel / Ta housing, H<sub>2</sub>O coolant)*



# The peak flux in MTS shows little dependence on test fuel composition

**Table. Composition [w/o] of MTS High-Enrichment Oxide and Four Futurix Fuel Forms**

	[HEU-WGPu]O <sub>2</sub>	TRU-Zr Metal	[TRU-Zr]N	DU-TRU-Zr Metal	[DU-TRU-Zr]N
U, UN or UO <sub>2</sub>	70	0	0	35	50
Pu, PuN or PuO <sub>2</sub>	30	48	32	29	25
Np, NpN or NpO <sub>2</sub>	0	0	0	2	10
Am, AmN or AmO <sub>2</sub>	0	12	32	4	15
Zr or ZrN	0	40	36	30	0
Density [g/cc]	11.0	9.7	8.3	11.5	11.3

**Peak Burnup Rate [% per Effective Full Power Month]**

	MTS	PHENIX *
[HEU-WGPu]O <sub>2</sub>	0.77	-
TRU-Zr Metal	0.69	1.43
[TRU-Zr]N	0.58	1.28
DU-TRU-Zr Metal	0.40	0.88
[DU-TRU-Zr]N	0.42	0.69

\*From Jaecki, et al., Global 2005.

**Peak Neutron Flux [n/cm<sup>2</sup>/s] - Materials Test Station**

	Total	Fast [ > 0.1 MeV]
[HEU-WGPu]O <sub>2</sub>	1.68E+15	1.33E+15
TRU-Zr Metal	1.63E+15	1.28E+15
[TRU-Zr]N	1.59E+15	1.23E+15
DU-TRU-Zr Metal	1.62E+15	1.26E+15
[DU-TRU-Zr]N	1.60E+15	1.25E+15

PHENIX 4th ring

1.92E+15



## *The schedule shows project completion near the end of FY10*

- Cleanout of Area A is estimated to take one year
- The total construction/installation process is estimated to take three years
- The actual completion date will depend on timely DOE Critical Decision approvals and adequate funding
- MTS Total Project Cost estimate is \$73M, including 25% contingency
- Operating cost is \$6M - \$10M per year

