



## Progress in Am transmutation targets from EFTTRA

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## The EFTTRA Collaboration: 15 years from 1992 to 2006

- Goal: transmutation of long-lived FP and MA
- Tc-99 and I- 129 irradiations performed and reported
- Inert matrices selection and irradiations in Phenix and HFR
- Pre-selection of  $MgAl_2O_3$  for Am targets transmutation
- Irradiations (Am) in HFR: T4 and T4bis (2 burn-up levels)
- Results not conclusive > Selection of Zirconia (**once-through scenario**)  
or MgO, Mo (**multiple recycling scenario**)
- Preparation of HELIOS irradiation in HFR
- Further developments



## **EFTTRA: a 15-years successful collaboration**

- Agreement on experimental programmes, seminars, publications
- Financing by the partners (case by case)
- Financial (partial) support through EC Framework Programmes:
- FP5 (EFTTRA T4)
- FP6 (EUROTRANS IP: HELIOS, BODEX)



## EFTTRA T4 and T4bis Objective

- $\text{MgAl}_2\text{O}_3$  spinel as IMF for **once-through transmutation**
- Development of Am spinel target fabrication
- First irradiation testing of Am spinel
- In-pile behaviour, transmutation demonstration
- Burn-up effect (two irradiation times)



## EFTTRA T4 and T4bis main conclusions

- Am spinel fabrication demonstrated, but improvements implemented (matrix powder infiltration instead of green pellet infiltration)
- Irradiation successful (up to 650 days): no failure, good pellet structure
- Very high gas production (mainly He), low release (low T°)
- Very large pellet swelling (gas bubbles)
- Am-Spinel chemical instability: formation of  $\text{AmAlO}_3$

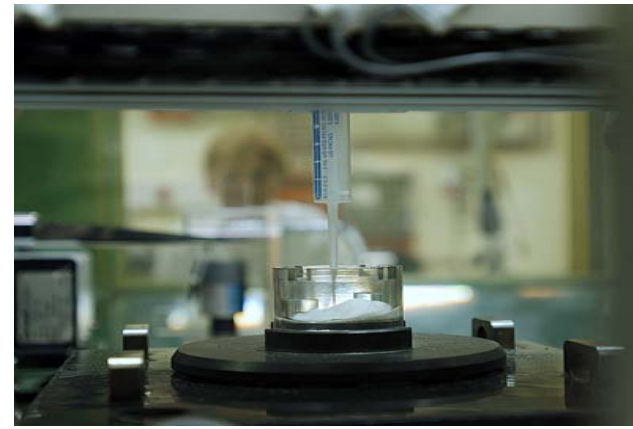
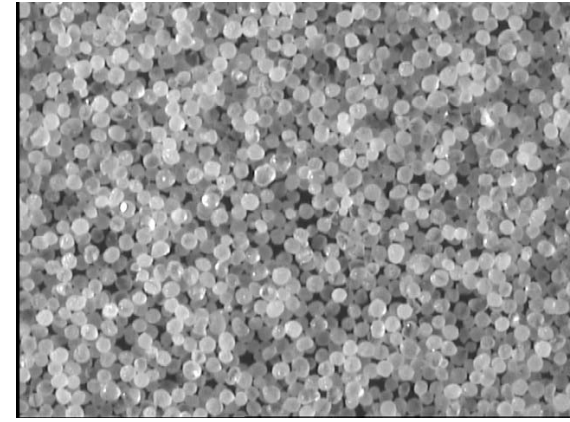
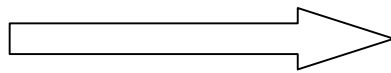
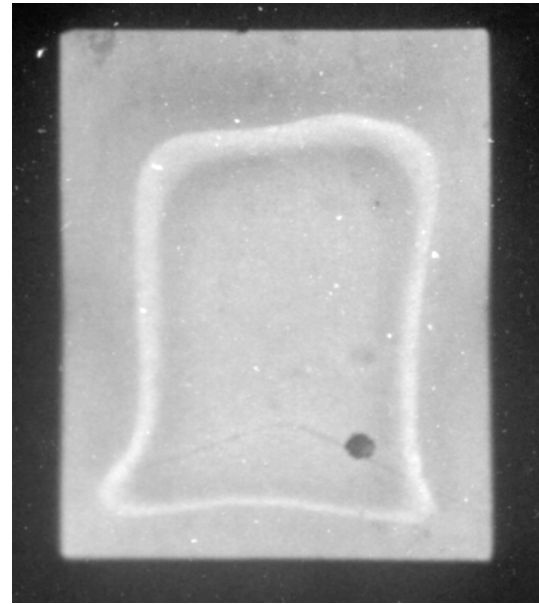


**EFTTRA-T4(bis) pellets: green pellet infiltration**

**HELIOS: sol-gel beads infiltration**

Inhomogeneous Am distribution in spinel

Solid-solution formation in the sintered pellet



MA-Laboratory



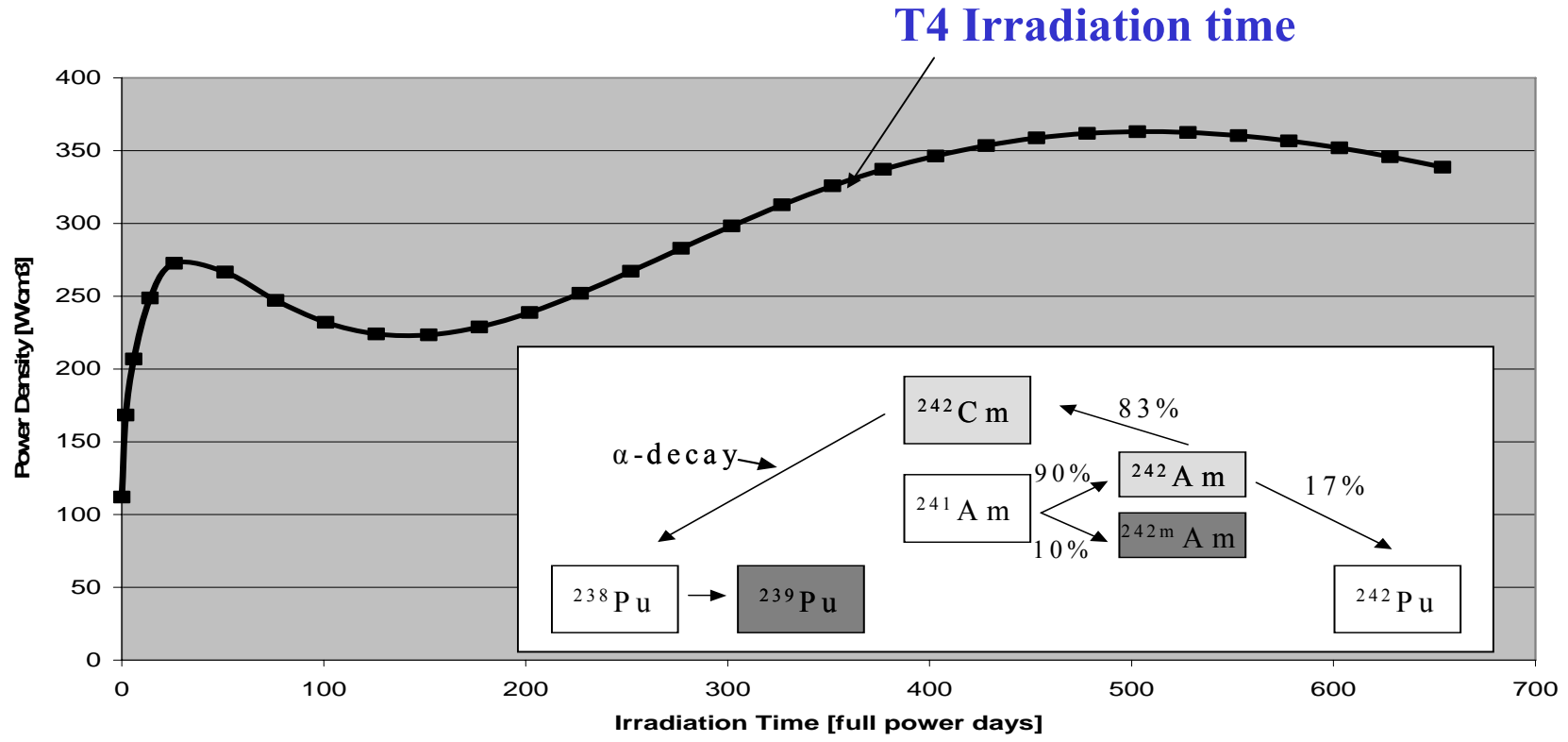
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# Power density in the EFTTRA T4bis irradiation

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The two maxima are due to formation and fission of Am-242m and Pu-239, respectively. The EFTTRA T4 irradiation showed a similar power profile, but it did not reach the second maximum due to the shorter irradiation time.

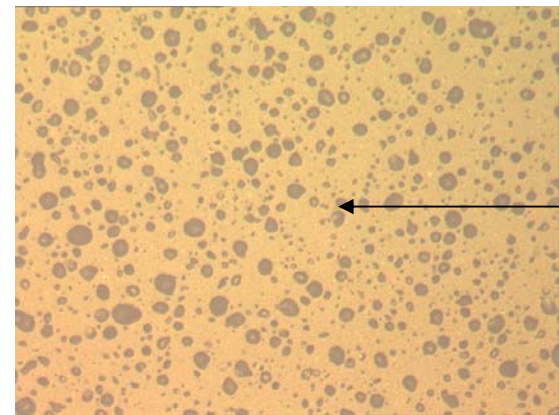
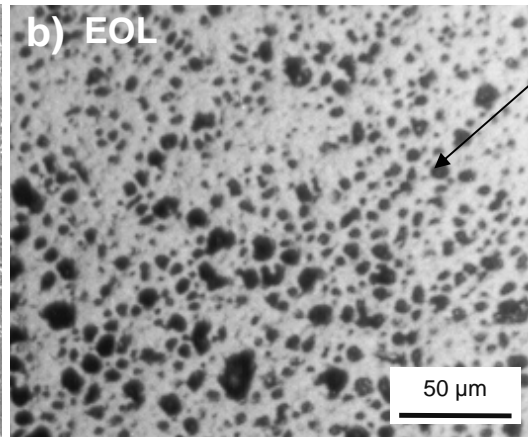
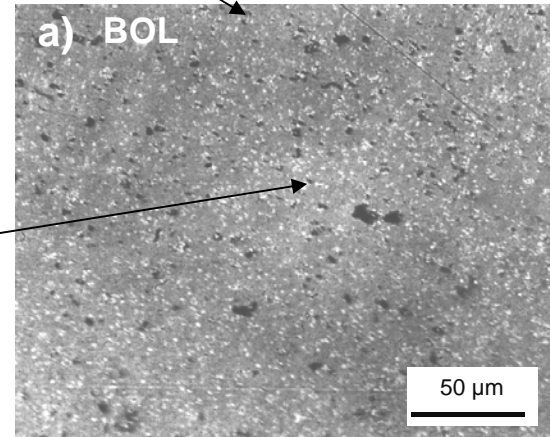




As-fabricated T4 fuel

Irradiated T4: 17% porosity

AmAlO<sub>3</sub>



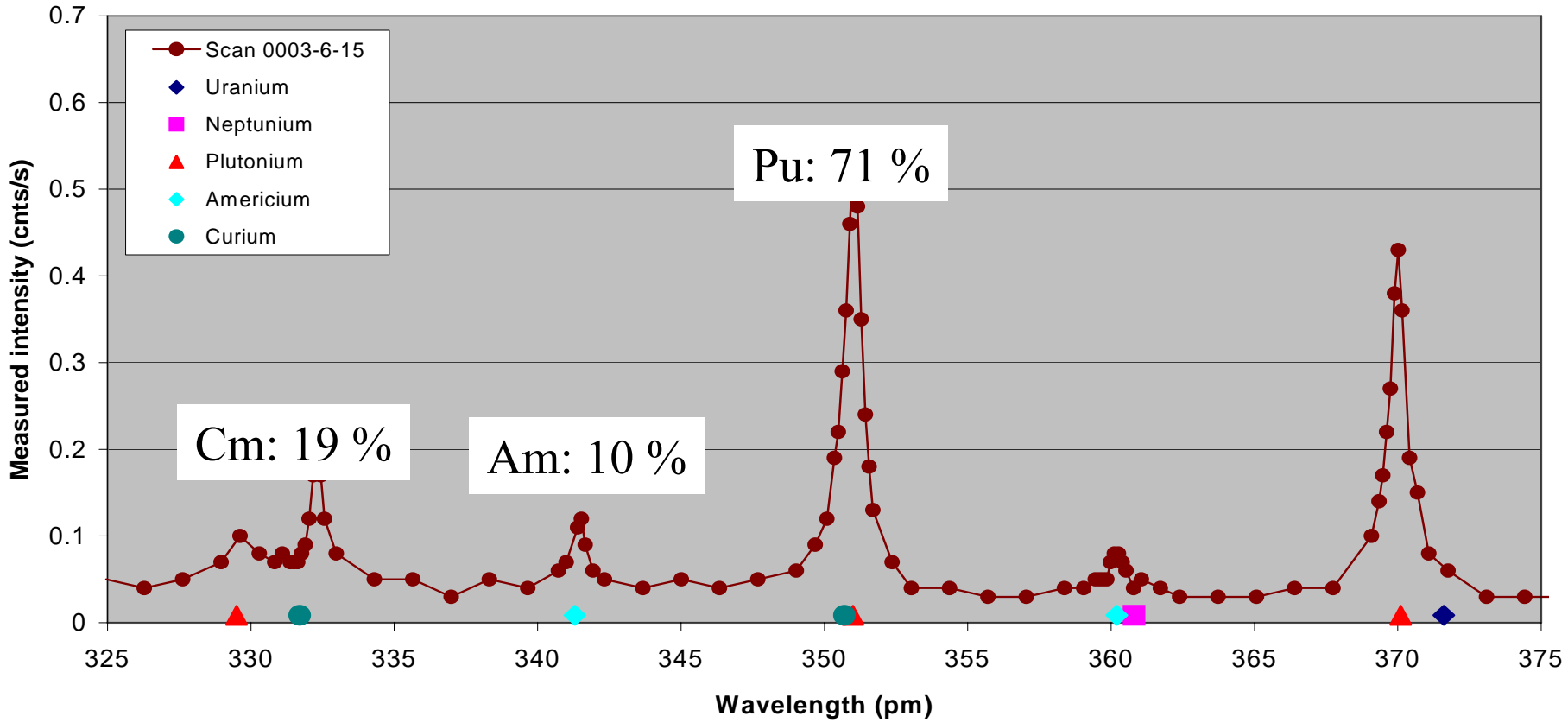
Irradiated T4bis: 25% porosity



# T4bis Electron Probe Micro-Analysis

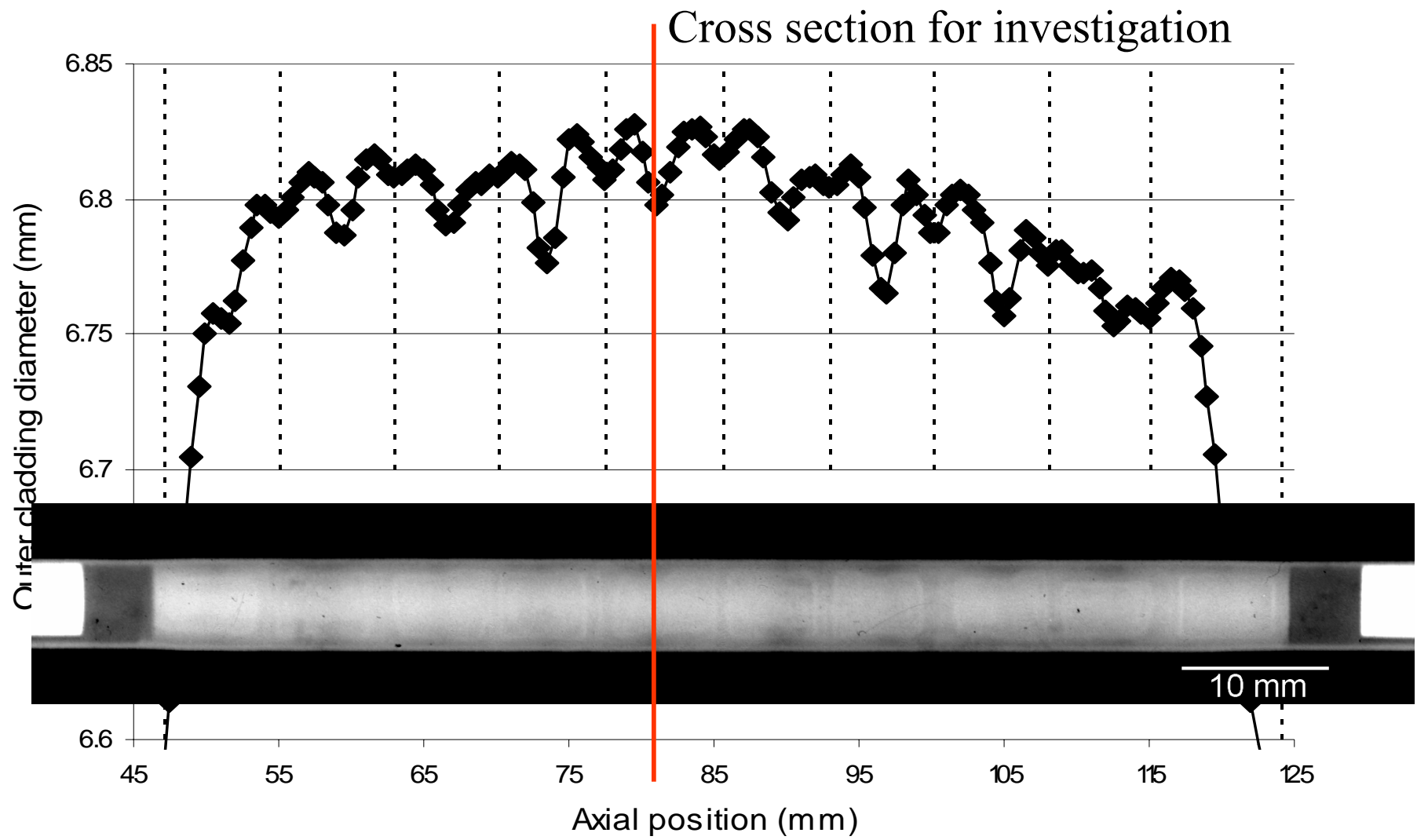
Composition of the 50% actinides remaining in the fuel

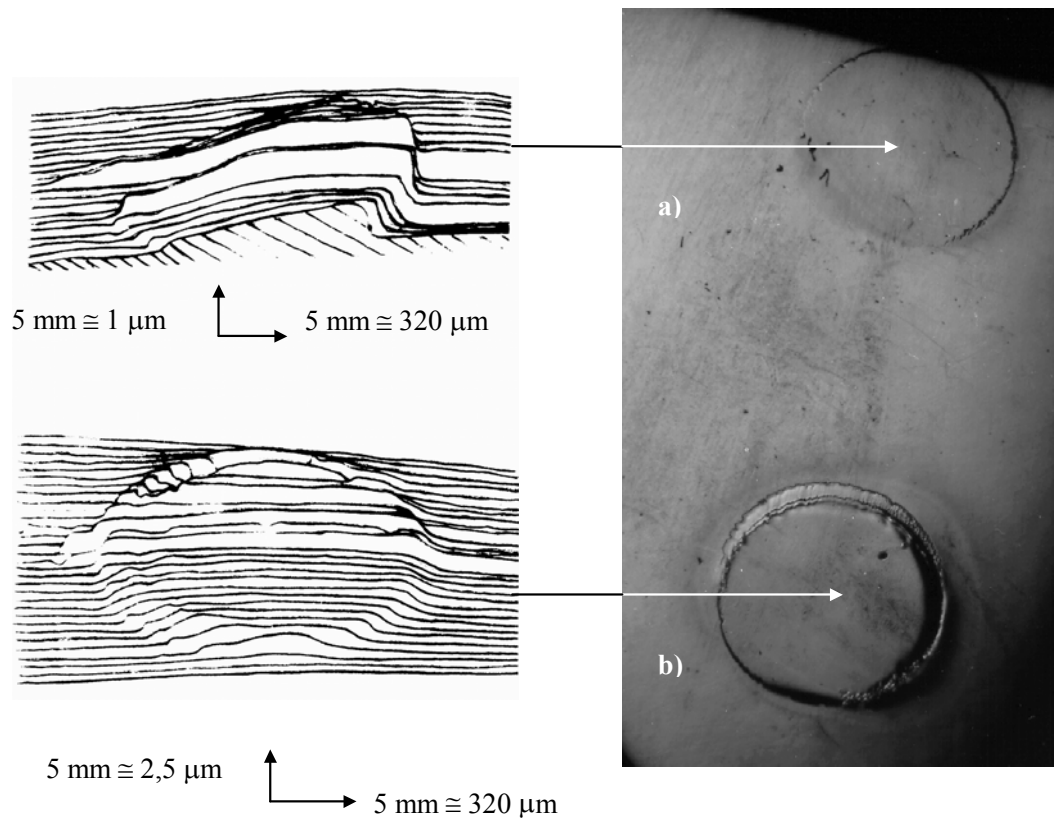
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# EFTTRA T4bis Profilometry and X-ray image

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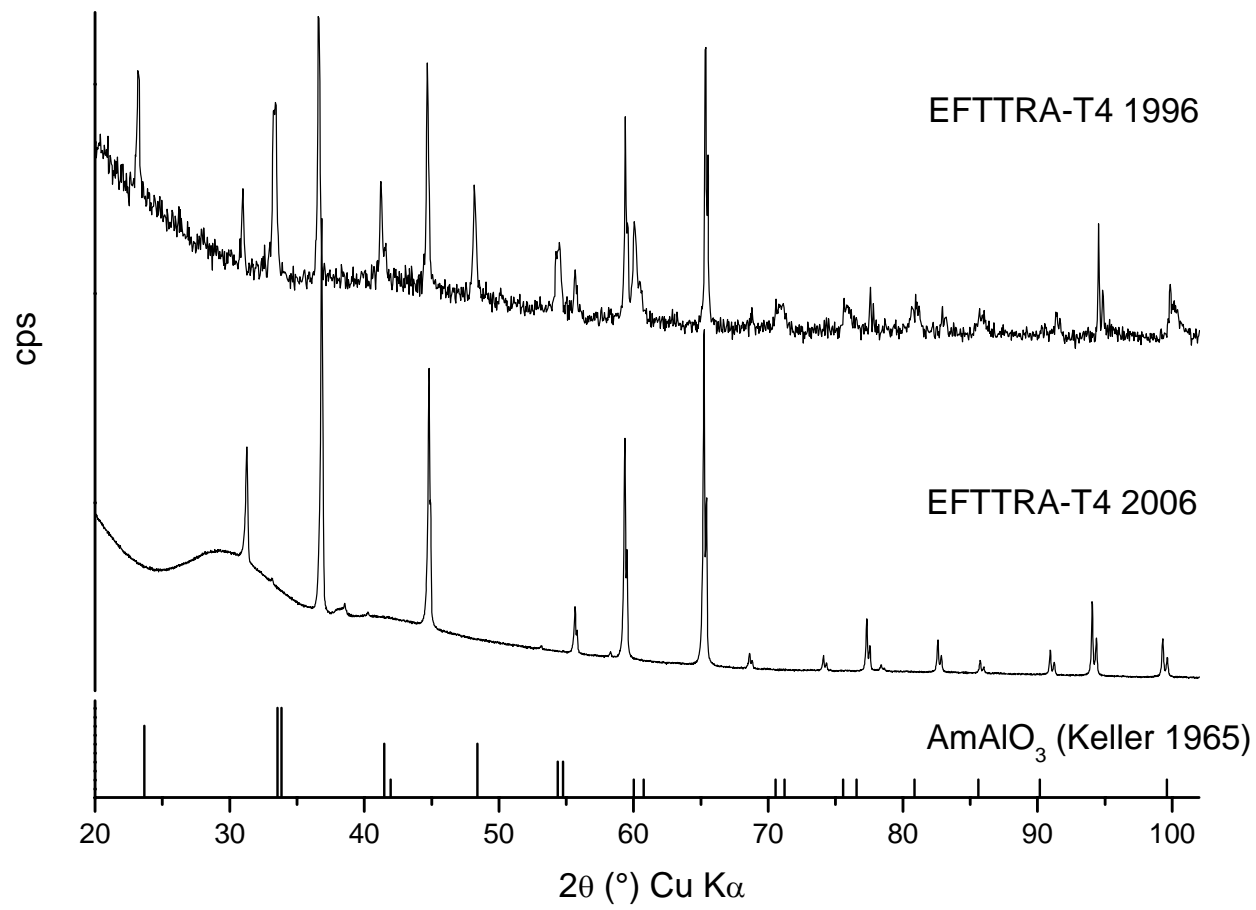
*Profilometry and optical micrograph of an MgAl<sub>2</sub>O<sub>4</sub> single crystal <110> orientated irradiated with iodine ions of 72 MeV energy. The respective fluences are 1015 for a) and 1017 ions/cm<sup>2</sup> for b). The optical micrograph shows clearly the irradiated area popping out from the original crystal surface.*



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# X-ray diffraction of EFTTRA-T4 fuel



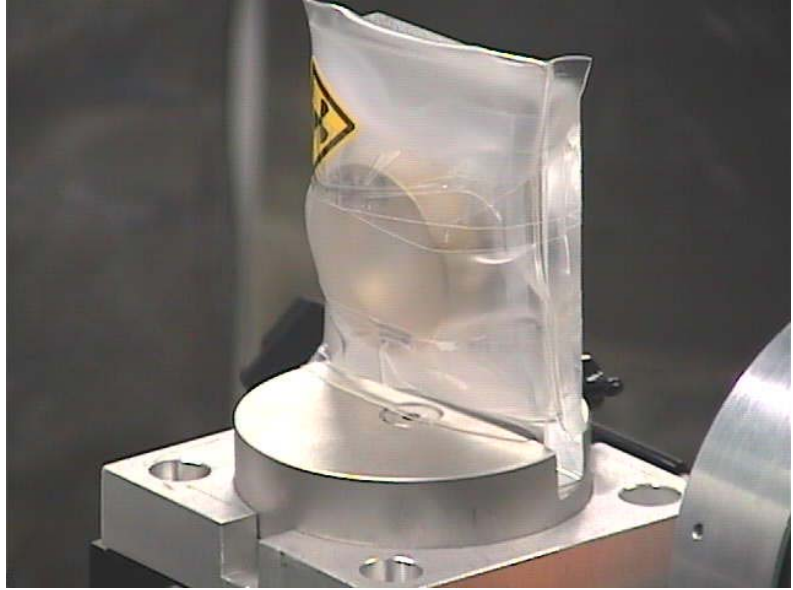
spinel +  
Am phase similar  
to AmAlO<sub>3</sub>

spinel (crystalline!)  
no Am phase



# EXAFS measurement

- annealing at 1600°C in Ar/H<sub>2</sub>  
(as for production in 1996)
- about 60 mg Am-spinel powder in a steel shielded Plexiglass cuvette
- measurement at INE Beamline at the Ångströmquelle Karlsruhe, ANKA

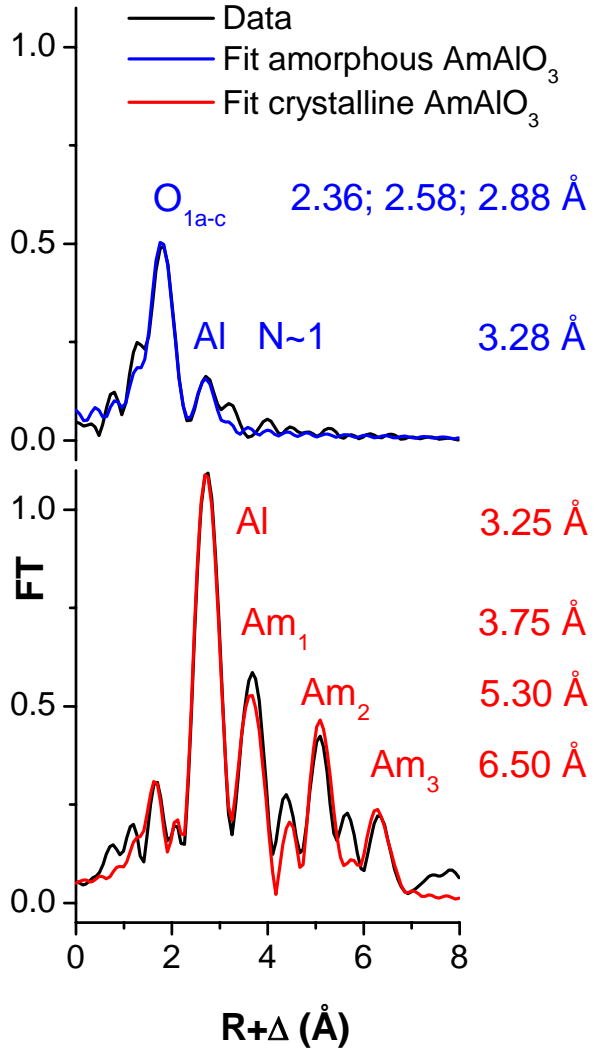


→ EXAFS provides information of local atomic structure:

- interatomic distance
- coordination number
- type of atom in coordination shells
- disorder



## Am L<sub>III</sub>-edge EXAFS of **aged** and **annealed** EFTTRA-T4



10 years **aged** EFTTRA-T4 fuel shows no peaks corresponding to a periodic structure

Am L<sub>3</sub>-edge EXAFS of **annealed** EFTTRA-T4 fuel can be fitted with AmAlO<sub>3</sub> structure

→ **AmAlO<sub>3</sub> is predominant in EFTTRA-T4 fuel**





## General conclusion related to the use of spinel as IMF

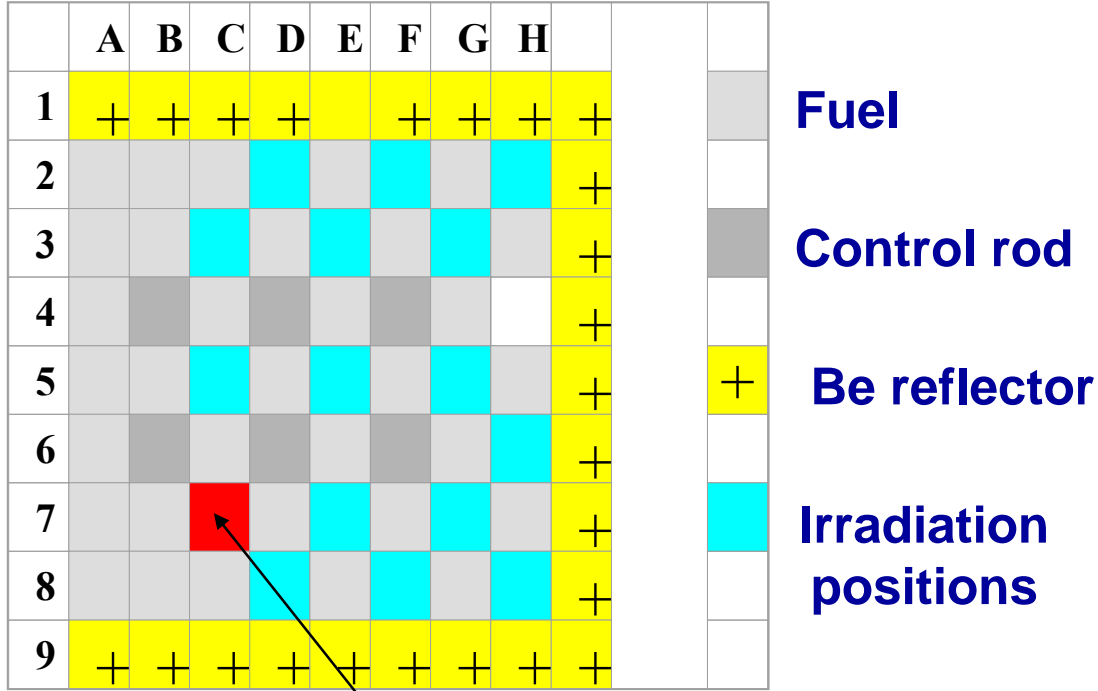
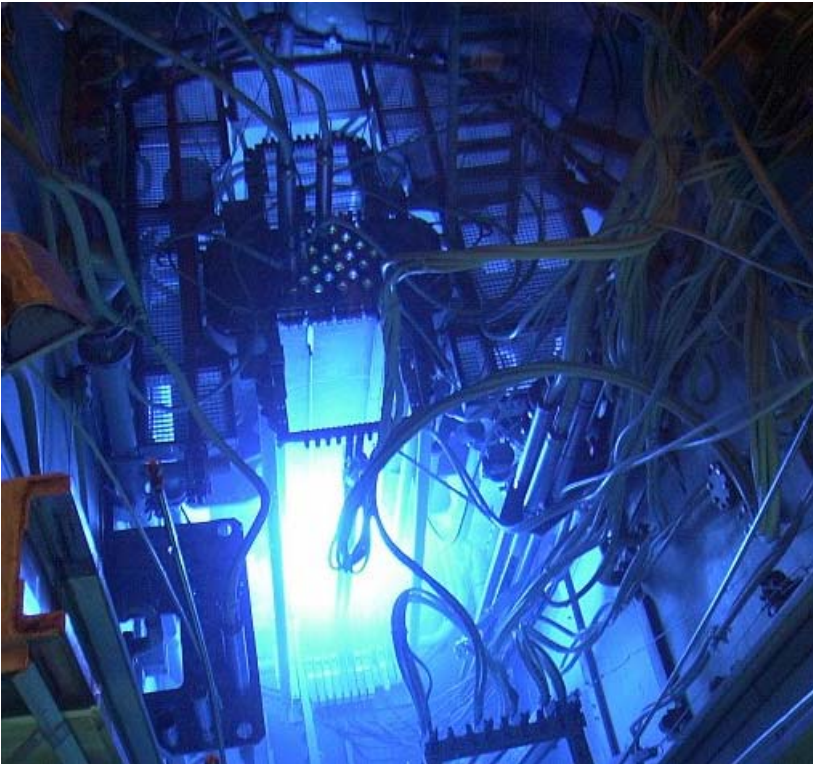
- Conclusion based on T4+T4bis + other experiments in HFR and Phenix
- At low temperature, He retention leading to swelling
- At high temperature He release, but chemical instability
- Study started of a more stable IMF (once-through): Y-stabilised Zirconia
- Additional investigation of MgO and Mo for recycle strategies
- Need of irradiation experiments: **HELIOS in HFR**



## EUROTRANS IP (FP6) Irradiation Tests

- **FUTURIX**: irradiation test in Phénix of TRU-fuels under EFIT relevant conditions
- **BODEX**: irradiation test in HFR to study the helium buildup and release from inert matrixes for IMF's
- **HELIOS**: irradiation test in HFR to study the in-pile behaviour vs temperature of U-free Am targets

# HFR-Petten



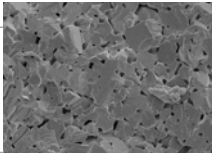
**HELIOS**



# HELIOS test matrix



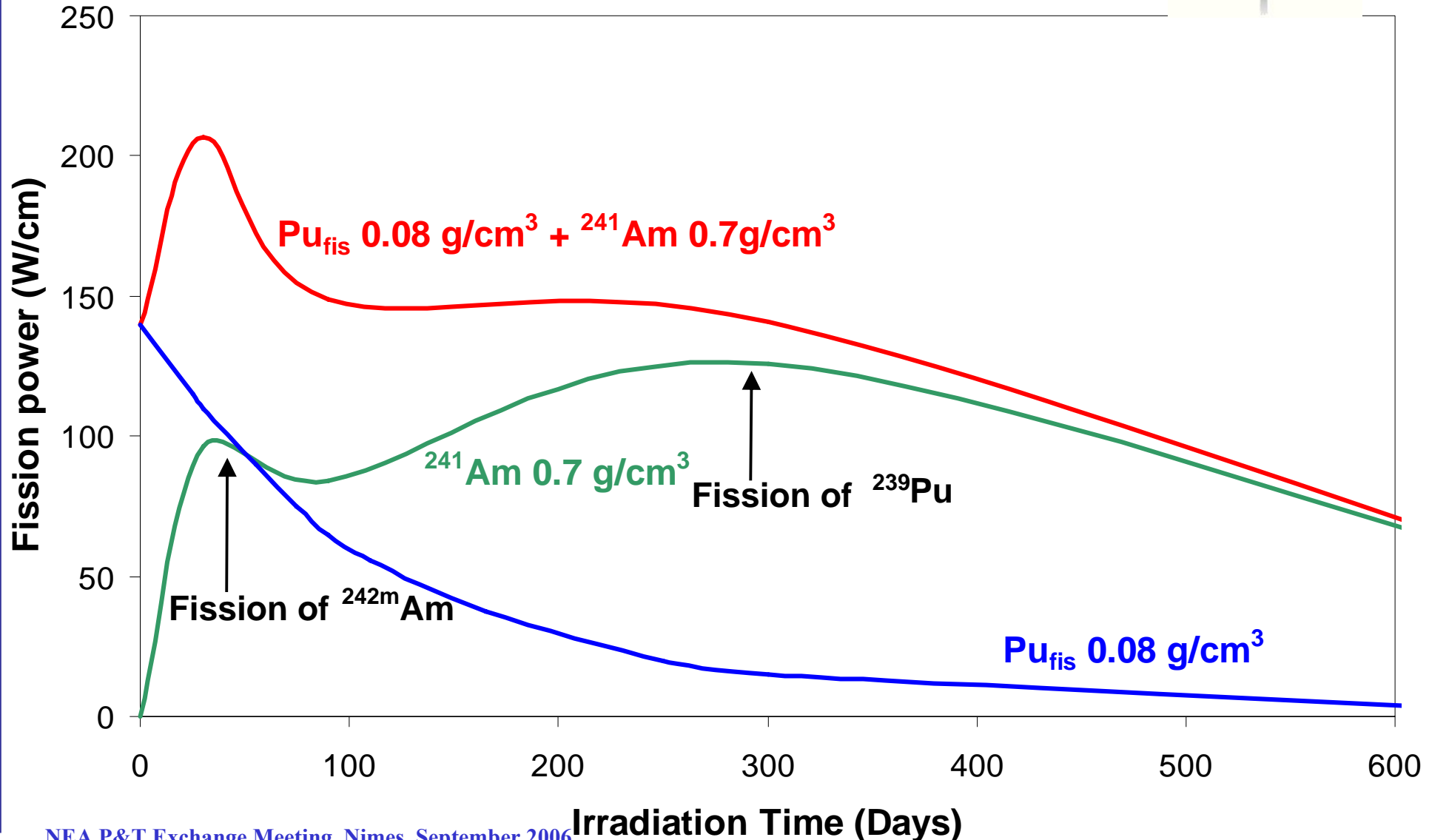
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Pin Nr	Composition	Microstructure	As-fabricated density [g/cm <sup>3</sup> ]		Fuel Manufacturer	Fuel Type	Remarks
			<sup>241</sup> Am	Pu <sub>tot</sub>			
1	<b>Am<sub>2</sub>Zr<sub>2</sub>O<sub>7</sub>+MgO</b>	10-50 μm	0.7	0	CEA	Cer Cer	
2	<b>(Am,Zr,Y)O<sub>2</sub></b>	Solid solution	0.7	0	JRC-ITU		Instrumented with Central TC
3	<b>(Am,Pu,Zr,Y)O<sub>2</sub></b>	Solid solution	0.7	0.39	JRC-ITU		Instrumented with Central TC
4	<b>(Zr,Am,Y)O<sub>2</sub>+Mo</b>	60-120 μm max 30 vol%	0.7	0	JRC-ITU	Cer Met	
5	<b>(Pu,Am)O<sub>2</sub>+Mo</b>	40-150 μm max 30 vol%	0.3	1.2	JRC-ITU	Cer Met	

# Power calculations



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# Estimated temperatures



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Pin	Composition	Pu-content [g/cm <sup>3</sup> ]	Am-content [g/cm <sup>3</sup> ]	Max Power [W/cm <sup>3</sup> ]	Linear Power [W/cm]	T <sub>central</sub> [°C]	Calculated with power [W/cm <sup>3</sup> ]
1	Am <sub>2</sub> Zr <sub>2</sub> O <sub>7</sub> + MgO	0	0.7	240	56	741	300
2	(Am,Zr,Y)O <sub>2</sub>	0	0.7	170	40	811	230
3	(Pu,Am,Zr,Y)O <sub>2</sub>	0.39	0.7	740	170	1439	787
4	(Am,Zr,Y)O <sub>2</sub> + Mo	0	0.7	250	58	687	300
5	(Pu,Am)O <sub>2</sub> + Mo	1.2	0.3	1600	370	1477	2020

➤ **Onset of He release: ≈ about 550 °C**

➤ **Complete He release: ≈ about 1350 °C**



# (Am,Zr,Y)O<sub>2</sub> pre-fabrication tests



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**Sintered density > 90%TD**



## HELIOS Status

- HELIOS irradiation shall start in 2007 in the HFR and last for about 1 year (i.e. 10 HFR cycles)
- Fuel production and hardware procurement & manufacturing underway
- Predicted temperatures in the fuels are acceptable (slightly lower than aimed)
- Destructive PIE's results shall be available in 2009





## **Future EFTTRA plans**

- **Complete HELIOS experiment within EUROTRANS IP**
- **Integrate the results within other experiments: BODEX**
- **Demonstrate the high Am transmutation capability of Zirconia IMF with a 5-years irradiation in HFR: HELIOSbis (not decided – funding dependent)**
- **Looking at the feasibility of Cm transmutation in once-through mode**