

Features needed for traditional evaluation methods at CNDC

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CNDC/SG50

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Required from Arjan:

1. experimental covariances?
2. how you currently deal with that?
3. how you would like to see that handled in EXFOR?

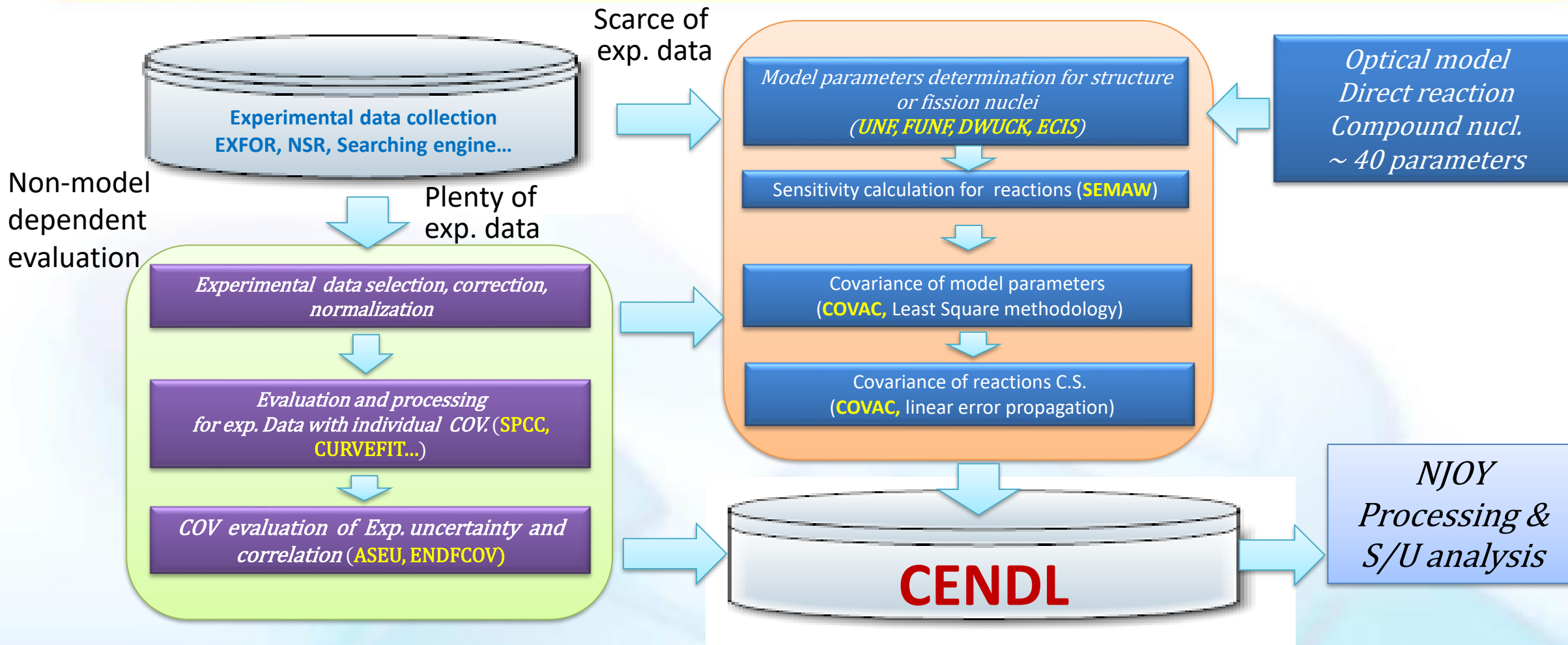


Four issues

1. Traditional evaluation methods
2. Samples to deal with the experimental covariance
3. China's contribution to EXFOR compilation
4. Some considerations to future EXFOR



1. traditional evaluation methods



Correlations among single (or multiple) set(s) of experimental data are vital elements to get an 'honest' covariance.

1. traditional evaluation methods

Descriptions to the current COV scheme at CNDC:

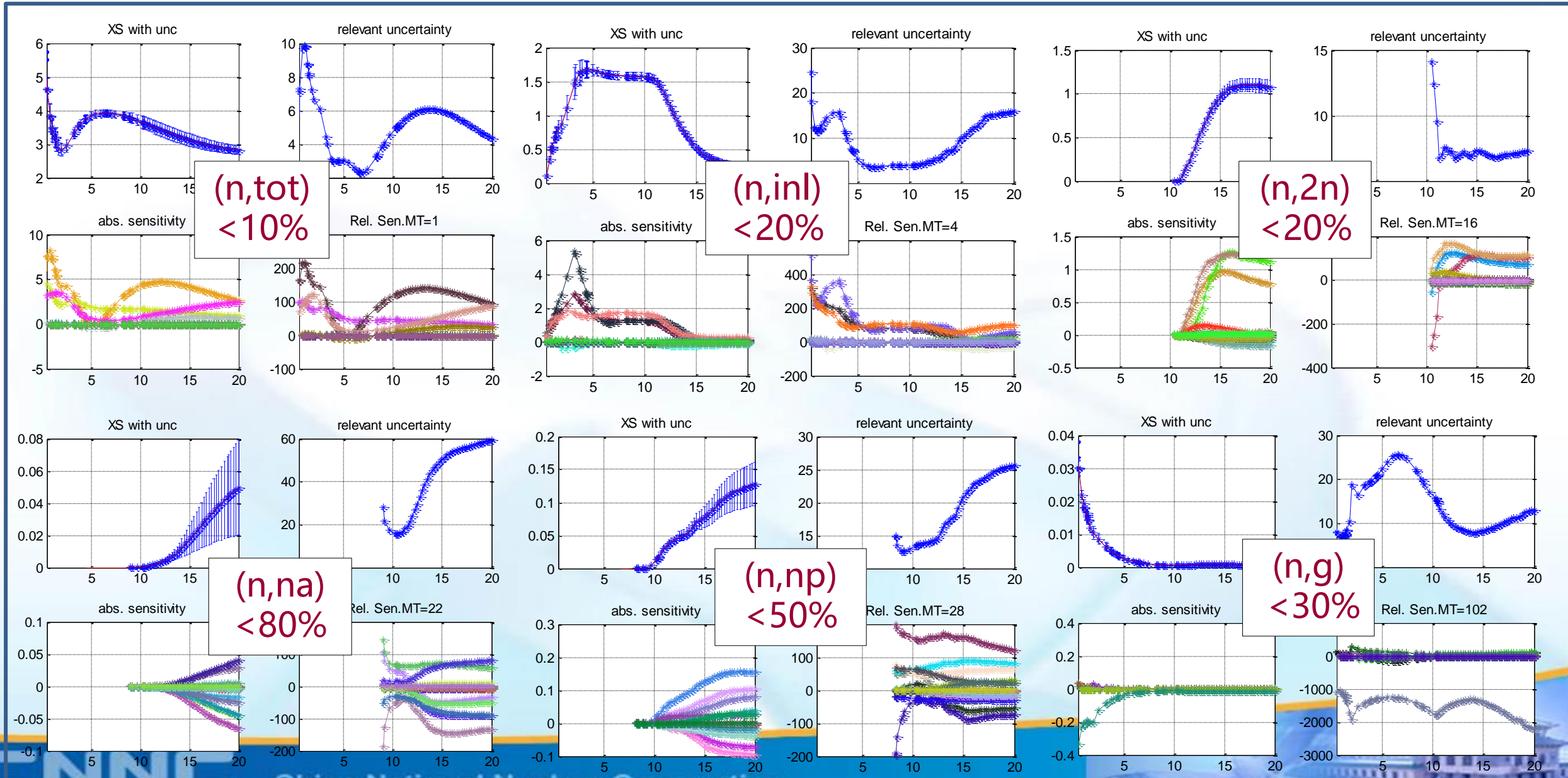
- ① Tech. for **non-model & model** dependent
- ② Energies for **structure & smooth** regions
- ③ COV data types for **NI & NC**
- ④ Tech. deal with **single & multiple** measurements
- ⑤ Tech. for parameter sensitivity selection
- ⑥ COV matrix positive definition treatment

(n,tot) (n,tot)	(n,tot) (n,inl)	(n,tot) (n,y)	(n,tot) (n,p)	(n,tot) (n,d)	(n,tot) (n,t)	(n,tot) (n,2n)	(n,tot) (n,np)	(n,tot) (n,nα)
↑ NM	(n,inl) (n,inl)	(n,inl) (n,y)	(n,inl) (n,p)	(n,inl) (n,d)	(n,inl) (n,t)	(n,inl) (n,2n)	(n,inl) (n,np)	(n,inl) (n,nα)
		(n,y) (n,y)	(n,y) (n,p)	(n,y) (n,d)	(n,y) (n,t)	(n,y) (n,2n)	(n,y) (n,np)	(n,y) (n,nα)
			(n,p) (n,p)	(n,p) (n,d)	(n,p) (n,t)	(n,p) (n,2n)	(n,p) (n,np)	(n,p) (n,nα)
			↑ NM	(n,d) (n,d)	(n,d) (n,t)	(n,d) (n,2n)	(n,p) (n,np)	(n,p) (n,nα)
				(n,t) (n,t)	(n,t) (n,2n)	(n,t) (n,np)	(n,t) (n,nα)	(n,t) (n,nα)
						(n,2n) (n,2n)	(n,2n) (n,np)	(n,2n) (n,nα)
9个反应道以及彼此之间的关联						↑ NM	(n,np) (n,np)	(n,np) (n,nα)
								(n,nα) (n,nα)

(n,tot) (n,tot)	(n,tot) (n,inl)	(n,tot) (n,y)	(n,tot) (n,p)	(n,tot) (n,d)	(n,tot) (n,t)	(n,tot) (n,2n)	(n,tot) (n,np)	(n,tot) (n,nα)
	(n,inl) (n,inl)	(n,inl) (n,y)	(n,inl) (n,p)	(n,inl) (n,d)	(n,inl) (n,t)	(n,inl) (n,2n)	(n,inl) (n,np)	(n,inl) (n,nα)
	↑ MODEL	(n,y) (n,y)	(n,y) (n,p)	(n,y) (n,d)	(n,y) (n,t)	(n,y) (n,2n)	(n,y) (n,np)	(n,y) (n,nα)
		↑ MODEL	(n,p) (n,p)	(n,p) (n,d)	(n,p) (n,t)	(n,p) (n,2n)	(n,p) (n,np)	(n,p) (n,nα)
				(n,d) (n,d)	(n,d) (n,t)	(n,d) (n,2n)	(n,p) (n,np)	(n,p) (n,nα)
				↑ MODEL	(n,t) (n,t)	(n,t) (n,2n)	(n,t) (n,np)	(n,t) (n,nα)
					↑ MODEL	(n,2n) (n,2n)	(n,2n) (n,np)	(n,2n) (n,nα)
9个反应道以及彼此之间的关联						↑ MODEL	(n,np) (n,np)	(n,np) (n,nα)
							↑ MODEL	(n,nα) (n,nα)

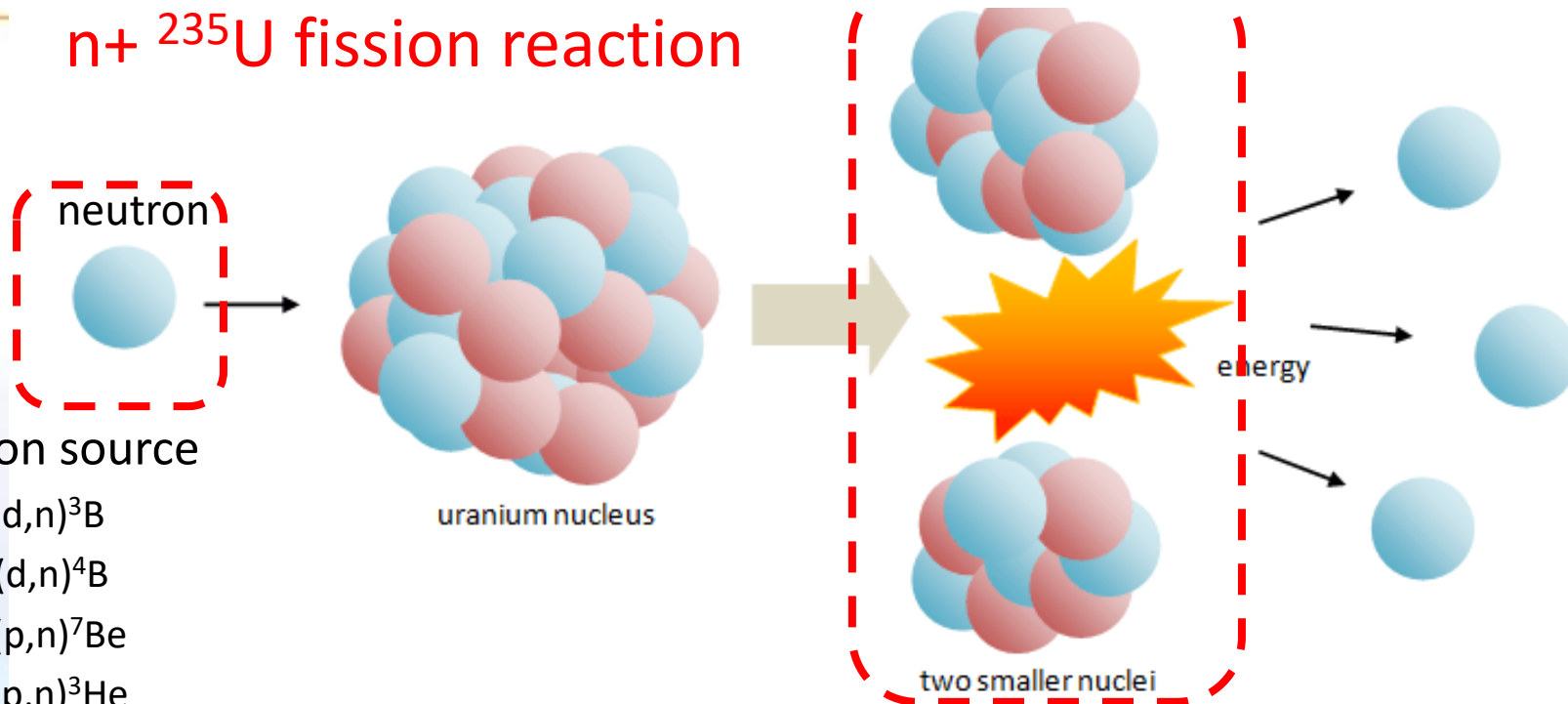
ENDF文件中, MF=32, 33包含19111行, 占到Zr90全套数据文件的1/3 or 1/4

CENDL-3.2 COV for main reactions of FP(70) , averaged unc with LS



2. Samples to deal with the exp. COV

$n + {}^{235}\text{U}$ fission reaction



Information from EXFOR

• Neutron source

D-D ${}^2\text{H}(d,n){}^3\text{H}$

D-T ${}^3\text{H}(d,n){}^4\text{He}$

P-Li7 ${}^7\text{Li}(p,n){}^7\text{Be}$

P-T ${}^3\text{H}(p,n){}^3\text{He}$

PHOTO Photoneutron

SPALL Spallation

P-Be $\text{Be}(p,n)\text{B}$

P-V51 ${}^{51}\text{V}(p,n){}^{51}\text{Cr}$

D-U

REAC Reactor

EVAP Evaporation neutrons

EVPLD Nuclear explosive device

■ Neutron Flux and Energy

TOF: Time-of-flight

ASSOP: Associated particle

COINC: Coincidence

MANGB: Manganese bath

■ Detector

FISCH: Fission chamber

SCIN: Scintillation detector

IOCH: Ionization chamber

SIBAR: Silicon surface barrier

TRD: Track detector

SOLST: Solid-state detector

BF3: Boron Trifluoride

NAICR: Sodium-Iodide crystal

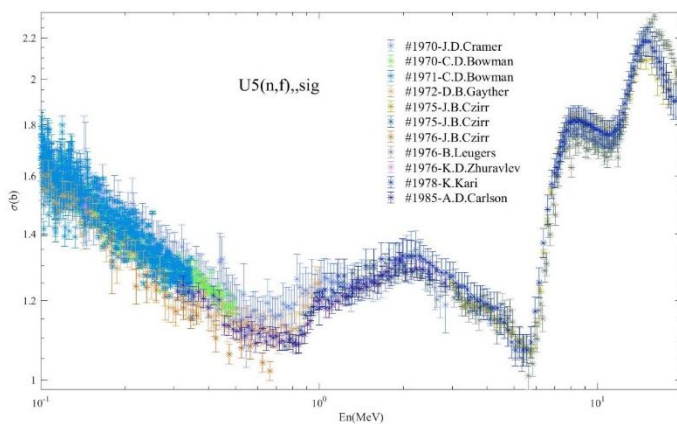
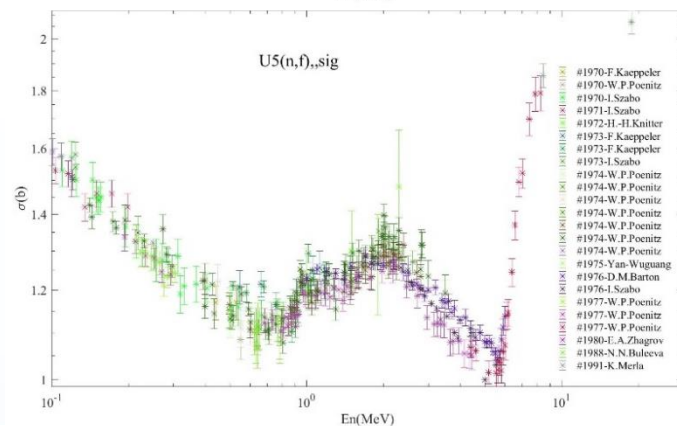
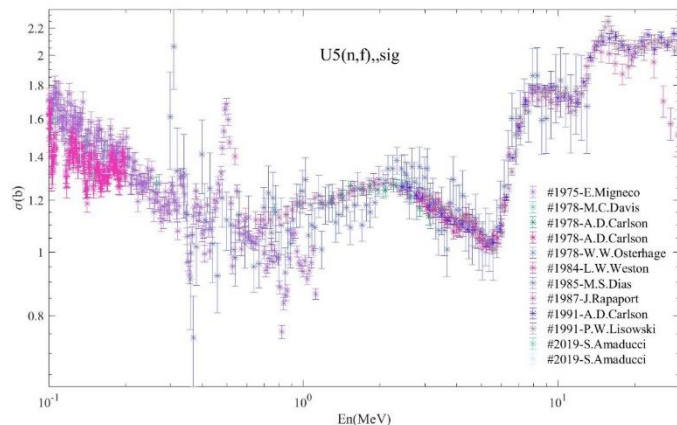
TELES: Counter telescope

LONGC: Long counter

PROPC: Proportional counter

GELI: Germanium-Lithium

2. Samples to deal with the exp. COV



Index	Year	Author	E_Min(MeV)	E_Max(MeV)	Institute	Facility	n-Source	Monitor	Method	Detector	
1.1	2019	S.Amaducci	1.780E-08	1.780E-01	2ITYLNS	SYNCY	SPALL	5U(n,f)/6Li(n,t) 5U(n,f),,INT 6Li(n,t)	TOF	SI	1)U5(n,f)/Li6(n,t), 2)U5(n,f)/B10(n,a)
1.2			1.780E-08	1.780E-01			SPALL				
1.3	1991	A.D.Carlson	2.470E+00	3.020E+01	1USANIS	LINAC	SPALL	H(n.el),,da.p	TOF	IOCH, TELES	
1.4	1991	P.W.Lisowski	3.030E+00	2.020E+02	1USALAS	LINAC	SPALL	H(n.el),,da.p	TOF	IOCH, TELES	1)U8/U5, 2)Pu9/U5
1.5	1987	J.Rapaport	5.390E-01	9.480E+02	1USALAS	LINAC	SPALL	-	-	IOCH, SCIN	
1.6	1985	M.S.Dias	1.070E+00	5.990E+00	1USANBS	LINAC	PHOTO	H(n.el)	TOF	FISCH, SCIN	
1.7	1984	L.W.Weston	9.730E-06	2.000E-01	1USAORL	LINAC	PHOTO	6Li(n,t),10B(n,a)	TOF	FISCH, GLASD	1)U5(n,f),,sig,,av, 2)Pu9(n,f),,sig, 3)Pu9(n,f),,sig,,av, 4)Pu0(n,f),,sig, 5)Pu0(n,f),,sig,,av
1.8	1978	M.C.Davis	1.400E-01	9.640E-01	1USAMH G		PHOTO	Efficiency of manganese bath determined relative to secondary national standard source NBS-II.	MANGB	TRD	1)Pu9(n,f), 2)Pu9/U5
1.9	1978	A.D.Carlson	1.171E+00	3.114E+00	1USANBS	LINAC	PHOTO	H(n.el),,da.p	TOF	FISCH, SILI	
1.10			2.788E+00	6.203E+00			PHOTO				
1.11	1978	W.W.Osterhage	3.000E-01	1.300E+01	2UK GLS	LINAC	PHOTO	5U(u,f)	TOF	SCIN	U8(n,f)
1.12	1975	E.Migneco	7.470E-04	1.120E+00	2ZZZGEL	LINAC	PHOTO	5U(n,f),,are	TOF	SCIN, IOCH, BF3	

information is incomplete.

Important information

Spallation

	2019-S.Amaducci	1991-A.D.Carlson	1991-P.W.Lisowski	1987-J.Rapaport
En(MeV)	1.78E-08~1.78E-01	2.47E+00~3.02E+01	3.03E+00~2.02E+02	5.39E-01~9.48E+02
Error		Total uncertainty	Total uncertainty	
	(S)Counting statistics	Statistics	Statistics	
	Statistical uncertainty of $^{235}\text{U}(n,f)/^{6}\text{Li}(n,t)$ or $^{235}\text{U}(n,f)/^{10}\text{B}(n,a)$ in the 7.8-11 eV cross section integral	Fission fragment counting efficiency	Efficiency	
	^{6}Li and ^{10}B efficiency correction	Transmission and scattering		
		Background		

	1985-M.S.Dias	1984-L.W.Weston	1978-M.C.Davis	1978-A.D.Carlson	1978-W.W.Osterhage	1975-E.Migneco
En(MeV)	1.07~5.99	9.73E-06~2.00E-01	1.40E-01~9.64E-01	1.171-6.203	3.00E-01~1.30E+01	7.47E-04~1.12E+00
Error	Total uncertainty		Total uncertainty	Total uncertainty		Total uncertainty
	Statistical uncertainty			Statistical uncertainty	Statistical uncertainty	Statistical uncertainty
	^{235}U mass+fission fragment		Net fission counts/source			

	1985-M.S.Dias	1984-L.W.Weston	1978-M.C.Davis	1978-A.D.Carlson	1978-W.W.Osterhage	1975-E.Migneco
En(MeV)	1.07~5.99	9.73E-06~2.00E-01	1.40E-01~9.64E-01	1.171-6.203	3.00E-01~1.30E+01	7.47E-04~1.12E+00
Correction		Constant-in-time background was small for flux monitors and the ^{235}U and ^{239}Pu fission plates but appreciable for ^{240}Pu , which was determined accurately.	fragment emission anisotropy		alpha-background	Background
			angular distribution normalization to lab.		photomultiplier-afterpulses	
			D(g,n) reaction in solution		incident flux monitor efficiency	
			parasitic absorption		detector geometry	
			leakage		events not recorded due to stopped fragments in the sample foil	
			scattering in Pt backings			
			scattering in other structures			
			foil impurities			
			energy spectrum.			

Photo-neutron

$^{235}\text{U}(n,f)$ XS error analysis

	1991-K.Merla	1988-T.Iwasaki	1988-Li Jingwen	1983-Li Jingwen	1982-O.A.Wasson	1982-M.Mahdavi	1978-M.Cance	1977-V.M.Adamov
En(MeV)	14.7	13.5~14.9	14.2	14.7	14.1	14.6	13.9	14.8
Error	Total	Total uncertainty	Total uncertainty		Total uncertainty	Total uncertainty	Total	Total error
	Statistics	(S)Fission counts including extrapolation and background	Statistics		(S)Ratio of net alpha-fission coincidence yield to alpha yield	(S)Fission track counting		
	Random coincidence	Sample assay	areal sample density		Neutrons scattered from beam	Fission fragment anisotropy	Loss of fissions	Error in the solid angle arising at the weighing of the sample in the small-solid-angle camera
	Correlated background	n-p cross section	Fission chamber detection efficiency		Fission fragment absorption	Angular dist. normalization to lab	Number of atoms per cm^2	Statistical error arising at the sample weighing in the small-solid-angle camera
	Fission fragment spectrum extrapolation	anisotropy	effects of incident neutron momentum		Fission spectrum loss	Total scattering perturbation	Neutron attenuation in target backing	Error of the isotope fraction determination at the sample weighing
	Fission fragment absorption		self-absorption of fragment in sample		Neutron beam shape and deposit uniformity	Total geometric error	Neutron attenuation in front face of fission chamber	Error of the fissionable isotope half-life influencing on the sample weighing
	Associated particle background		anisotropic of fission fragments		Standard ^{235}U reference mass	Total flux uncertainty	Fissions due to other isotopes	Statistical error arising by the number-of-fissions determination of isotope under
	Neutron scattering and foil thickness effect		nonuniformity of sample	uniformity of sample	Standard ^{235}U reference areal density	Deposit masses		Error due to the random coincidence number by the fission number determination
	Cone neutron outside angular extent of foils		neutron attenuation		Thermal neutron scattering from Pt backing of reference deposit			Error due to the neutron flux losses by the fission number determination
	Fission sample areal density		fission of other than ^{235}U		^{235}U areal density uncertainty			Error due to the extrapolation of the fission-fragments-spectrum to zero-energy arising by the number-of-fission determination
	Fission sample inhomogeneity		neutron cone spread		Estimated 1 standard deviation uncertainty			Error due to the correction on neutron absorption in the layer by the number-of-fissions determination
			(n,g) effect				(T)Quadrature sum of the following uncertainties:-	Error due to the fission of the admixture isotopes
9	D-T		neutron produced by secondary charged particle such as Ti(d,p), C(d,a)				Statistics including background subtraction;- Extrapolation to zero pulse height	Error due to the associated-particles number determination

	1991-K.Merla	1988-T.Iwasaki	1988-Li Jingwen	1983-Li Jingwen	1982-O.A.Wasson	1982-M.Mahdavi	1978-M.Cance
En(MeV)	14.7	13.5~14.9	14.2	14.7	14.1	14.6	13.9
Correction	random coincidence	coincidence losses of recoil protons	fission detector efficiency		variation in uranium deposit thickness	fragment emission anisotropy	background
	correlated background	fission counts extrapolation to zero pulse height	neutron flux attenuation in samples		uncertainty in neutron beam profile	angular distribution normalization to Lab.	ionization chamber efficiency
	fission fragment spectrum extrapolation	self-absorption of sample	fission events due to other isotopes in the sample		fission events below discrimination level	scattering in Pt backings	extrapolation to zero pulse height
	fission fragment absorption	interaction of recoiled protons			fission fragments absorbed in deposits	scattering in other structure materials	loss of fissions
	associated particle background	neutrons scattered by target material			fission by other than ^{235}U	foil isotopic composition	neutron attenuation in target backing
	neutron scattering and foil thickness effect	neutrons scattered by chamber material			neutron attenuation	dsigma/dEn adjustment for Fe(n,p) cross section	neutron attenuation in front face of fission chamber
		attenuation of source neutrons			accidental coincident background contributions	detector efficiency	fissions due to other isotopes

Neutron
source: D-T

- 1、 Fission detector efficiency
- 2、 Neutron attenuation
- 3、 Isotopes in the sample
- 4、 Fission fragment absorption
- 5、 Fission fragment spectrum extrapolation

ENTRY 41112001 20150627 20150817 20150810 4169
 SUBENT 41112001 20150627 20150817 20150810 4169
 BIB 12 43
 TITLE Correction of the results of absolute measurements of U-235 fission cross-section by neutrons with energy 1.9 and 2.4 MeV
 AUTHOR (V.A.Kalinin, V.N.Kuz'min, L.M.Solin, B.I.Shpakov, K.Merla)
 INSTITUTE (4RUSRI) Kalinin, Kuz'min, Solin, Shpakov (2GERDRE) Merla, former 3DDRTUD
 # (4RUSRI) t.Petersburg, Russia
 # (2GERDRE), Germany
 REFERENCE (J,AE,71,(2),181,1991) Main Reference.
 (J,SJA,71,700,1991) Engl.translation of AE,71,(2),181
 (J,AE,64,(3),194,1988) Preliminary results.
 (J,SJA,64,239,1988) Engl.translation of AE,64,(3),194
 # (J,AE,71,(2),181,1991) Jour: Atomnaya Energiya, Vol.71, Issue.2, p.181 (1991),
 #+ #Title=Correction of the results of absolute measurements of U-235 fission cr
 #+ #Authors=V.A.Kalinin, V.N.Kuz'min, L.M.Solin, B.I.Shpakov, K.Merla
 # (J,SJA,71,700,1991) Jour: Soviet Atomic Energy, Vol.71, p.700 (1991), SAU
 #+ #URL=http://dx.doi.org/10.1007/BF01121671
 #+ #DOI=10.1007/BF01121671
 # (J,AE,64,(3),194,1988) Jour: Atomnaya Energiya, Vol.64, Issue.3, p.194 (1988),
 #+ #NSR=1988KA34
 #+ #Title=Absolute Measurements of the Cross Section for Fission of ²³⁵U by 1.
 #+ #Authors=V.A.Kalinin, S.S.Kovalenko, V.N.Kuzmin, Yu.A.Nemilov, L.M.Solin, V
 # (J,SJA,64,239,1988) Jour: Soviet Atomic Energy, Vol.64, p.239 (1988), SAU
 #+ #URL=http://dx.doi.org/10.1007/BF01123132
 #+ #DOI=10.1007/BF01123132

MONITOR Absolute counting
 METHOD (ASSOP) Method of He-3 (~ 3 MeV). Separation of CP t
 # (ASSOP) Associated particles
 INC-SOURCE (D-T) Deuteron energy 2.8 MeV and 3.4 MeV .
 # (D-T) 3H(d,n)

Information wrong !

This paper presents the results of absolute TCAP measurements of ²³⁵U fission cross sections at neutron energies E_n of 1.9 and 2.4 MeV. These energy points lie in the range 1-5 MeV, where the existing experimental data have the greatest discrepancy (10% or more) and are in two groups: with higher values of σ_j and a convex form of the dependence σ_j(E_n) and with lower values of σ_j and a concave form (i.e., the values of the cross sections cannot be brought into agreement by renormalization). The ENDF/BV library estimate that is accepted at the present time is midway between the two. The results of previous TCAP measurements for 2.6-MeV neutrons belong to the group with lower values of σ_j.

In this study, the D(d, n)³He reaction at a deuteron-beam energy E_d = 3 MeV was used to obtain neutrons. The reaction kinematics in this case is such that neutrons of the required energy are emitted into the rear hemisphere and the associated helium ions are emitted

Translated from Atomnaya Energiya, Vol. 64, No. 3, pp. 194-198, March, 1988. Original article submitted January 19, 1987.

ENTRY 10711001 20170628 20171110 20171103 1432
 SUBENT 10711001 20170628 20171110 20171103 1432
 BIB 9 20
 TITLE Additional measurements of the ²³⁵U(n,f) cross section in the 0.2 to 8.2-MeV range
 AUTHOR (W.P.Poenitz)
 INSTITUTE (1USAANL)
 # (1USAANL) Argonne National Laboratory, Argonne, IL,
 REFERENCE (J,NSE,64,894,1977)
 # (J,NSE,64,894,1977) Jour: Nuclear Science and Engineering, V
 #+ #URL=http://dx.doi.org/10.13182/NSE77-A14509
 #+ #NSR=1977PO19 #DOI=10.13182/NSE77-A14509
 #+ #Title=Additional Measurements of the ²³⁵U(n,f) Cross S
 #+ #Authors=W.P.Poenitz
 FACILITY (VDG,1USAANL)
 # (VDG Van de Graaff
 #,1USAANL) Argonne National Laboratory, Argonne, IL,
 MONITOR Flux measured using black neutron detector
 DETECTOR (SCIN) Black neutron detectors:
 - small (~15 cm height, ~6.4 cm radius) below 1.5 MeV
 - large (~40 cm height, ~20 cm radius) above 0.7 MeV
 # (SCIN) Scintillation detector

The Black Neutron Detector (BND)
 The fast-neutron flux is often determined with proton-recoil counters,⁵²⁻⁵⁴ utilizing the well-known total cross section of hydrogen. The flux measurements with these detectors depends on the knowledge of the scattering cross section of hydrogen and its angular distribution, the active volume of the counter, the amount of hydrogenous material, the extrapolation to zero pulse height, etc., depending on the type of detector used. For the present measurements, a total energy conversion detector was designed for the measurement of the absolute neutron flux which was based on the detection of proton recoils but which did not depend in first order on the above-cited effects and cross sections. The detector is a medium-sized scintillator of cylindrical shape with a neutron detector information need to be explicit.

EN KEV	EN-ERR KEV	DATA NO-DIM	ERR-S PER-CENT	ERR-T PER-CENT	20880002	18
1.2456E+01	6.2000E-02	6.5500E-01	5.0550E+01	5.1060E+01		
1.7274E+01	1.0100E-01	4.9200E-01	2.4930E+01	2.5950E+01		
2.2098E+01	1.4600E-01	3.1700E-01	1.3590E+01	1.5380E+01		
2.2489E+01	1.5000E-01	2.9900E-01	1.4180E+01	1.5900E+01		
2.6029E+01	1.8700E-01	4.2100E-01	9.8700E+00	1.2220E+01		
2.8684E+01	2.1600E-01	3.2700E-01	8.6500E+00	1.1250E+01		
3.1112E+01	2.4400E-01	3.5800E-01	7.4400E+00	1.0350E+01		
3.1767E+01	2.5100E-01	3.3700E-01	7.3300E+00	1.0270E+01		
3.3862E+01	2.7700E-01	3.8500E-01	6.8600E+00	9.9400E+00		
3.6170E+01	3.0500E-01	3.6400E-01	6.7600E+00	9.8800E+00		
3.8723E+01	3.3800E-01	3.7900E-01	6.2800E+00	9.5500E+00		
4.1556E+01	3.7600E-01	3.9300E-01	5.8100E+00	9.2500E+00		

H. BEER AND F. KÄPPELER

Data corrected without indicating the

$$E_n \pm \Delta E_n \text{ (keV)}$$

12,5

STATUS .Data taken from priv. Comm..
 HISTORY (19790330C) Wwo.
 (19790518E)
 (19800303A) Main reference for the experimental setup added.
 (19800331E)
 (19810402A) Err-analys corrected.
 (19810429E)
 (20070314A) Date is corrected

3. China's contribution to EXFOR compilation

Measurements carried out in China



Main Facility of China:

- (REAC,3CPRAEP) Heavy-water reactor
- (CYCLO,3CPRAEP) Cyclotron
- (REAC,3CPRAEP) 35 kW Miniature Neutron Source Reactor
- (VDGT,3CPRAEP) The HI-13 Tandem Accelerator
- (VDG,3CPRAEP) 2.5 MV Van de Graaff
- (CCW,3CPRAEP) 600 kV Cockroft-Walton accelerator
- (VDG,3CPRBJG) 4.5 MV Van de Graaff accelerator
- (NGEN,3CPRLNZ) ZF-300-II intense neutron generator
- (CYCLO,3CPRIMP) Heavy Ion Research Facility
- (FRS,3CPRIMP) Radioactive Ion Beam Line
- (CCW,3CPRIMP) 320 kV,600kV Cockroft-Walton accelerator
- (NGEN,3CPRIMP) T-600 neutron generator
- (NGEN,3CPRNPC) The K-400 Intense Neutron Generator
- (NGEN,3CPRNPC) Pd-300 Neutron Generator
- (CYCLO,3CPRNRS) 1.2 Meter Cyclotron
- (LINAC,3CPRIHP) The Back-streaming white neutron source (Back-n WNS) at China Spallation Neutron Source (CSNS)

3. China's contribution to EXFOR compilation

Scanning Chinese journals or conference proceedings

1) Chinese Physics C (**CPH/C**, English edition, 2008)

Former Title: High Energy Physics and Nuclear Physics (PHE, Chinese edition, 1977)

2) Atomic Energy Science and Technology (**CST**, Chinese edition, 1959)

3) Journal of Nuclear and Radiochemistry (**HFH**, Chinese edition, 1979)

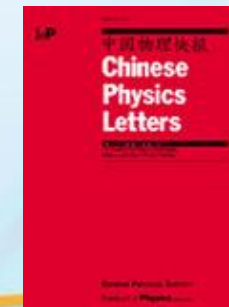
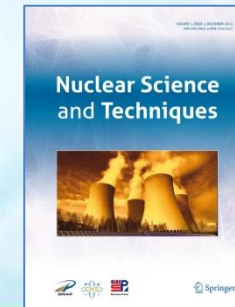
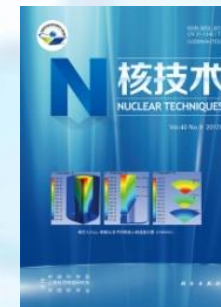
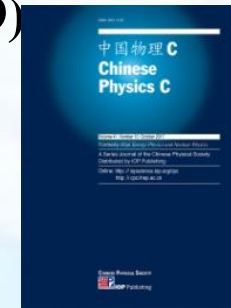
4) Nuclear Techniques (**NTC**, Chinese edition, 1978)

5) Nuclear Science and Techniques (**CNST**, English edition, 1989)

6) Chinese Physics Letters (**CPL**, English edition, 1984)

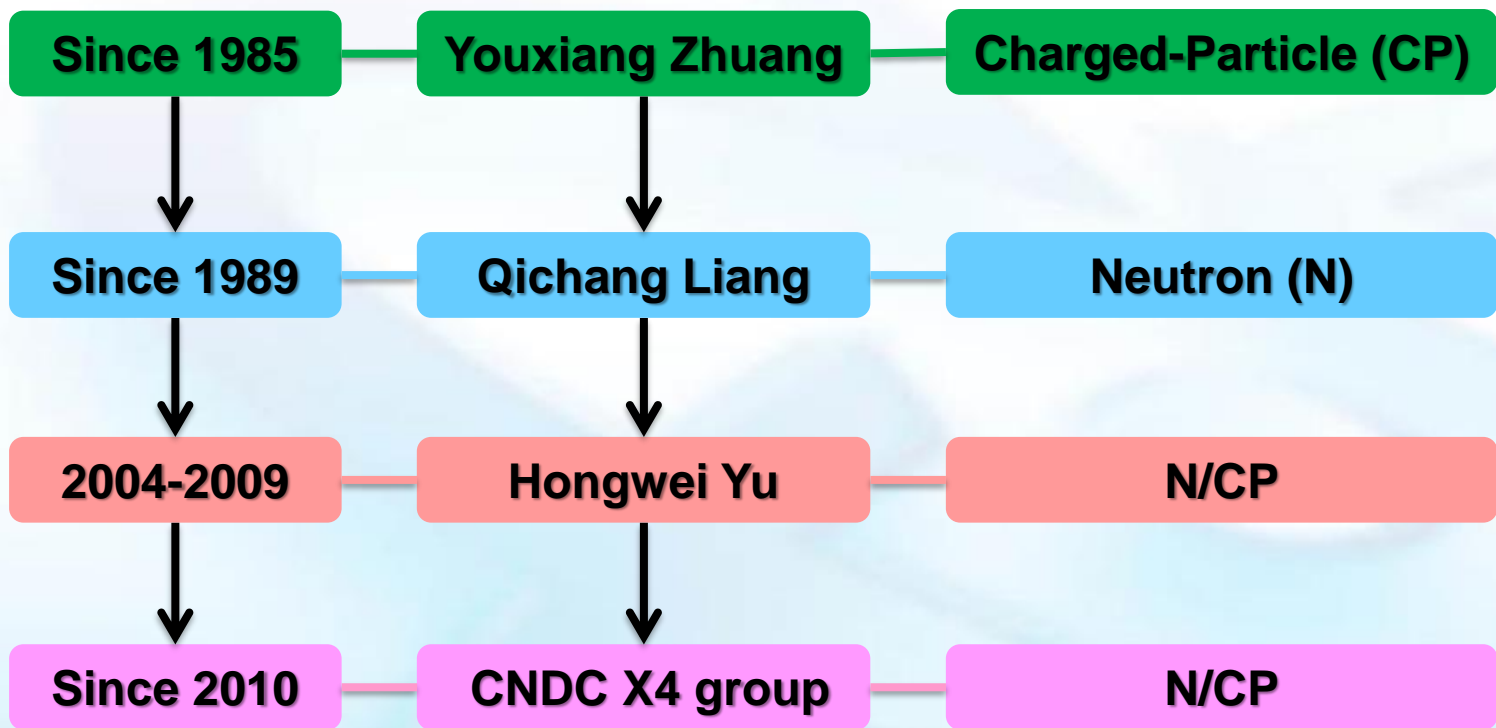
7) Acta Physica Sinica (**ASI**, English edition, 1933)

8) Conference Proceedings, Journal of University, etc.



EXFOR Compilation Status

- Compilation history and CNDC X4 Group



ERR-ANALYS

- For Some EXFOR Entries, there are detailed source of uncertainty and relative or absolute values in ERR-ANALYS. For example, the EXFOR entry 32766 for ‘Measurement of neutron-induced fission cross section of ^{238}U at 14 MeV’, and 10314 for ‘Absolute measurements of ^{235}U and ^{239}Pu fission cross-sections with photoneutron sources’.
- For Some EXFOR Entries, there are not detailed source of uncertainty, or no relative or absolute values in ERR-ANALYS. There are only “No information”, or only Total uncertainty, or only Statistical uncertainty.



ERR-ANALYS

ENTRY	32766	20181121	20190225	20190223	318732766000	1
SUBENT	32766001	20181121	20190225	20190223	318732766001	1
BIB	11	31			32766001	2
TITLE	Measurement of neutron-induced fission cross section of 238U at 14 MeV				32766001	3
AUTHOR	(Hu Zhongkang, Qi Bujia, Li Anli, Zhang Baisheng, Zhou Huiming, Dong Mingli)				32766001	4
INSTITUTE	(3CPRAEP)				32766001	5
REFERENCE	(J, CST, 14, 201, 1980)				32766001	6
FACILITY	(CCW, 3CPRAEP)				32766001	7
INC-SOURCE	(D-T) 0.7 mg/cm ² thick TiT for 150 & 200 keV deuterons at 45 & 90 deg. 0.24 mg/cm ² thick TiT for 220 keV deuterons at 0 deg.				32766001	8
SAMPLE	Natural U foils (898 ug and 899 ug) electrodeposited on Pt foil (0.1 mm thick). The masses of U foils were determined by weighing, coulometry, and alpha spectrometry.				32766001	9
DETECTOR	(FISCH) Ar (95%)+ CO ₂ (5%) at 500 mmHg (SOLST) Detection of associated alpha particles at 90 deg.				32766001	10
METHOD	(ASSOP) Detection of alpha from T(d,n)4He				32766001	11
CORRECTION	Corrected for				32766001	12
	- Anisotropic emission of associated alpha in lab system				32766001	13
	- Subthreshold fission number (2%)				32766001	14
	- Materials between TiT target and U sample (1.6%)				32766001	15
	- Central position of U foil				32766001	16
	- The sum (3%) of D(d,n)3He neutron contribution (<1%) and scattered neutrons				32766001	17
	- Self-absorption of fission fragment				32766001	18
	- Alpha originated from 3He(d,p)4He with 3He decayed from T (0.2% at 14.10 MeV, 0.4% at 14.60 MeV, 1.8% at 15.04 MeV)				32766001	19
HISTORY	(20181121C) Wang Jimin (CNNC) + Otsuka Naohiko (NDS)				32766001	20
ENDBIB	31				32766001	21
NOCOMMON	0	0			32766001	22
ENDSUBENT	34				3276600199999	23

SUBENT	32766002	20181121	20190225	20190223	318732766002	1
BIB	3	17			32766002	2
REACTION	(92-U-238(N,F),,SIG)				32766002	3
ERR-ANALYS	(ERR-T) Total uncertainty				32766002	4
	(ERR-1) Solid angle of alpha detector (1.1%)				32766002	5
	(ERR-2) Background of alpha detection (0.3%)				32766002	6
	(ERR-3) Direction of alpha emission (0.5%)				32766002	7
	(ERR-4) Effect of material between TiT target and U sample (0.8%)				32766002	8
	(ERR-5) U sample mass (0.8%)				32766002	9
	(ERR-6) Distance between TiT target and U sample (1.2%)				32766002	10
	(ERR-7) Extrapolation of fission fragment spectrum (0.7%)				32766002	11
	(ERR-8) Background subtraction (0.6%)				32766002	12
	(ERR-9) Statistics deviation (0.5%)				32766002	13
	(ERR-9) 3He(d,p)4He contribution (1% at 15.04 MeV, <0.4% at 14.10 and 14.60 MeV)				32766002	14
STATUS	(TABLE) Table 2 of Atom. Energy Sci. Tech. 14(1980)201				32766002	15
ENDBIB	17				32766002	16

Dear Wang-san,

I am going to adopt it this EXFOR entry in my least-squares analysis for JENDL update!

This entry summaries uncertainty information very nicely!

Best regards,

Naohiko Otsuka

4. Some considerations to future EXFOR

ERR-ANALYS

- Very detailed information for **U**ncorrelated, **F**ully correlated, **P**artially correlated and **C**orrelated.

SUBENT	10314002	20171207	20180625	20180619	143910314002	1
BIB	5	24			10314002	2
REACTION	(92-U-235(N,F),,SIG)				10314002	3
SAMPLE	U308 deposits of 8.16 and 6.85 mg on Pt (+/-30 ug). The mass was determined by microbalance weighings and confirmed by thermal neutron fission and alpha counting.				10314002	4
					10314002	5
					10314002	6
					10314002	7
ERR-ANALYS	(ERR-T) Total uncertainty				0314002	8
	(ERR-1,,U) Net fission counts/source neutron				0314002	9
	(ERR-2,,U) Manganese bath comparison of sources				0314002	10
	(ERR-3,,C) Fragment emission anisotropy				0314002	11
	(ERR-4,,C) Angular distribution normalization to lab.				0314002	12
	(ERR-5,,C) Half-life extrapolation				0314002	13
	(ERR-6,,F) NBS-II reference source	(0.50%)			0314002	14
	(ERR-7,,F) Fissile foil masses	(0.50%)			0314002	15
	(ERR-8,,C) Scattering in Pt backings				0314002	16
	(ERR-9,,C) Scattering in other structures				0314002	17
	(ERR-10,,U) Compensated beam geometry	(0.57%)			0314002	18
	(ERR-11,,C) Energy spectrum	(0.30%)			0314002	19
	(ERR-12,,U) Aperture diameter	(0.13%)			0314002	20
	(ERR-13,,U) Deposit-aperture distance	(0.14%)			0314002	21
STATUS	(TABLE) Tables 6 and 8 of Ann.Nucl.Energy 5(1978)569				0314002	22
HISTORY	(19790315A) Data updated.				10314002	23
	(20170724A) On. Partial uncertainties added.				10314002	24
	DATA-ERR -> ERR-T and %-uncertainties adopted for it.				10314002	25
	(20171207A) On. ERR-12 and ERR-13 added.				10314002	26

- No information or incomplete.

SUBENT	22304007	20180724	20190424	20190423	227522304007	1
BIB	5	5			22304007	2
REACTION	(92-U-238(N,F),,SIG)				22304007	3
FACILITY	(NGEN,2GERDRE)				22304007	4
ERR-ANALYS	(DATA-ERR) No information on the source of uncertainty				22304007	5
STATUS	(TABLE) Text (p.513) of 1991 Juelich Conf. p.510				22304007	6
HISTORY	(20180724A) On. Re-compiled.				22304007	7
ENDBIB	5				22304007	8
NOCOMMON	0	0			22304007	9

ERR-ANALYS	(ERR-T) The total error given includes all the errors, except those due to the uncertainties in the isotopic composition and chemical purity of the boron foils.	21520001	66
		21520001	67
		21520001	68
		21520001	69
STATUS	(NDD) Data taken from NEUDADA.	21520001	70
HISTORY	(19800422T) G.C.	21520001	71
	(19800429E)	21520001	72
	(20070926A) Date is corrected. DATA-ERR To ERR=T , units in SAN 004 changed - SM	21520001	73
	(20071001U) Last corrections have been done.	21520001	74
		21520001	75

CORRECTION	DATA ARE CORRECTED FOR DEAD TIME, CONSTANT AND TIME-DEPENDENT BACKGROUND	12905001	24
		12905001	25
COMMENT	DATA SHOULD BE USED WITH CAUTION ABOVE 20 EV.	12905001	26
ERR-ANALYS	(ERR-S) STATISTICAL ERROR	12905001	27
STATUS	DATA RECEIVED IN PRIVATE COMMUNICATION 86/7/31 FROM R. GWIN	12905001	28
	(APRVD) R.GWIN, 86/9/5.	12905001	29
HISTORY	(860821C)	12905001	30
	(870605A) MONITOR CORRECTED.	12905001	31
		12905001	32

- It will be better that the detailed information on the source of uncertainty was included in ERR-ANALYS, from articles or authors.

4. Some considerations to future EXFOR

- Real-time feedback platform for EXFOR is expected to build, which provides users from various countries and regions a more convenient channel to record the insufficient, and the maintainer can know them quickly;
- For same source of the incident particle beam, or same experimental technique, etc. standardization of the main sources of uncertainty was handled in EXFOR will makes processing more effective for experimental covariance;



4. Some considerations to future EXFOR

- The correlated coefficient of each experimental sources (beam, sample, detector...) are looking forward to presenting in entries by the experimenter, which is important to create a more reliable non-model dependent COV;
- The content in COMMENTS requires more complete description, especially why compiler modified the data and if the data in the subentries are better recommendation for application?

Thank you for your attention!

