



# R&D in Nuclear Data for Reactor Physics Applications in CNL (CNL = Canadian Nuclear Laboratories)

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Canadian Nuclear  
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**Improvement of TSL (Thermal Scattering *Laws*, or  $S(\alpha, \beta)$  data)**  
**New Evaluations for  $D_2O$ ,  $H_2O$ ,  $UO_2$ ,  $ThO_2$ , ...**  
**New Models, Measurements, and Testing (Benchmarking)**

**D. Roubtsov, J.C. Chow, G. Bentoumi, Gang Li**

Retired: K. Kozier, D. Altiparmakov, B. Wilkin, ...

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# From AECL to CNL

AECL is divided into:

- CANDU Inc. ( ← Toronto Reactor division of old AECL )
- **CNL** ( ← Chalk River Labs of old AECL )
- AECL (or R-AECL)



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CNL will be managed following Canadian GoCo model  
(GoCo = Government-owned, Contractor-operated)

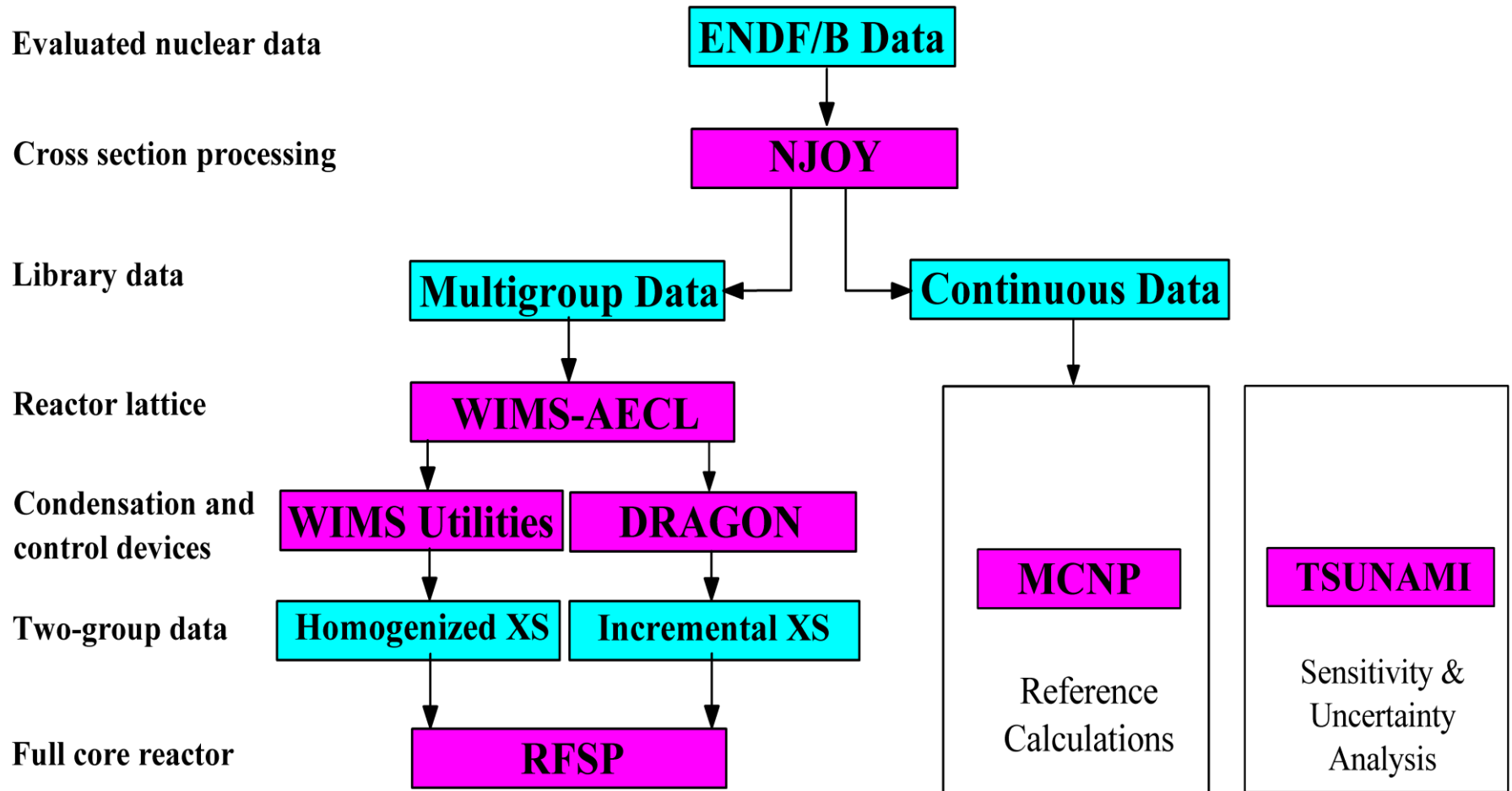
<http://www.cnl.ca/en/home/about/Restructuring/default.aspx>

*"Restructuring of AECL's Nuclear Laboratories"*

[http://www.cnl.ca/en/home/about/Restructuring/NRU\\_decision.aspx](http://www.cnl.ca/en/home/about/Restructuring/NRU_decision.aspx)

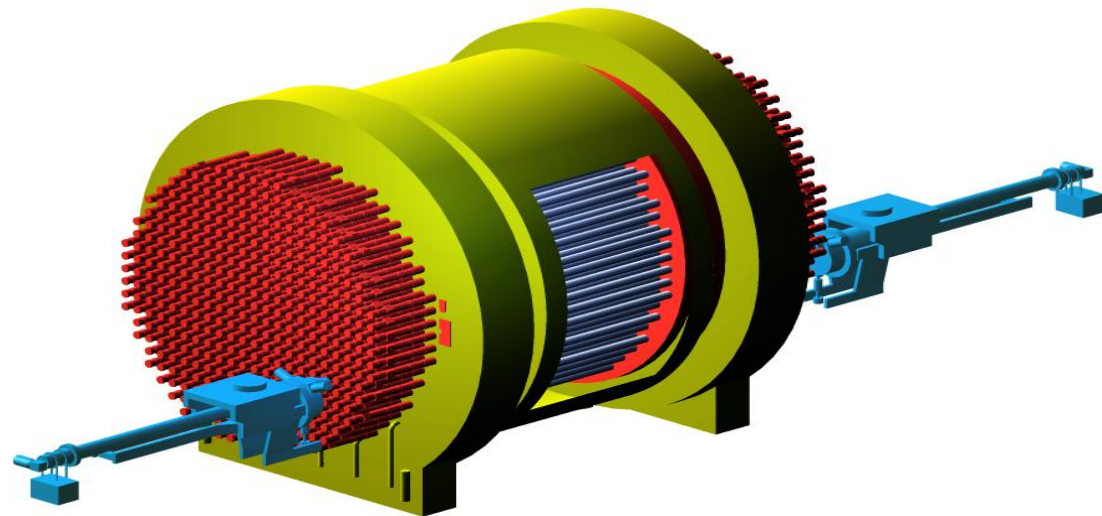
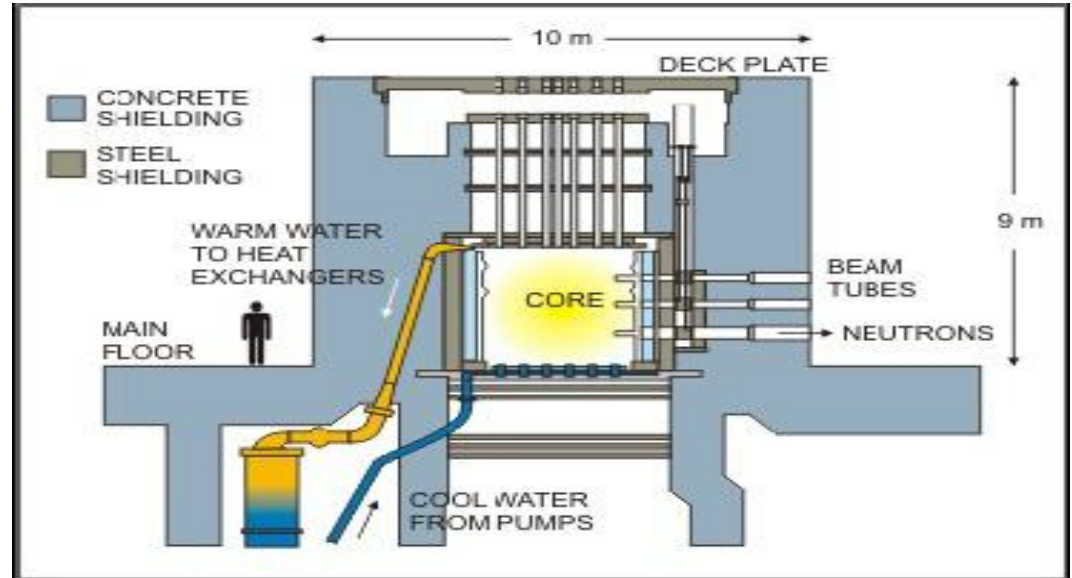
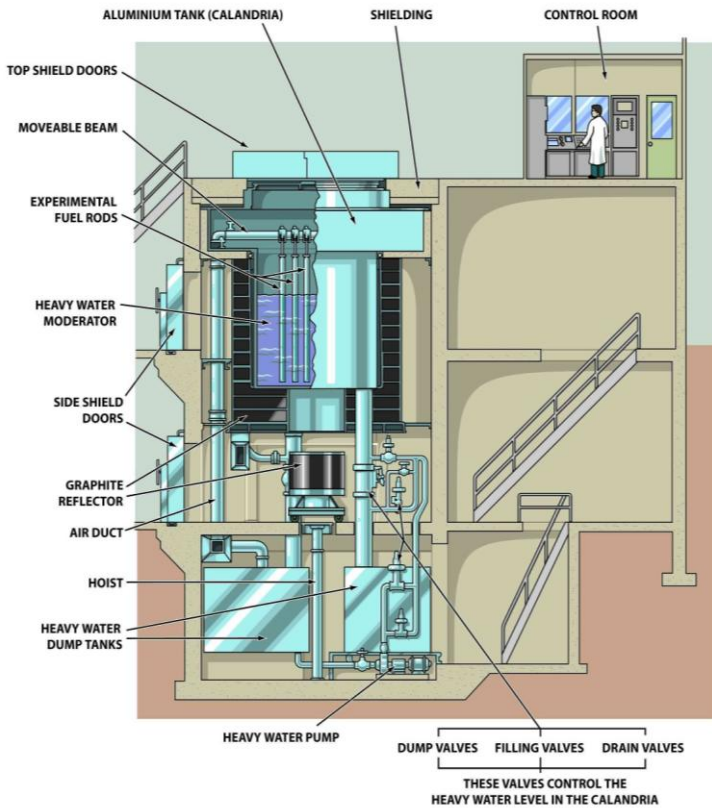
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# CNL (AECL) Reactor Physics Computational Scheme

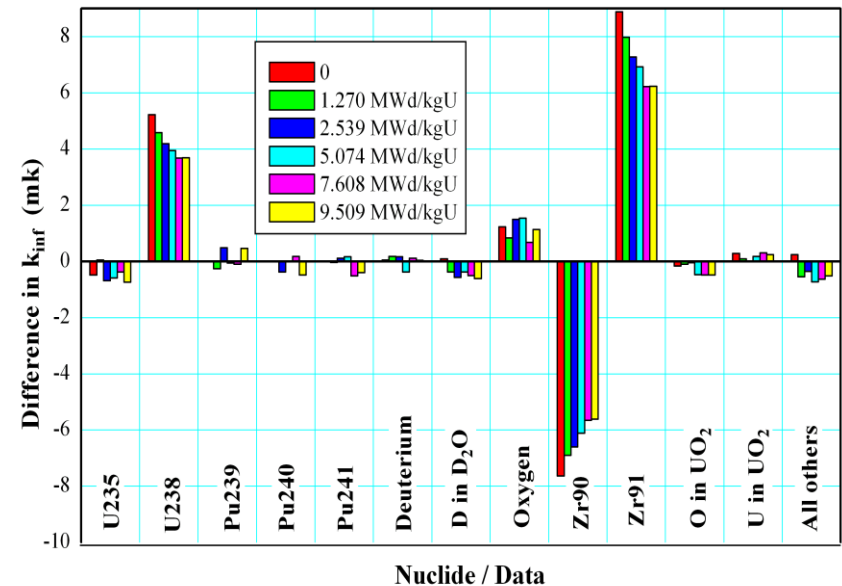
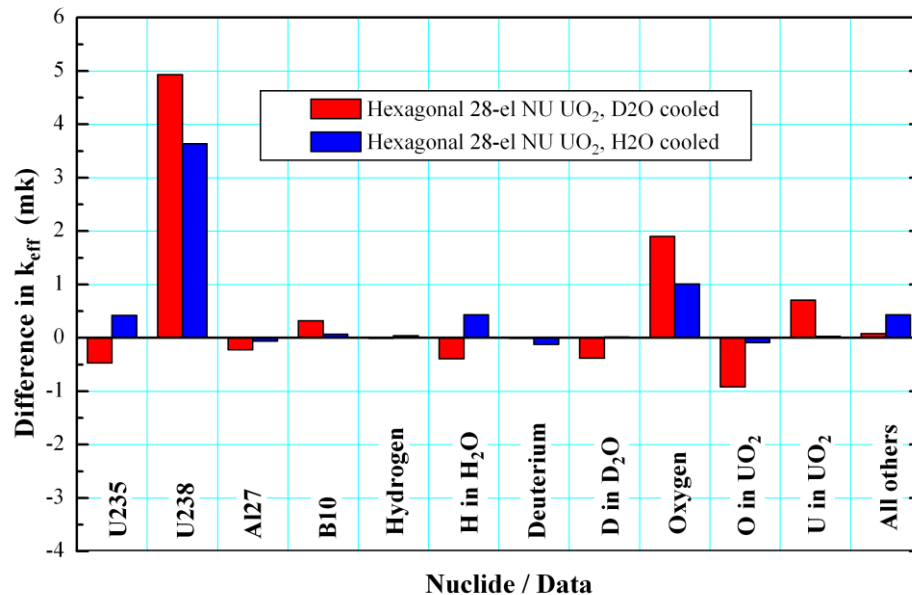




# CNL libraries applied for ZED-2, NRU (Chalk River) and CANDU-type PHWRs



# Importance of accuracy of ND and TSL: case study (e.g., in n spectra with optimal moderation by D<sub>2</sub>O )



Impact of particular ENDF/B-VII.0 data on ZED-2 reactor simulation (left);

1 mk = 100 pcm

Effect of particular ENDF/B-VII.0 data on MCNP calculation of 37-element CANDU lattice cell

D. Altiparmakov, PHYSOR-2010

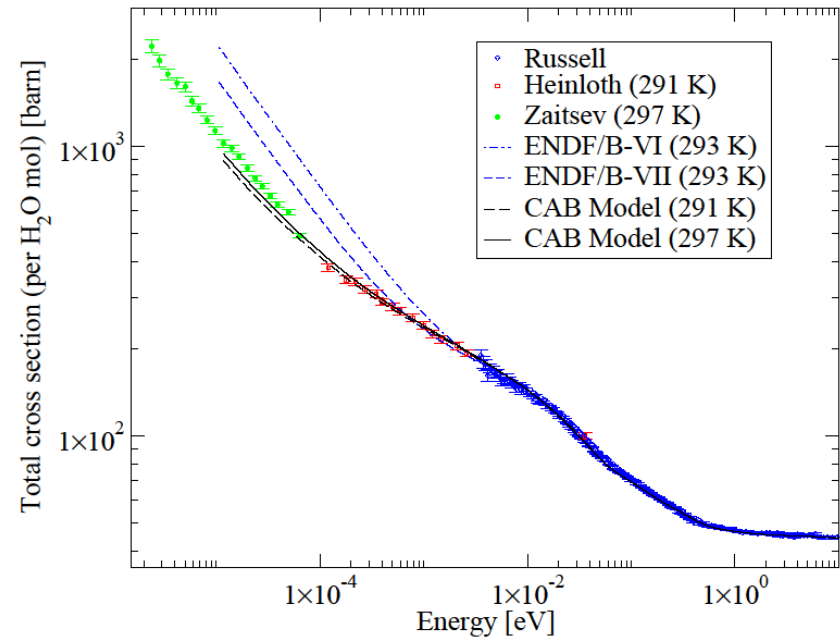
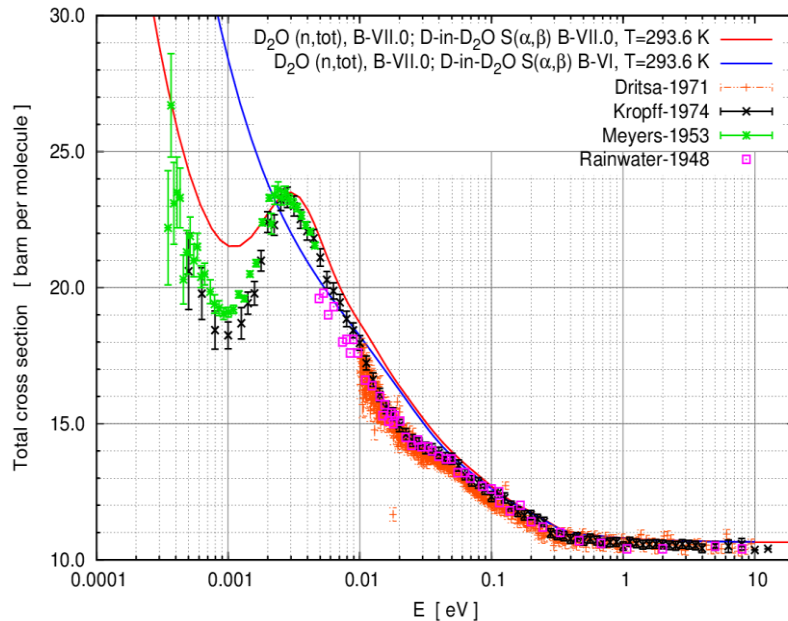


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# TSL for heavy and light water:

interaction of thermal neutrons with liquid  $D_2O$  /  $H_2O$  at different  $T$  and  $p$



- **Discrepancy for heavy water in modern evaluated ND libraries:** in integral values, at neutron energy  $E$  near  $E_{th} = 0.0253$  eV, for liquid  $D_2O$  at **room temp.  $T$** , we have  $(Exp. - Calc.) / Calc. \approx -8.4\%$  (using ENDF/B-VII.0  $S(\alpha,\beta)$  for **D-in- $D_2O$**  and Free Gas for  $^{16}O$ )

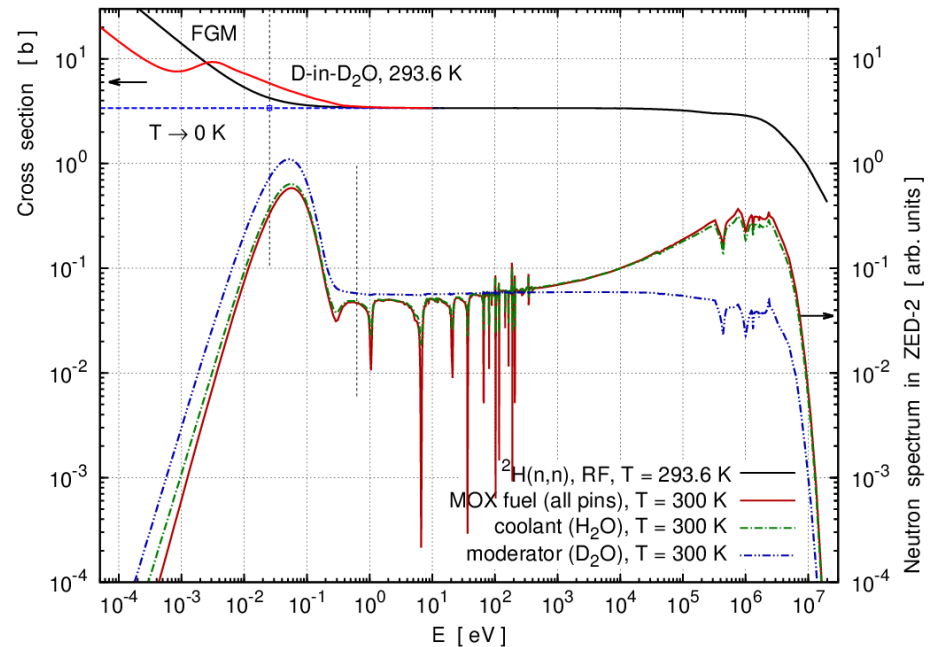
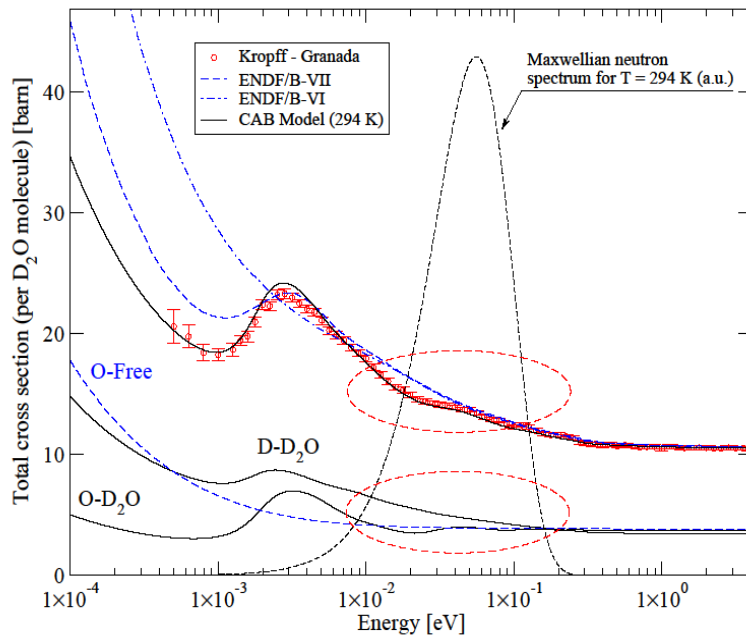
- What to do?

TSL evaluation is a **model** of thermal neutron interaction with a condensed medium (liquid, solid) based on QM and Stat. Phys.; converted into dimensionless tables  $S(\alpha_i, \beta_j; T_n)$  following **ENDF** format. So, improve modeling, improve numerics, ...

- How to *improve* modeling?

# TSL for heavy water (FY 2014/2015)

interaction of thermal neutrons with liquid  $D_2O$  at room  $T$



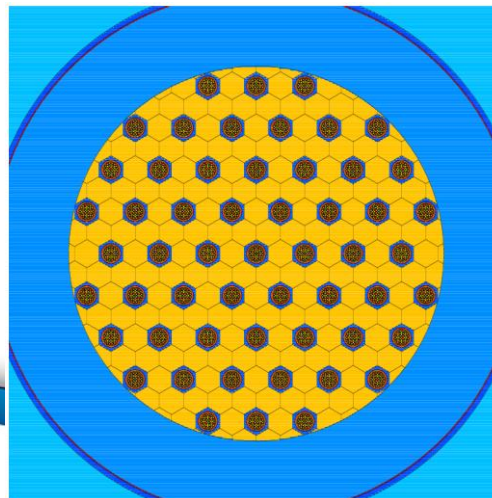
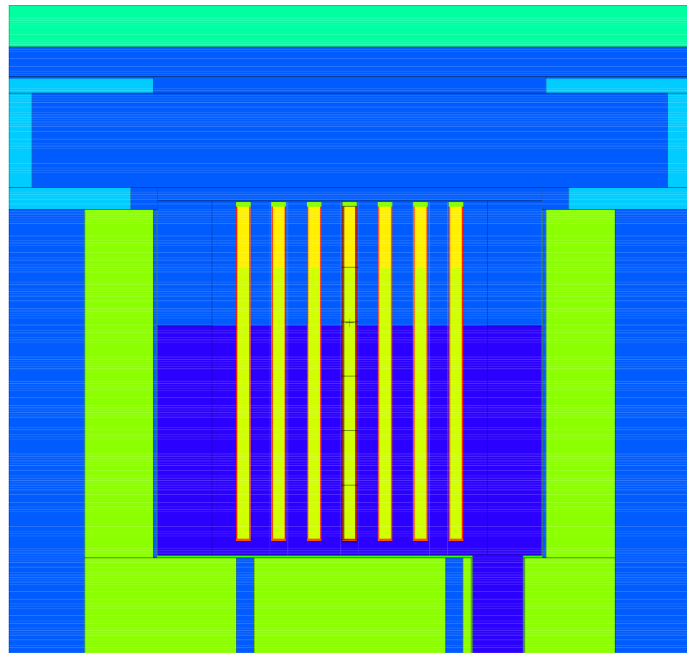
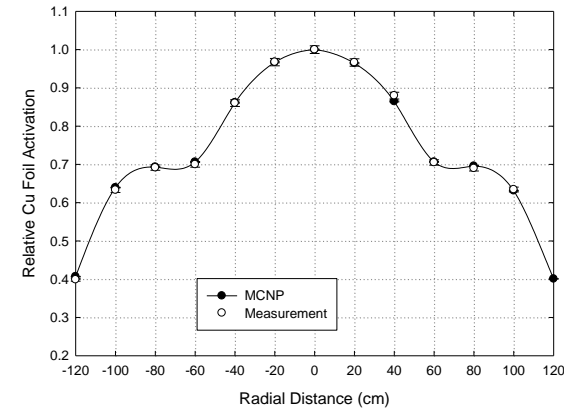
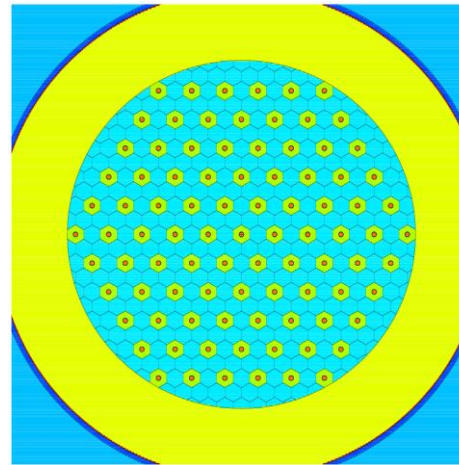
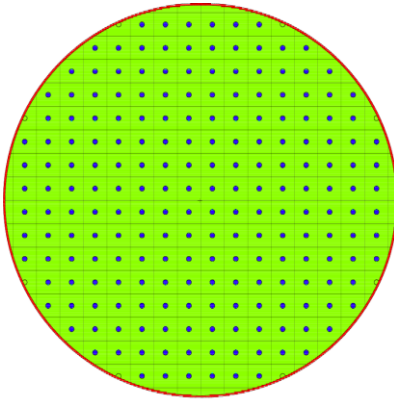
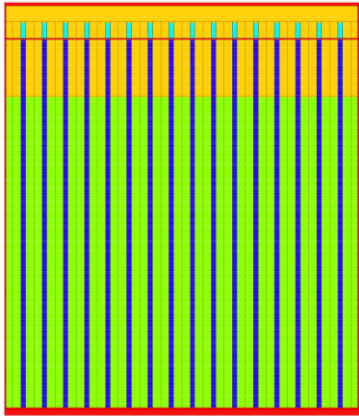
- Developing **new** TSL evaluations for water (**CNL**, Canada and **CAB**, Argentina): based on combining **Mol. Dynamics (MD)** simulations (GROMACS) and *experimental data*
- The resulting new **models** are formulated as LEAPR input files for **LEAPR** module of **NJOY99 (up396 with additional patches)**  
NJOY = nuclear data post-processing code, LANL, USA





# Benchmarking

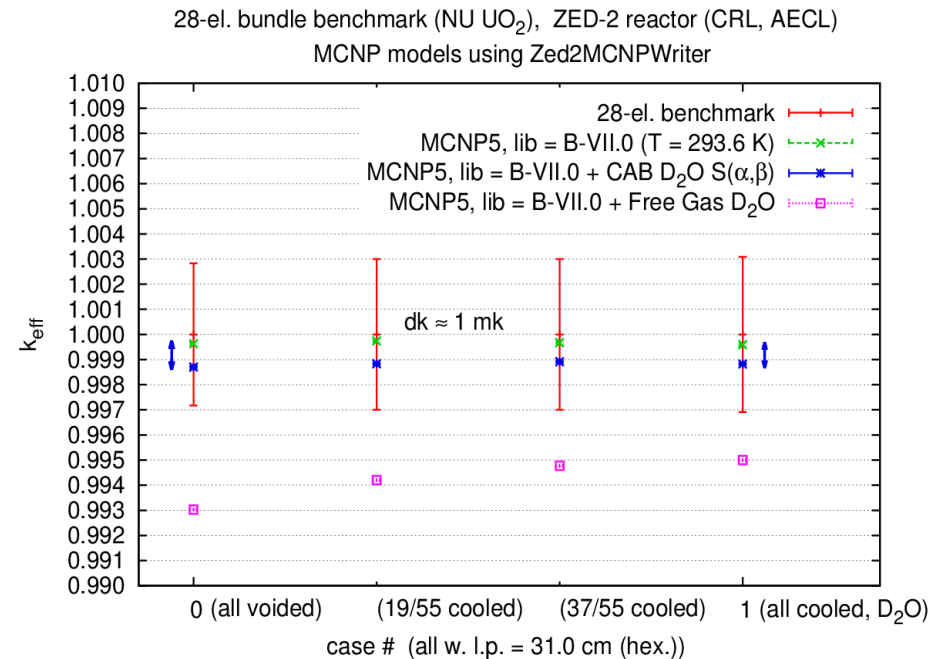
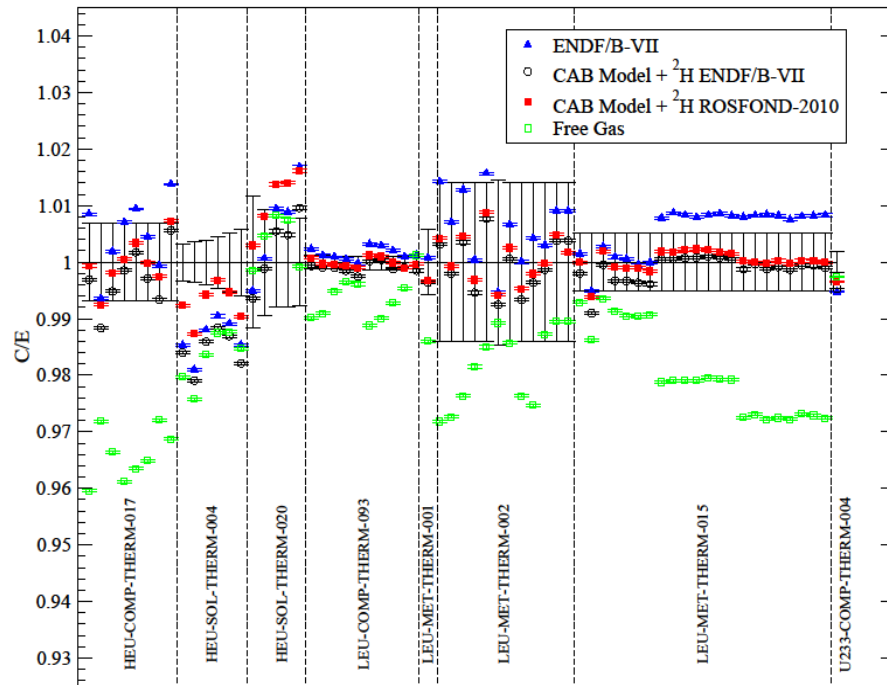
Integral experiments: k-eff, also reaction rates, spectral indices



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# TSL for heavy water (FY 2014/2015)

interaction of thermal neutrons with liquid D<sub>2</sub>O (at room T)



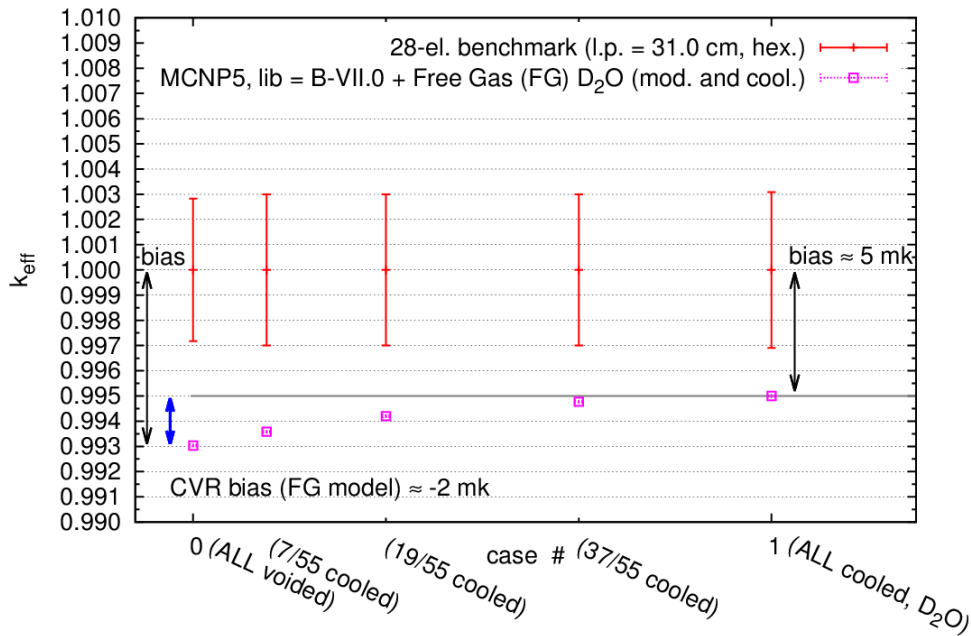
- **Testing:** using Crit. Safety **Benchmarks** with D<sub>2</sub>O, overall, it is an improvement in  $k_{eff}$  **C/E** if ENDF/**B-VII.0** D<sub>2</sub>O S( $\alpha,\beta$ ) → **new** D<sub>2</sub>O S( $\alpha,\beta$ ) (we call it **CAB** models)
- We expect that  $k_{eff}$  (**CAB** D<sub>2</sub>O) <  $k_{eff}$  (**B-VII.0** D<sub>2</sub>O), but what is the difference  $dk = k_{eff}$  (B-VII.0 D<sub>2</sub>O) –  $k_{eff}$  (CAB D<sub>2</sub>O) = ? ; **cases with  $dk > \Delta k_{Bench}$  of special interests**

**answer** for ZED-2 reactor:  **$dk \approx 100$  pcm** ,  **$dk < \Delta k_{Bench}$**  , **ZED2-HWR-EXP-001** benchmark (28-element NU UO<sub>2</sub> bundles, at room T, evaluator = J. Atfield, CNL)

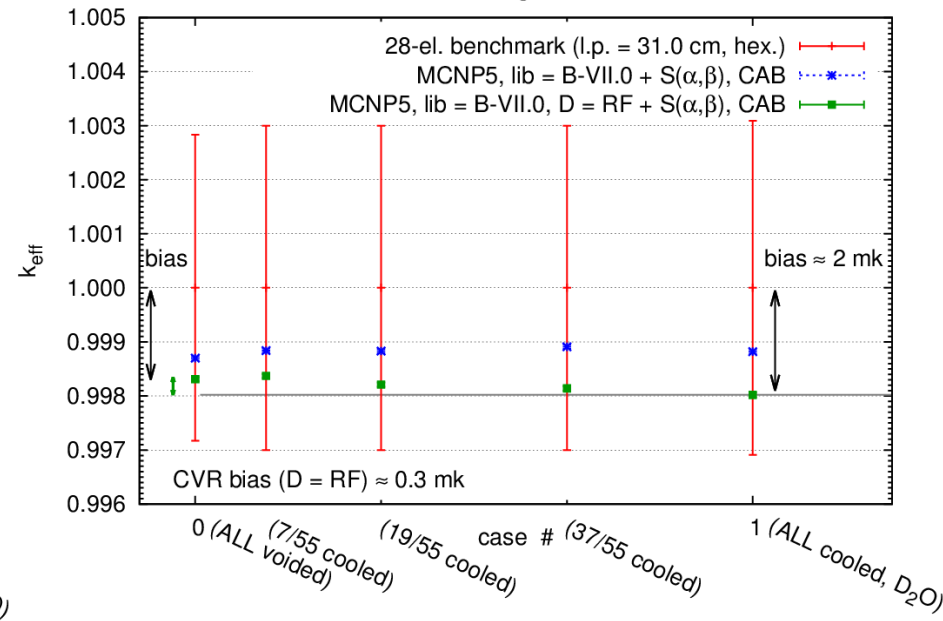
# TSL for heavy water (FY 2014/2015)

## interaction of thermal neutrons with liquid D<sub>2</sub>O (at room T)

28-el. bundle benchmark (NU UO<sub>2</sub>), ZED-2 reactor (CRL, AECL)  
MCNP models using Zed2MCNPWriter



28-el. bundle benchmark (NU UO<sub>2</sub>), ZED-2 reactor (CRL, AECL)  
MCNP models using Zed2MCNPWriter



Discuss: how accurately do we model criticality (critical zero-power assemblies), *i.e.*,  $k_{\text{eff}} = ?$   
 $k_{\text{eff}} = 1.0$ , in theory. In practice (*i.e.*, in modeling), .....

- we have a **Bias** (in  $k_{\text{eff}}$ )
- **CVR bias: assume we have cooled and voided critical cases, then :**

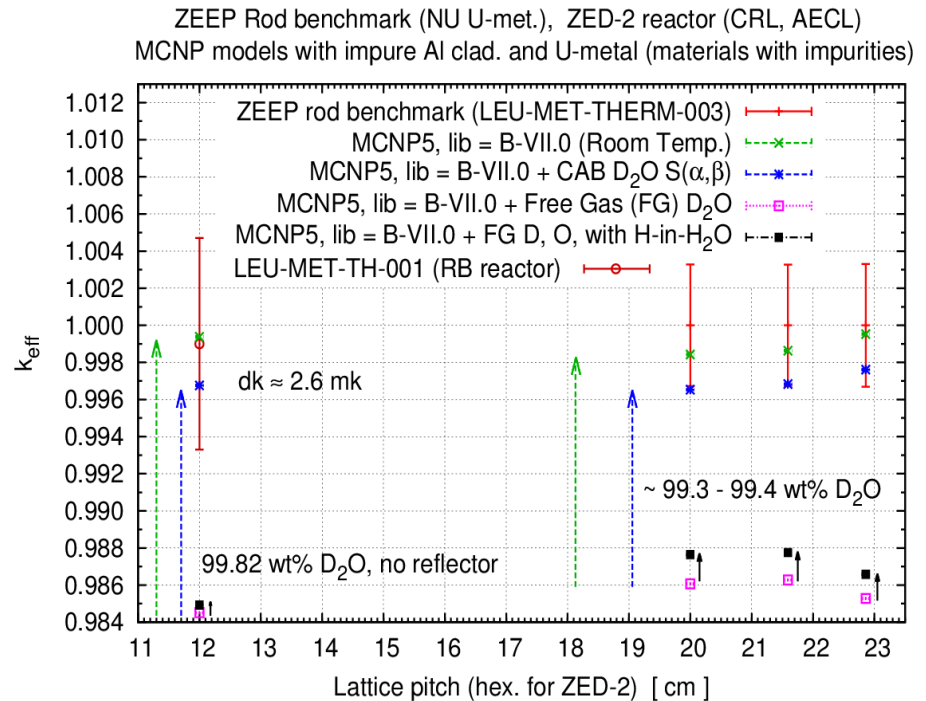
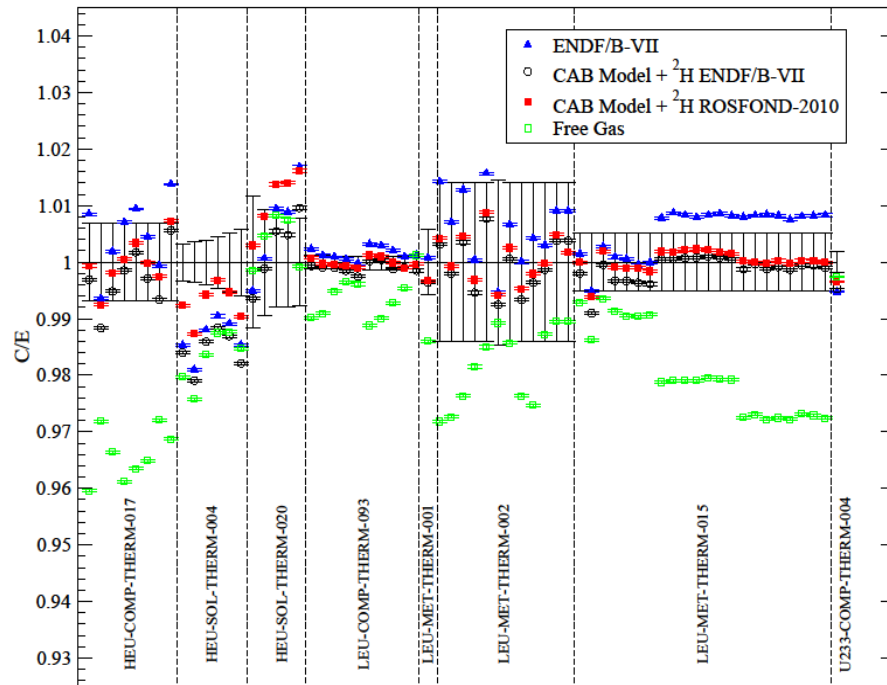
**Is any dependency / trend of  $k_{\text{eff}}$  bias upon voiding of the coolant in a crit. assembly ?**

**Is it possible to decrease the experimental (benchmark) uncertainty by factor of ... (3-5) ?**



# TSL for heavy water (FY 2014/2015)

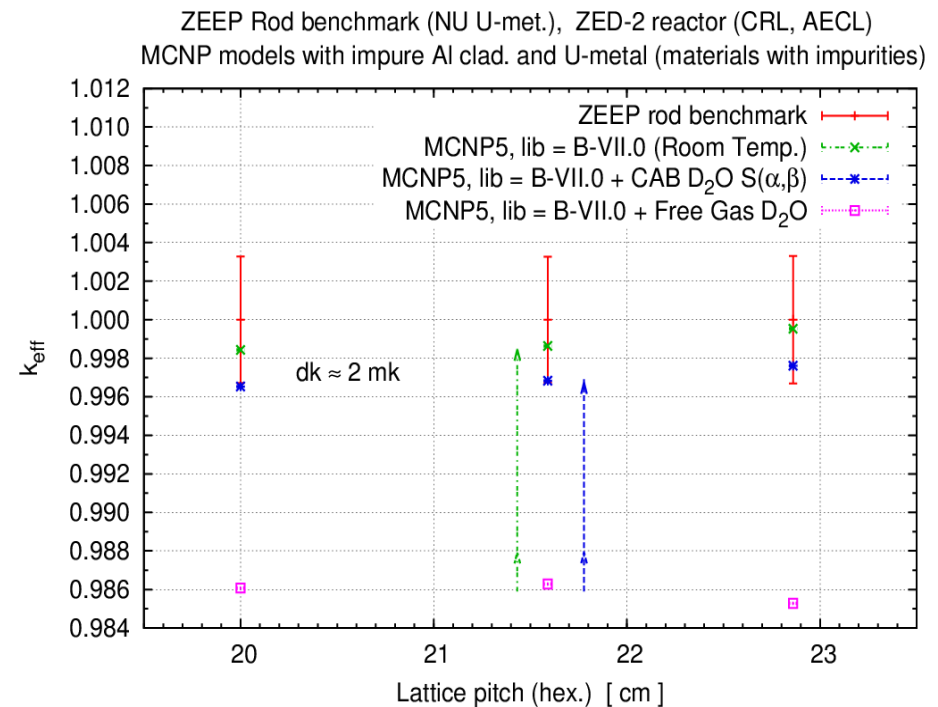
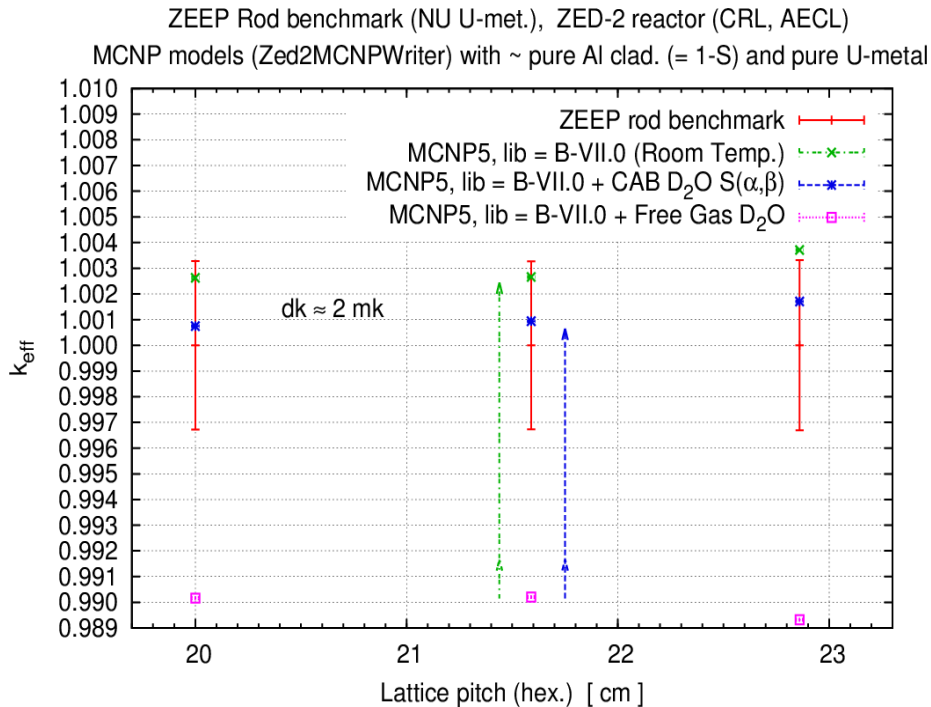
interaction of thermal neutrons with liquid D<sub>2</sub>O (at room T)



- **Testing:** using Crit. Safety **Benchmarks** with D<sub>2</sub>O, overall, it is an improvement in  $k_{eff}$  C/E **if** ENDF/**B-VII.0** D<sub>2</sub>O S( $\alpha,\beta$ )  $\rightarrow$  **CAB** D<sub>2</sub>O S( $\alpha,\beta$ )
- We expect that  $k_{eff}$  (**CAB** D<sub>2</sub>O)  $<$   $k_{eff}$  (**B-VII.0** D<sub>2</sub>O), but what is the difference  
 $dk = k_{eff} (\text{B-VII.0 D}_2\text{O}) - k_{eff} (\text{CAB D}_2\text{O}) = ?$ , **and check:**  $dk < \text{or } > \Delta k_{\text{Bench}}$  ?  
**answer for ZED-2 reactor critical core (CNL, Chalk River) :**  
 **$dk \sim 100$  pcm** using **LEU-MET-THERM-003** benchmark, note H-in-H<sub>2</sub>O for r.gr. heavy water  
 This is NU U metal in Al cladding, at room T, evaluator = J. Atfield (CNL)

# TSL for heavy water (FY 2014/2015)

## interaction of thermal neutrons with liquid D<sub>2</sub>O (at room T)



**LEW-MET-THERM-003:** ZEEP rods in ZED-2 reactor in CNL, Chalk River:

**art of modeling materials (J. Atfield): conversion of mass-spec data into MCNP data**

Example:

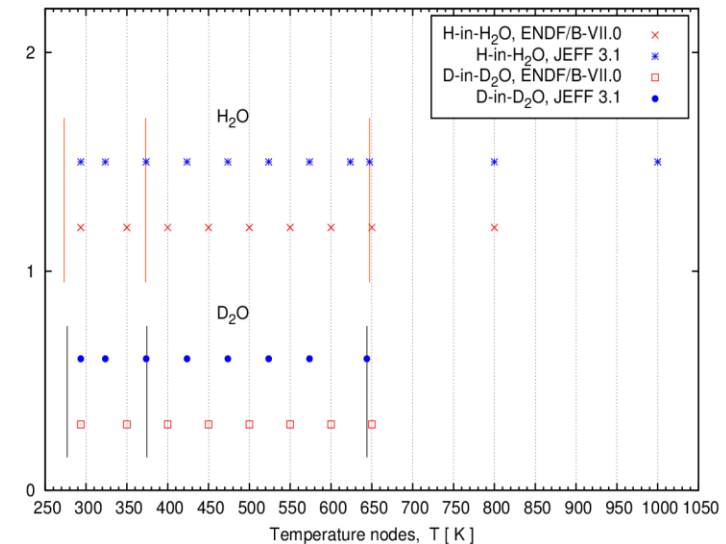
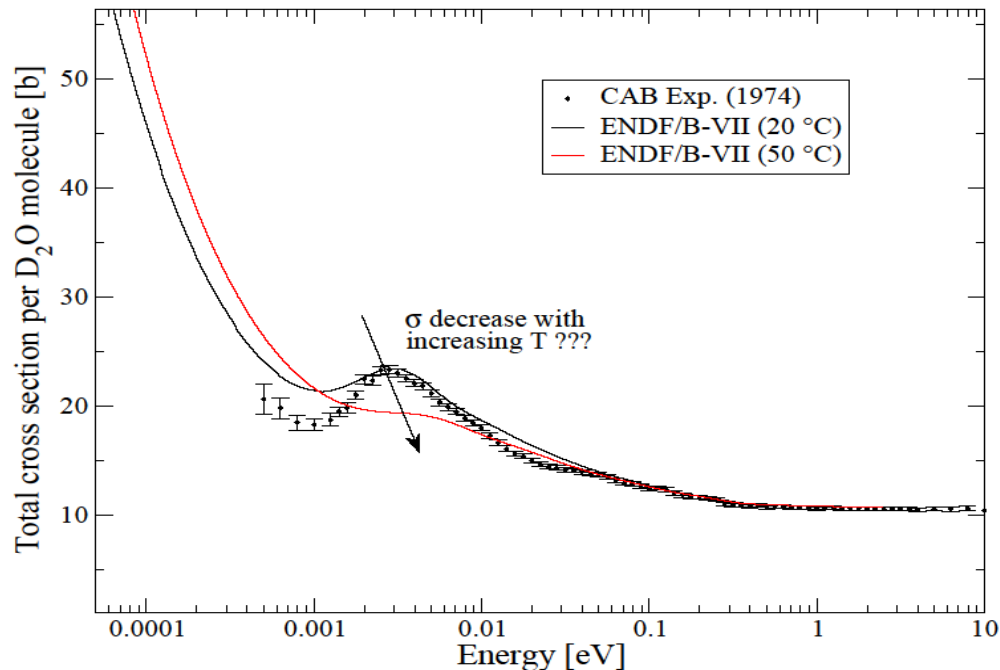
**benchmark  $k_{\text{eff}} = 1.0000 \pm 0.0033$**

**impurity worth  $\approx 4 \text{ mk}$  (e.g.,  $1.00371 - 0.99953 \approx 4.2 \text{ mk}$ , std. dev [ $k_{\text{eff}}$ ] =  $\pm 0.00004$ )**

**compare with  $S(\alpha,\beta)$  sensitivity:  $dk(\text{CAB vs. B-VII.0}) \approx -1.9 \text{ mk}$  (=  $0.99761 - 0.99953$ )**

# TSL for heavy water (2014/2015, 2015/2016 )

## interaction of thermal neutrons with liquid D<sub>2</sub>O at different $T$ and $p$



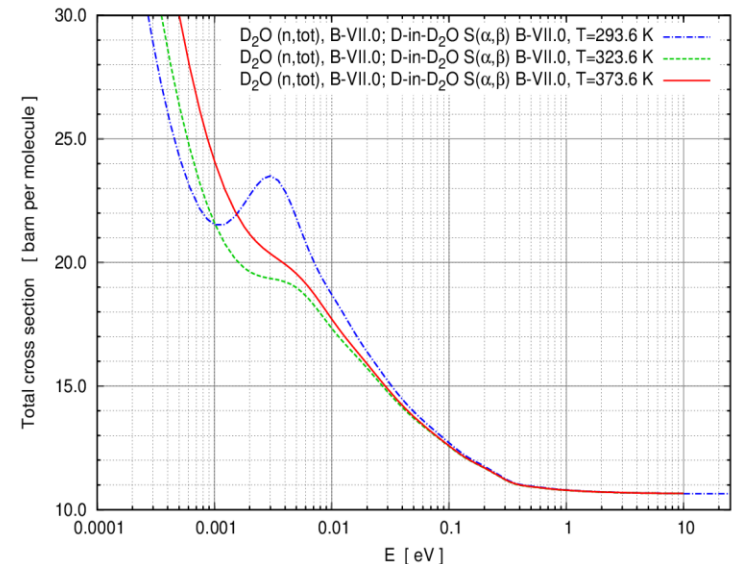
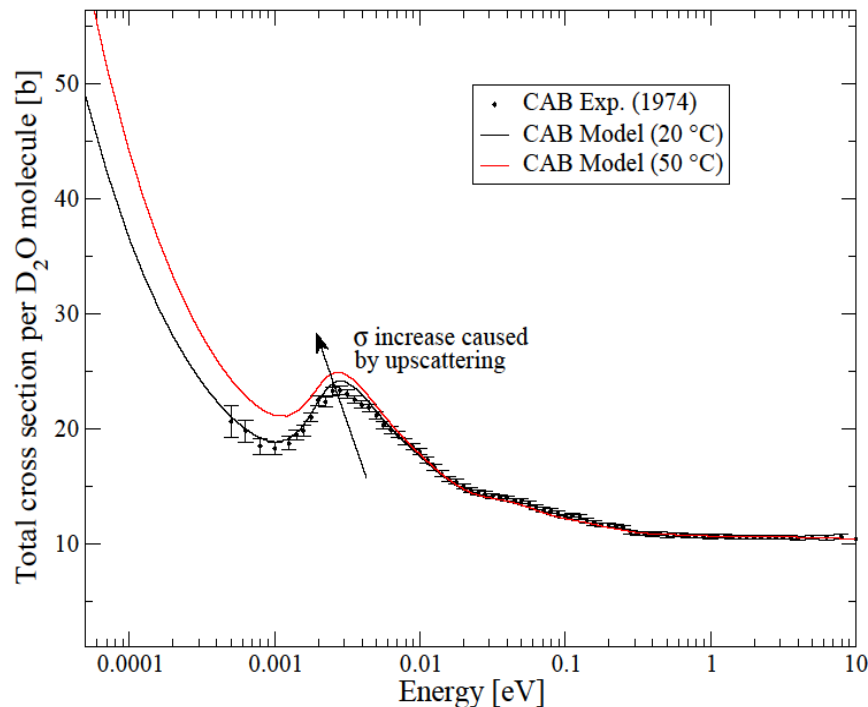
- **Inconsistency for heavy water TSL in modern evaluated ND libraries:**  
if  $T > T_{\text{room}}$  ( $p = 1$  atm), what happens with  $\sigma_s(E; T)$  ?
- D<sub>2</sub>O moderator T in CANDU-6:  $\approx 68$  deg. C ( $\approx 341$  K)
- D<sub>2</sub>O moderator T in some ZED-2 experiments up to  $\sim 50$ -60 deg C. **What for ?**  
Moder. T coefficient of reactivity is an important parameter in CANDU reactor physics analysis





# TSL for heavy water (2014/2015, 2015/2016 )

interaction of thermal neutrons with liquid D<sub>2</sub>O at different  $T$  and  $p$



- **Inconsistency (?) for heavy water in modern evaluated ND libraries:**  
if **100 °C** >  **$T$**  >  **$T_{\text{room}}$**  ( $p = 1$  atm), what happens with  $\sigma_s(E; T)$  ?
- New experimental results at 50 deg. C ( $\sim 323$  K) : J.I. Márquez Damián and D. Baxter
- What can we do to improve  $S(\alpha, \beta)$  for D<sub>2</sub>O at **100 °C** >  **$T$**  > **10 °C** (at  $p = 1$  atm) ?
- New (improved) **models** → **NJOY** (LEAPR) → ACE files & testing (benchmarking) using MCNP, **migrate** to GROMACS 5.0, NJOY99 → NJOY2012, MCNP5 → MCNP6 & SERPENT (student-friendly MC)

# TSL for heavy water: new & improved models → new evaluation

New **evaluation** (in **ENDF** format) can be based on combining **molecular dynamics (MD)** simulations and reliable *experimental data*, and the resulting new **models** can be implemented in / have to be compatible with / **LEAPR** module of **NJOY** ( nuclear data post-processing code, LANL, latest is **NJOY 2012** )

The **key points** for building new  $S(\alpha, \beta)$  **models** are:

- 1.** use of molecular **(self)diffusion** for translational motion of liquid  $\text{H}_2\text{O}$  /  $\text{D}_2\text{O}$  ( instead of **free gas approximation** (FG) used in **all** evaluated ND libraries );
- 2.** continuous **vibrational spectra** computed from molecular dynamics (**MD**) simulation at a given thermodynamic state of the liquid, (  $p$ ,  $T$  ) and density  $\rho(p, T)$ , (instead of derived / adjusted spectra from neutron scattering experiments);
- 3.** a more precise description of **the structure of liquid**: e.g., models for D **and** O in  $\text{D}_2\text{O}$  based on **experimental results** (instead of using the **incoherent approximation** in ENDF/B-VI or the Lennard-Jones **model** for D-D structure in JEFF 3.1 and ENDF/B-VII.0 → ENDF/B-VII.1)
- 4.** better **numerics** (e.g., extended grid(s),  $\alpha_i$ ,  $\beta_j$ ,  $T_n$ , and NJOY data processing options revisited and we need NJOY patches in leapr and thermr ; )
- 5.** **ACE files** to be generated for **testing/benchmarking** with MCNP5, MCNP6, and SERPENT

The resulting scattering kernels & cross sections will be an improvement over existing evaluations: **they are compared with measurements** of double differential scattering cross sections, quasi-elastic neutron scattering measurements, angular distributions of out-scattered neutrons, average cosine of the scattering angle ( $\mu$ -bar), and total cross sections;

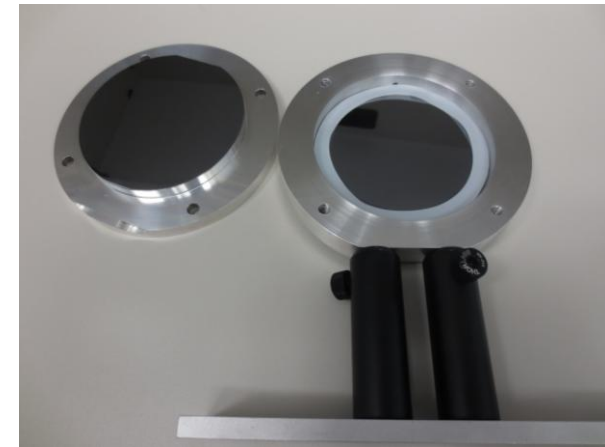
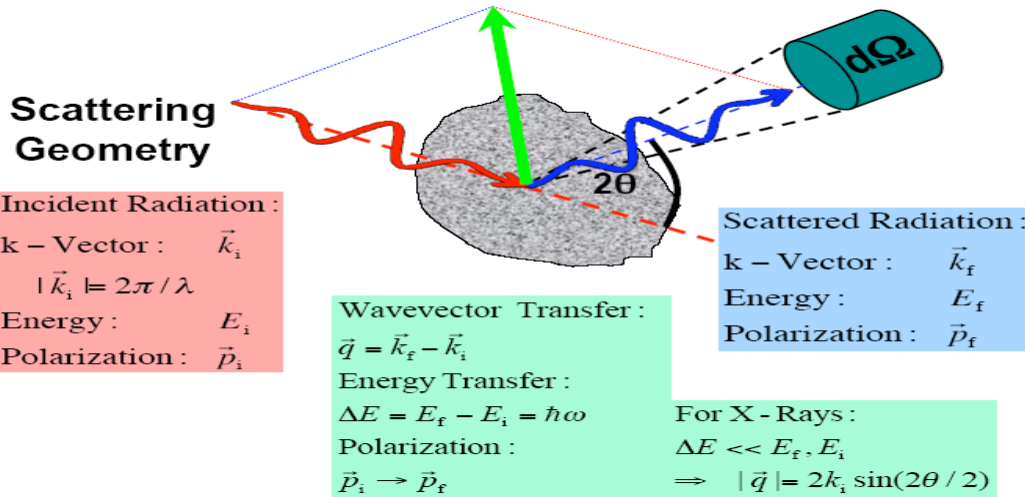
**Need to do all this at different (  $p$ ,  $T$  )**

# New TSL for heavy water and benchmarking: References

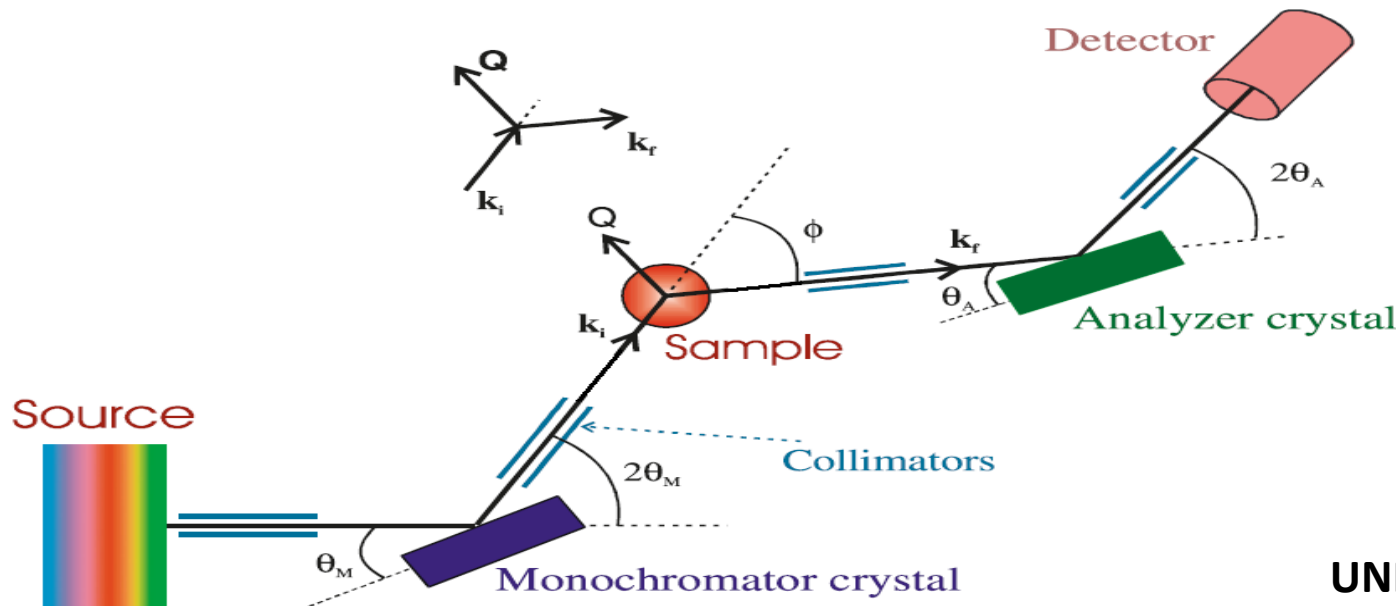
- J.I. Márquez Dámian, J.R. Granada, D.C. Malaspina,  
“*CAB models for water: A new evaluation of the thermal neutron scattering laws for light and heavy water in ENDF-6 format,*”  
Annals of Nuclear Energy, Vol. **65**, pp. 280-289, **2014** (March); [doi:10.1016/j.anucene.2013.11.014](https://doi.org/10.1016/j.anucene.2013.11.014)  
<http://www.sciencedirect.com/science/article/pii/S0306454913005987>
- J.I. Márquez Dámian, J.R. Granada, D. Roubtsov,  
“*Improvement on the calculation of  $D_2O$  moderated critical systems with new thermal neutron scattering libraries,*”  
Annals of Nuclear Energy, Vol. **71**, pp. 206-210, **2014** (September); [doi:10.1016/j.anucene.2014.03.024](https://doi.org/10.1016/j.anucene.2014.03.024)  
<http://www.sciencedirect.com/science/article/pii/S0306454914001467>
- J.I. Márquez Damián, J.R. Granada, D. Roubtsov, J.C. Chow,  
“*From Molecular Dynamics to Reactor Physics: Improvement on the Calculation of  $D_2O$  Moderated Critical Systems with New Thermal Neutron Scattering Libraries,*”  
in Proceedings of XXI Congress of Numerical Methods and Their Applications (**ENIEF 2014**),  
Bariloche, September 2014, Argentina;  
<http://www.cimec.org.ar/ojs/index.php/mc/article/view/4899>



# Neutron Scattering on Water using triple axis spectrometer at NRU reactor n beam at CNL (Chalk River)



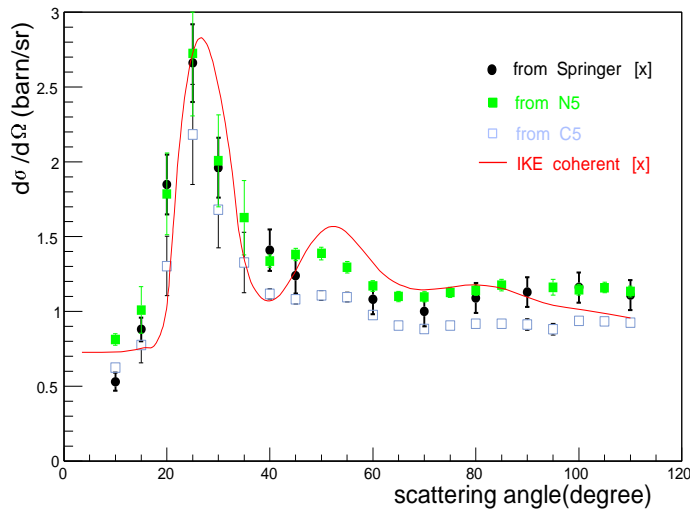
**Sample sizes:  
0.1 mm to 3 mm**



**Triple axis  
spectrometer**

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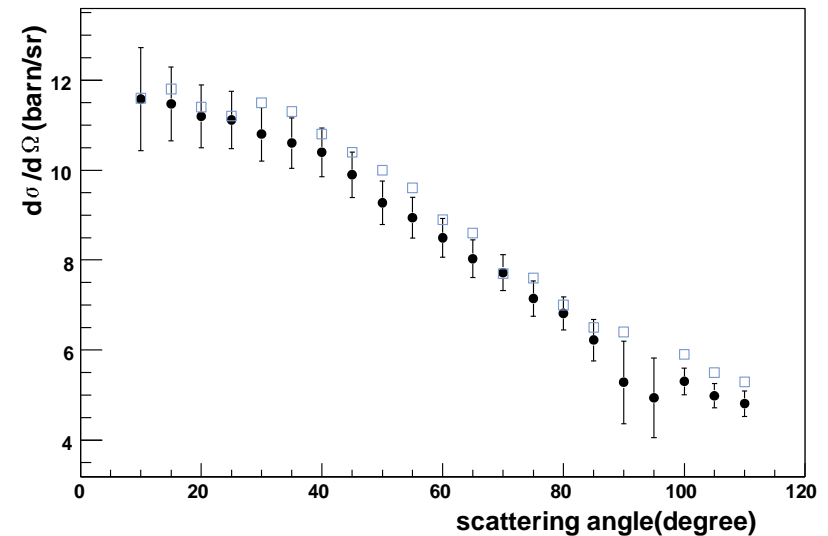
# New Measurements at NRU to reduce ND uncertainties to improve Reactor Safety



Differential scattering cross section of **D<sub>2</sub>O**.

Beam energy is 44 meV

(except for that the result from C5 is at  $E_0 = 41.44$  meV).



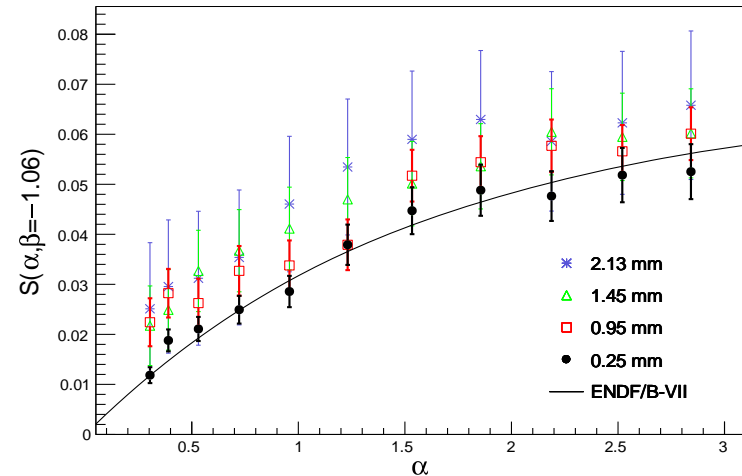
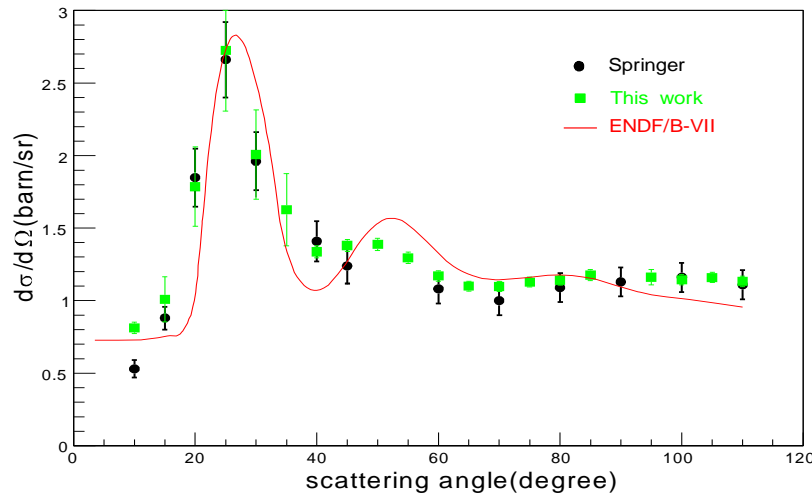
Absolute differential cross section (in barn/sr)

for **Light water**, at  $E_0 = 41$  meV,  
Beyster's data:  $\pm 5\%$ .

- G. Bentoumi, G. Li, B. Sur, and Z. Tun (Canadian Neutron Beam Centre)



# New Measurements at NRU to reduce ND uncertainties to improve Reactor Safety



NRU measured differential cross sections of  $D_2O$  (Beam energy is 44 meV) and  $S(\alpha, \beta)$  functions of  $H_2O$  (using samples with different widths), after multiple-scattering corrections, all at room T

## Importance:

- Safety and operating margins for nuclear reactors are crucially dependant on the availability and accuracy of nuclear data that are used for modeling, in particular for **safety analysis** and **licensing applications**

## Objectives:

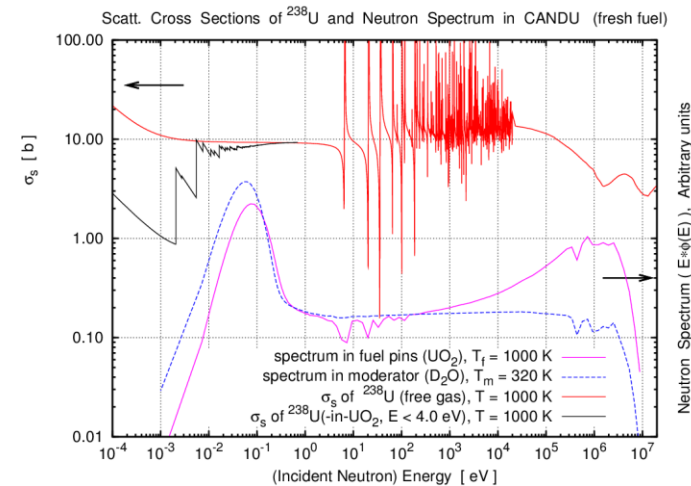
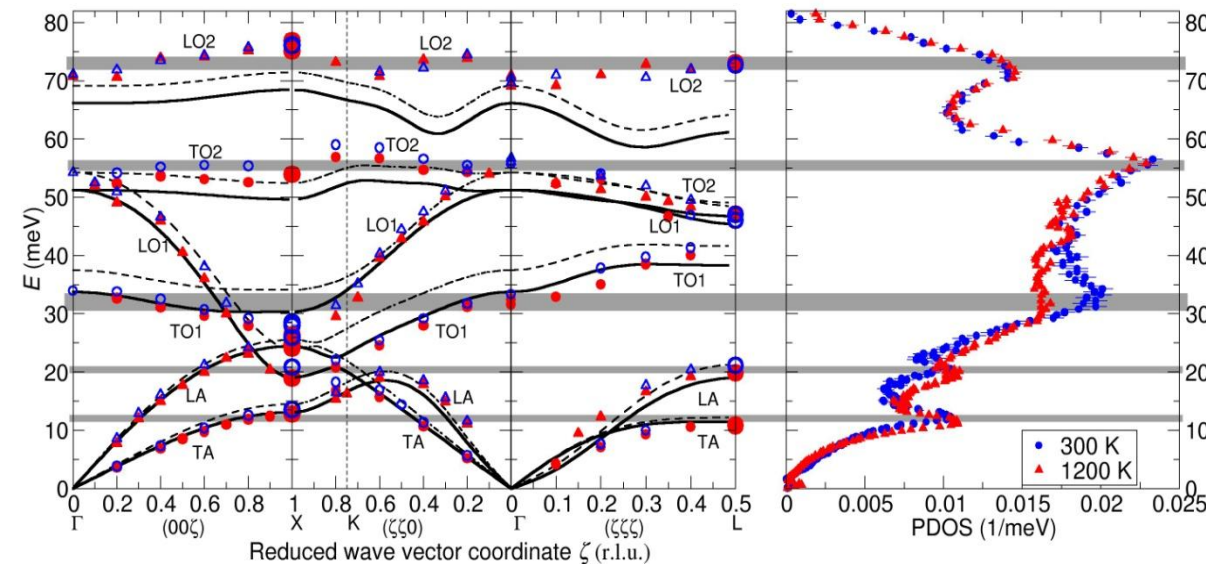
1. Carry-out new experiments at neutron beam facilities, such as NRU and (?) SNS (ORNL), and also at ZED-2 (CRL), to accurately measure the relevant nuclear data parameters
2. Investigate the discrepancy between the information obtained experimentally and existing nuclear data evaluations
3. **Compare new experimental results with predictions based on new models**





# TSL for $\text{UO}_2$ : phonon PDOS + coh. elastic

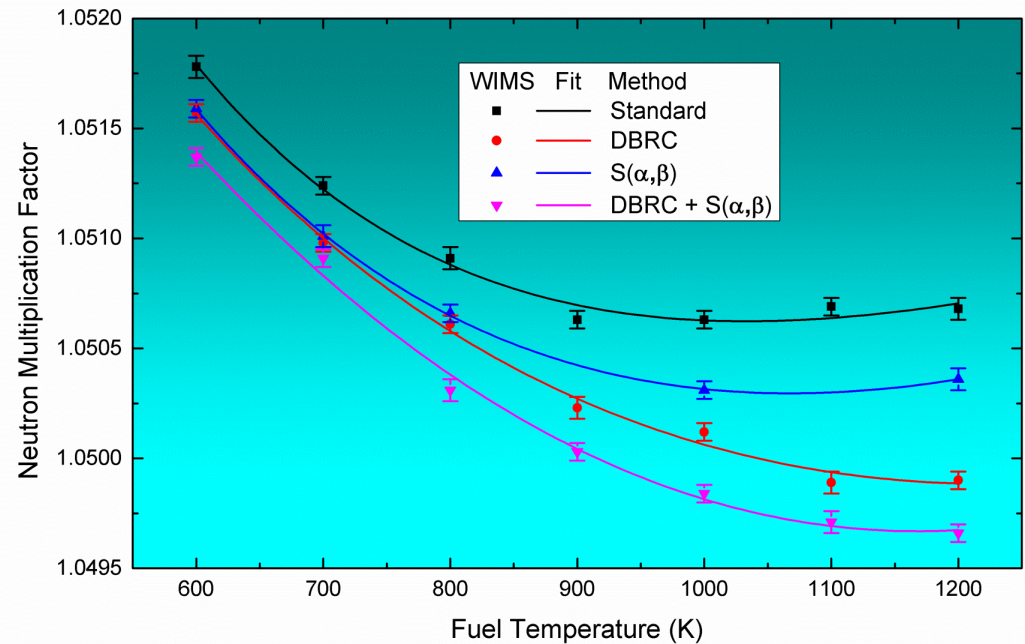
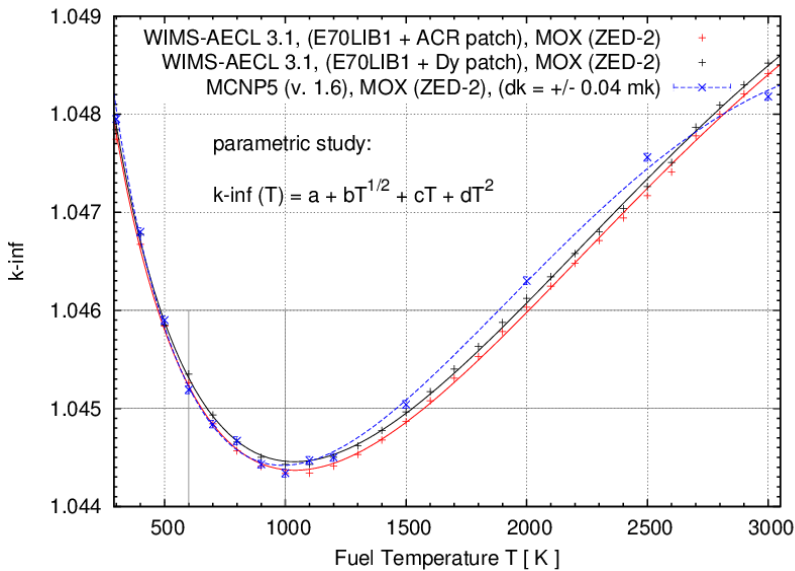
Phonons in actinides: active area of research,  
Judy Pang *et al.*, Phonon density of states and anharmonicity of  $\text{UO}_2$ ,  
PHYSICAL REVIEW B 89, 115132 (2014)



In reactor physics, for safety and licensing application:  
fuel temperature coefficient of reactivity (FTC);  $T$  can go  $\sim 2000$  K



# MCNP Results: Criticality Calculations, $k_{\text{inf}}$ vs. $T_f$



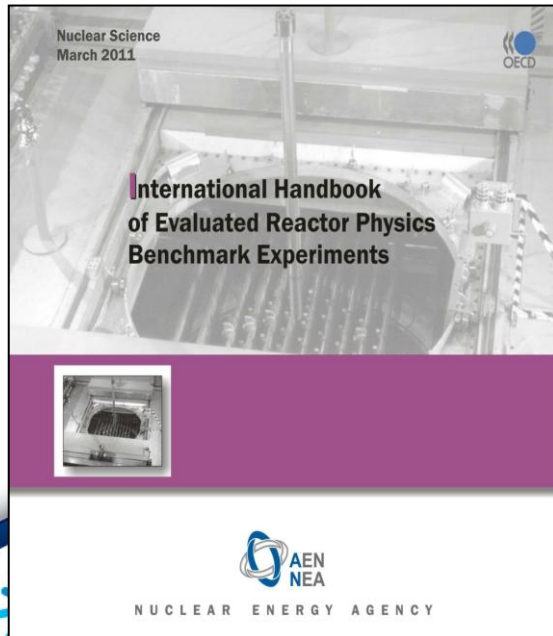
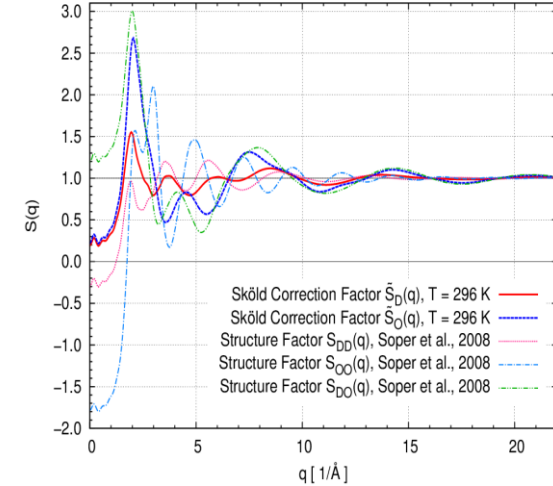
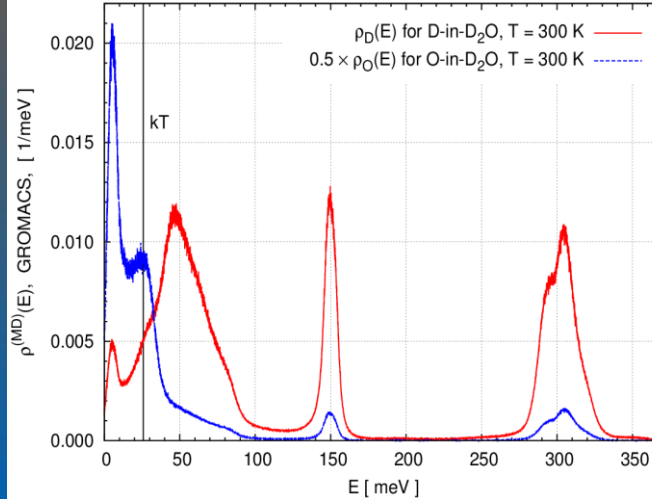
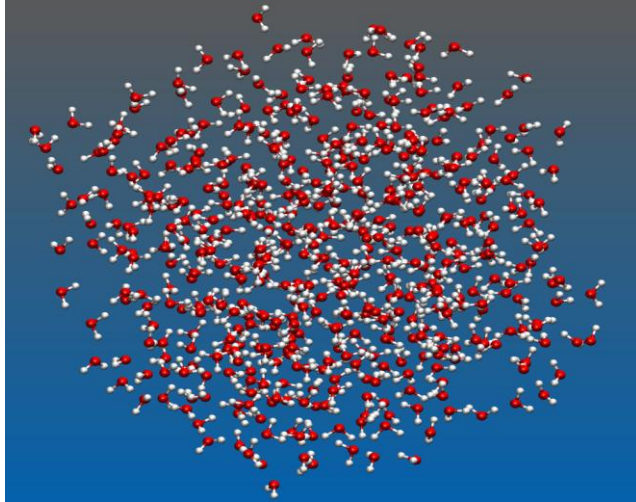
PHWR typical fuel bundle with  $\text{UO}_2$  fuel near the middle of burn-up  
 D. Altiparmakov and D.R.



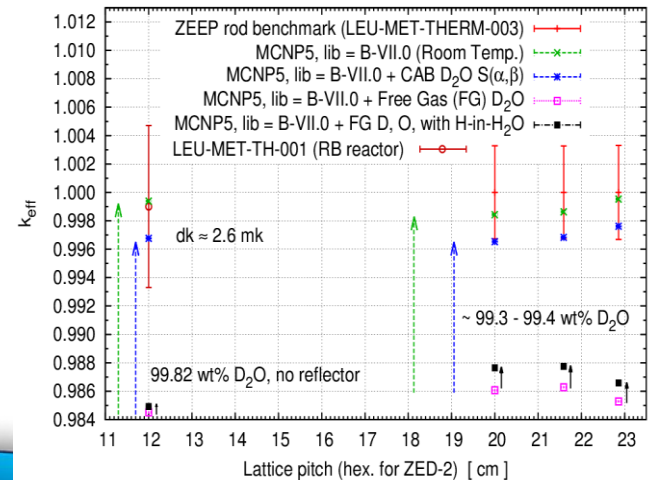
# CNL Collaborations in Nuclear Data R&D

- Member of **CSEWG, USA**  
CSEWG = Cross Section Evaluation Working Group  
(responsible for the U.S. Evaluated ND Files ENDF/B, latest = ENDF/B-**VII.1**, 2011)  
<https://www.nndc.bnl.gov/csewg/>
- Participation in **WPEC** sub-group activity under **OECD/NEA (EU)**  
WPEC = Working Party on International Nuclear Data Evaluation Co-operation,  
<https://www.oecd-nea.org/science/wpec/>  
e.g., **sub-group 40:**  
**The CIELO Collaboration: Neutron Reactions on  $^1\text{H}$ ,  $^{16}\text{O}$ ,  $^{56}\text{Fe}$ ,  $^{235,238}\text{U}$ , and  $^{239}\text{Pu}$ ,** <https://www.oecd-nea.org/science/wpec/sg40-cielo/>
- Continue our collaboration with scientists from Neutron Physics Department,  
**Centro Atómico Bariloche (CAB), Argentina:** New TSL evaluations for **D<sub>2</sub>O**, **H<sub>2</sub>O**  
and testing them using International Benchmarks (Crit. Saf. and Reactor Phys.)
- INERI USA-Canada R&D Collaboration, **2015 - ... :**  
**new collaborations** with US scientists (ORNL, LANL, ..., academia): ?

# TSL ( $S(\alpha, \beta; T)$ ) of water in a nutshell: from MD to RP



ZEEP Rod benchmark (NU U-met.), ZED-2 reactor (CRL, AECL)  
 MCNP models with impure Al clad. and U-metal (materials with impurities)



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