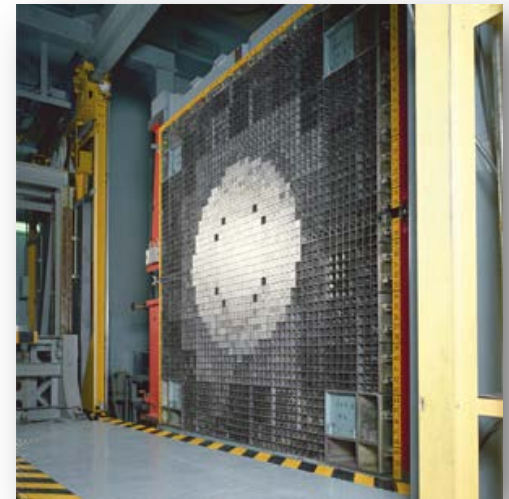


Benchmark tests for **ENDF/B-VIII.0 beta1** using *Sodium-void reactivity worth of FCA-XXVII-1*

M. Fukushima, K. Yokoyama,
O. Iwamoto, Y. Nagaya

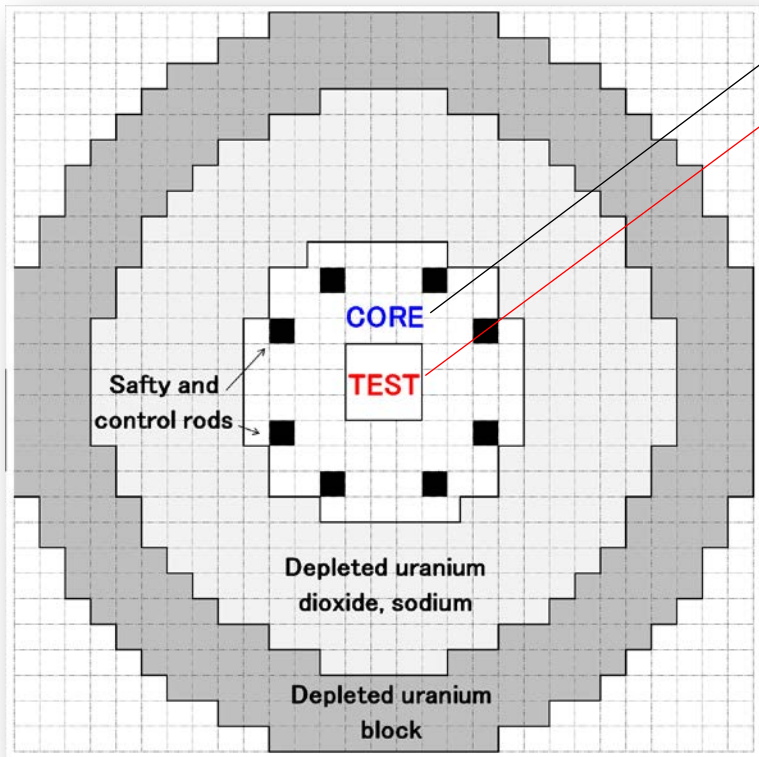


Fast Critical Assembly

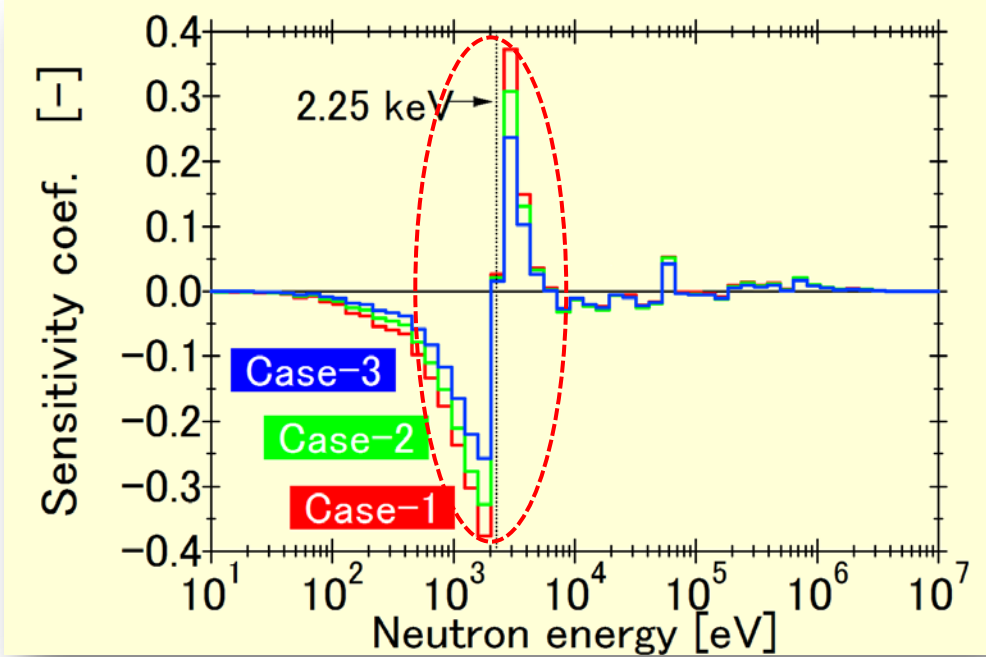
Sodium-void reactivity worth (SVRW) of FCA XXVII-1

M. Fukushima, et al., Prog. Nucl. Sci. Tech. 2, 306-311 (2011).

XY cross sectional view of FCA XXVII-1



Core region: Enriched uranium + Graphite
Test region for void measurements: Enriched uranium + Sodium
 (Average enrichment of ^{235}U : 30%)



Height of Test region	Case-1	Case-2	Case-3
	10.16cm	20.32cm	30.48cm

SVRWs were measured by replacing sodium-filled SS cans with empty cans

Sensitive to ^{235}U capture cross section around keV region
 owing to ^{23}Na resonance around 3 keV

Condition of benchmark tests and sensitivity analyses

□ Benchmark tests

Code: **Continuous-energy Monte Carlo code, MVP**

$$\rho = \left(k_{eff}^{void} - k_{eff}^{ref} \right) / \left(k_{eff}^{void} \cdot k_{eff}^{ref} \right)$$

200 million histories for each k_{eff} calculations

□ Sensitivity Analyses

Code: Generalized perturbation code SAGEP based on diffusion theory

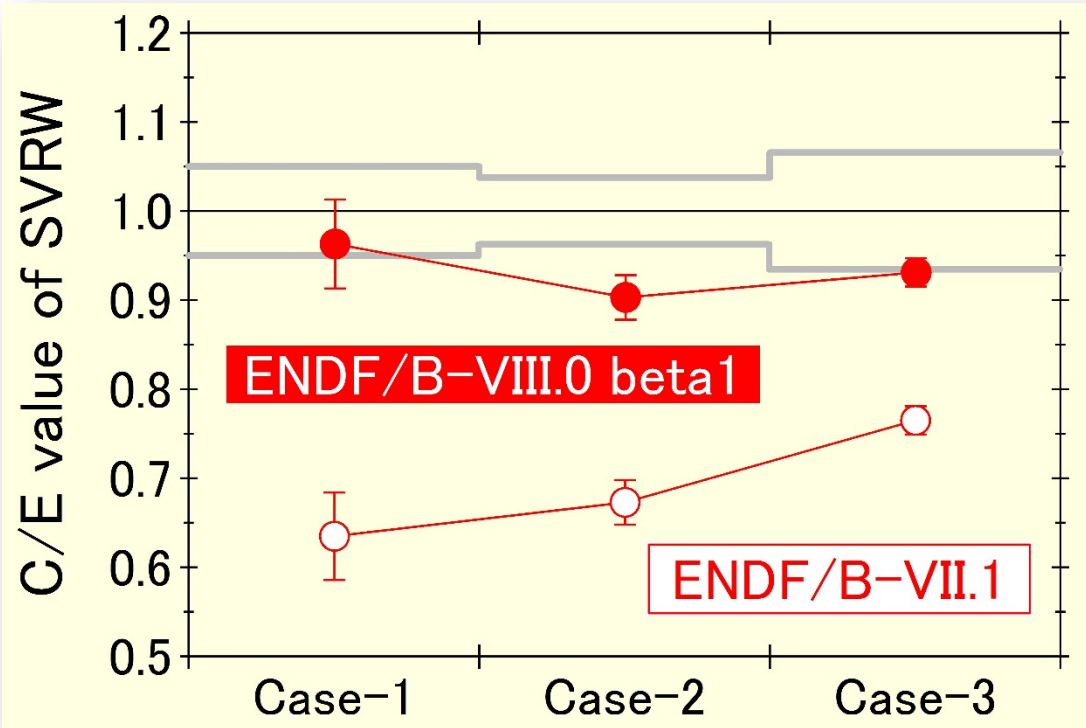
Sensitivity coefficients:
$$S_{X,G}^{B71} = \left(\frac{d\rho/\rho}{d\sigma_{X,G}^{B71}/\sigma_{X,G}^{B71}} \right)$$

Here, **X(nuclide, reaction)**, **G(energy group)**

In replacement from **ENDF/B-VII.1(B71)** to **-VIII beta (B8b1)**,
nuclide, reaction- and **energy-wise** changes were estimated by

$$\left(\frac{d\rho}{\rho} \right)_{X,G}^{B71 \rightarrow B8b1} = \underbrace{S_{X,G}^{B71}(R)}_{\text{sensitivity coefficients}} \cdot \underbrace{\left(\frac{\sigma_{X,G}^{B8b1} - \sigma_{X,G}^{B71}}{\sigma_{X,G}^{B71}} \right)}_{\text{relative discrepancy between B71 and B8b1}}$$

Benchmark results by MC calculations



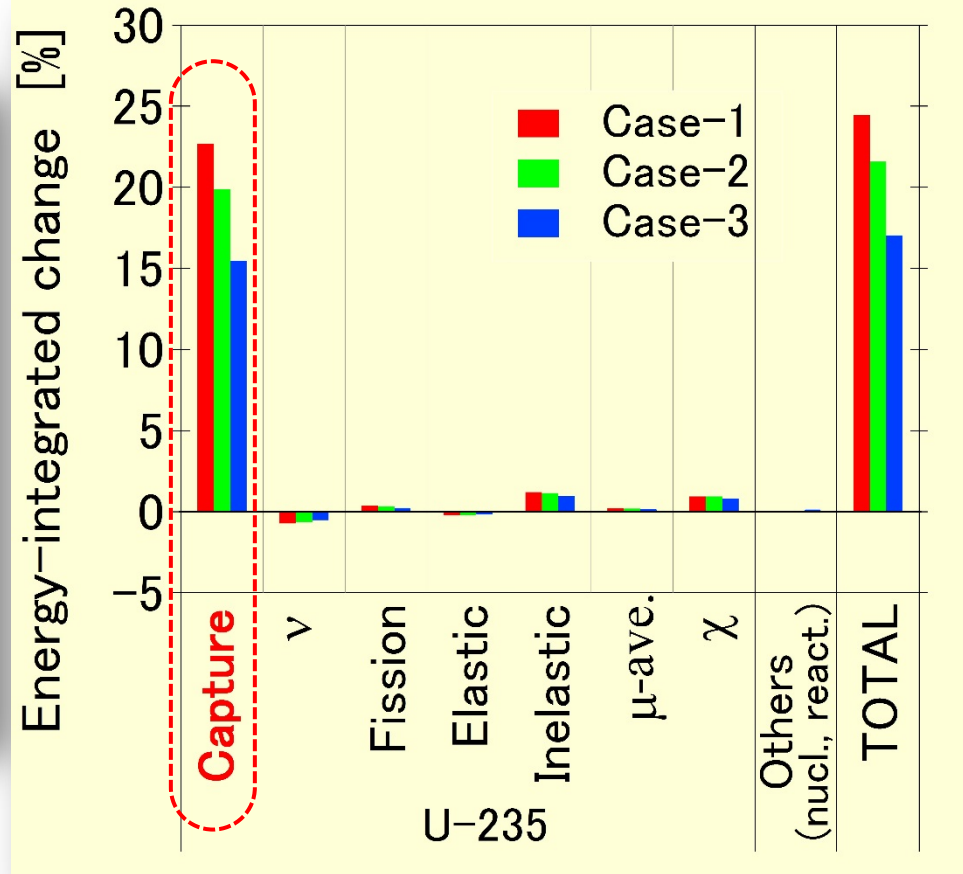
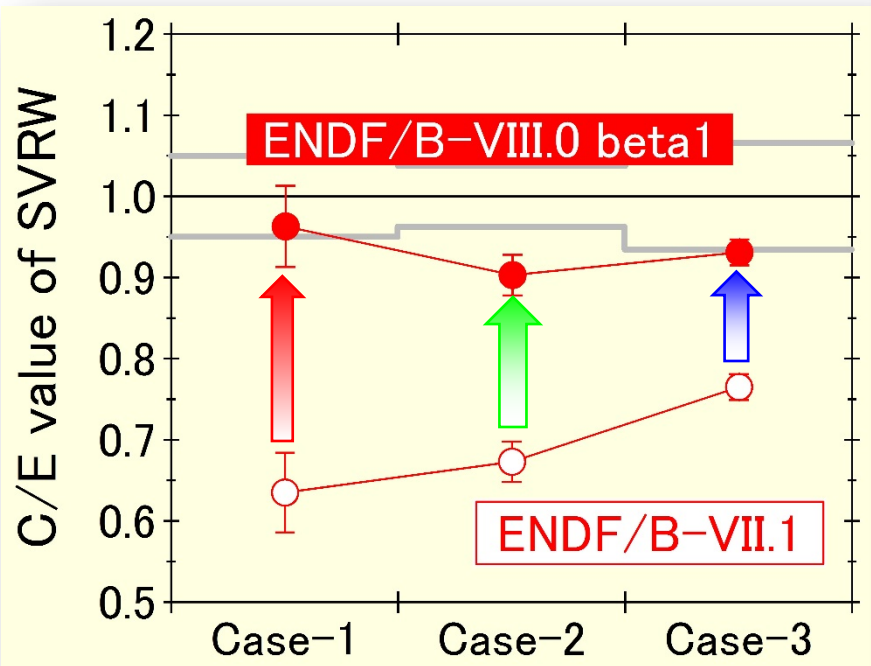
C/E ENDF/B- VIII.0 beta1	
Case-1	0.963 $\pm 0.049^{*1} \pm 0.048^{*2}$
Case-2	0.903 $\pm 0.025^{*1} \pm 0.034^{*2}$
Case-3	0.931 $\pm 0.016^{*1} \pm 0.061^{*2}$

C/E ENDF/B-VII.1	
Case-1	0.635 $\pm 0.049^{*1} \pm 0.032^{*2}$
Case-2	0.673 $\pm 0.025^{*1} \pm 0.025^{*2}$
Case-3	0.765 $\pm 0.016^{*1} \pm 0.050^{*2}$

*¹ Due to MC statistical errors.
 *² Due to Experimental uncertainties.

➤ ENDF/B-VIII.0 beta1 obviously improves the C/E values by ENDF/B-VII.1 for sodium-void reactivity worth of FCA-XXVII-1

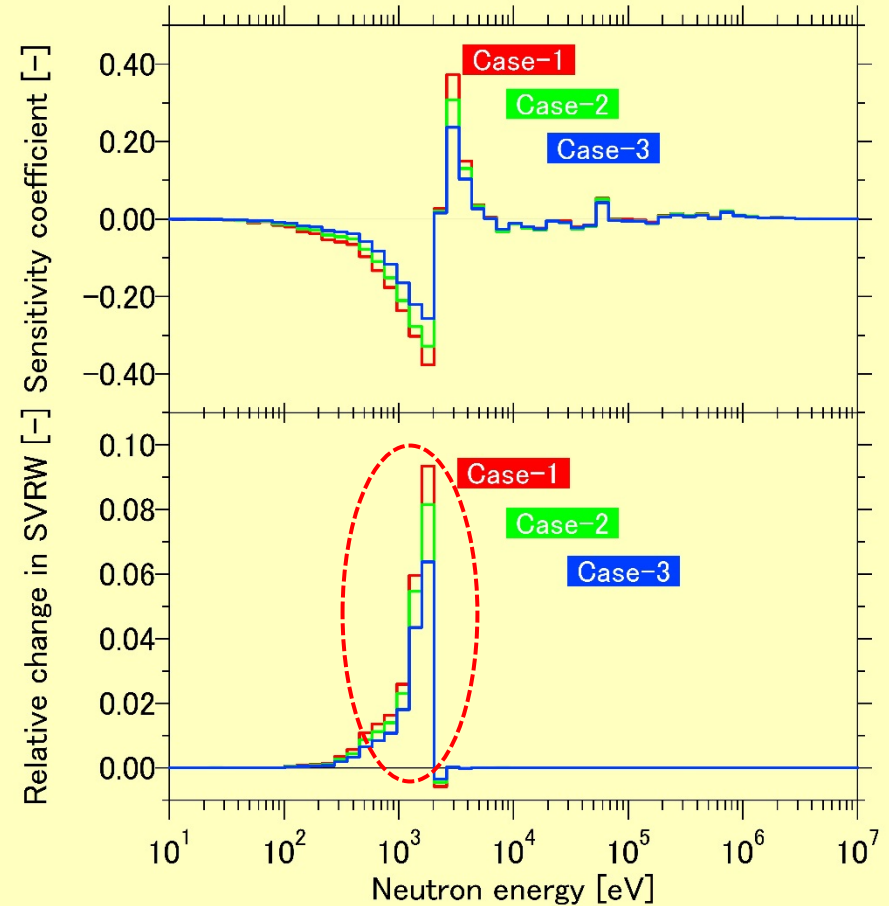
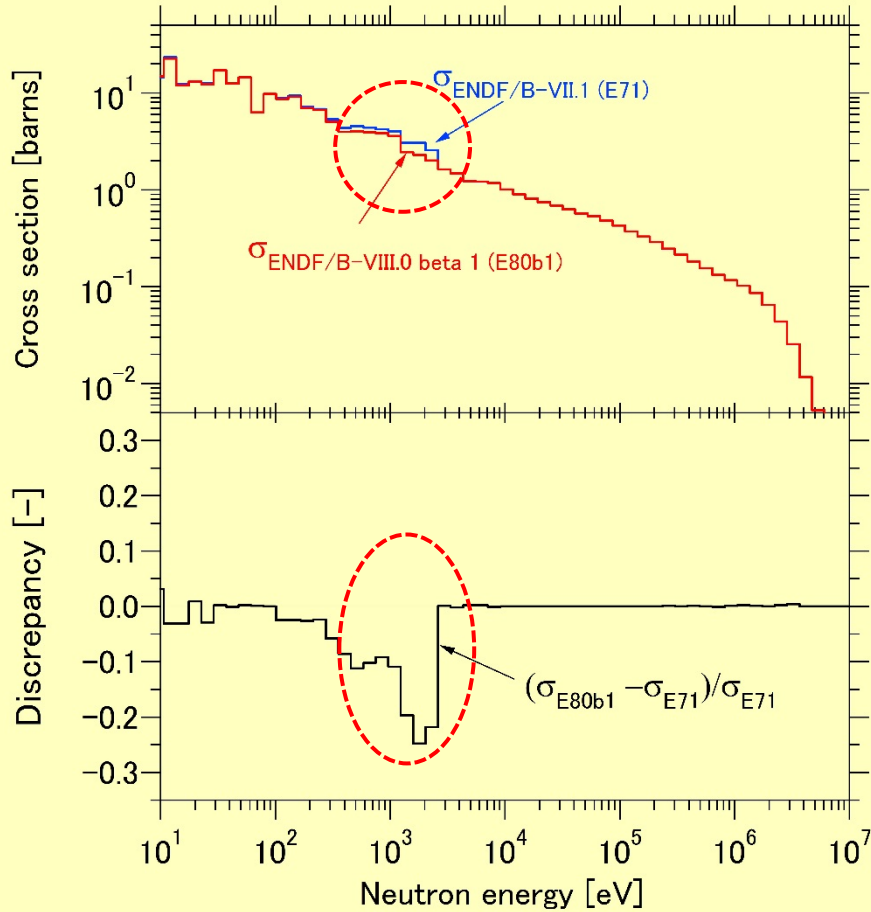
Sensitivity Analyses



Replacement from B71 to B80b1
increases void reactivity worth

Positive changes are mainly due to discrepancy between ²³⁵U capture cross sections

Sensitivity Analyses



Improvements from B71 to B80b1 are mainly caused by discrepancy of ^{235}U capture around keV region

Summary

- ✓ Adoption of **ENDF/B-VIII.0 beta1** instead of ENDF/B-VII.1 leads **obvious improvements** of the C/E values for sodium void reactivity worth of FCA XXVII-1.
- ✓ The improvements were caused mainly by the discrepancy between **^{235}U capture cross sections around keV region.**