

Application of Nuclear Data Libraries in Fusion Neutronics and Some Comments

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**WPEC Subgroup #38
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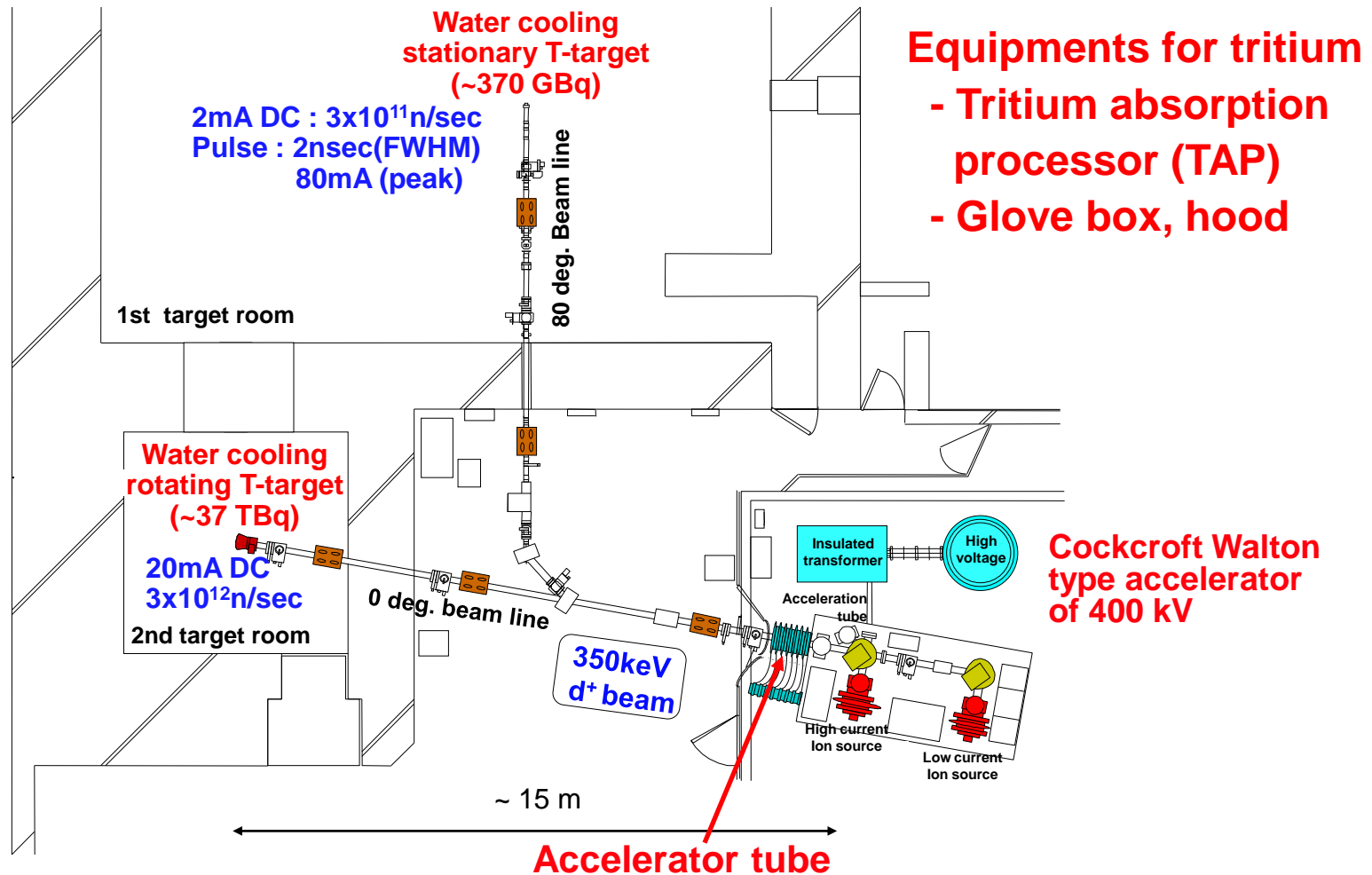
Japan Atomic Energy Agency, Tokai, Ibaraki, Japan

- 1. Introduction of our group (Fusion Neutronics Group)**
2. Our recent studies related to nuclear data

Fusion Neutronics Source (FNS)

JAEA/FNS [3]

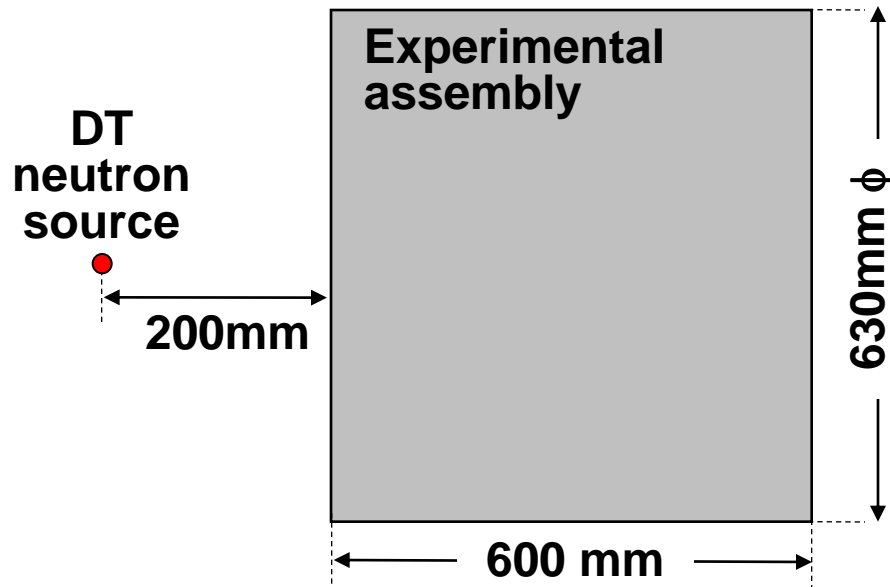
- We have an accelerator-based DT neutron source **FNS** (Fusion Neutronics Source) at Tokai site in JAEA.



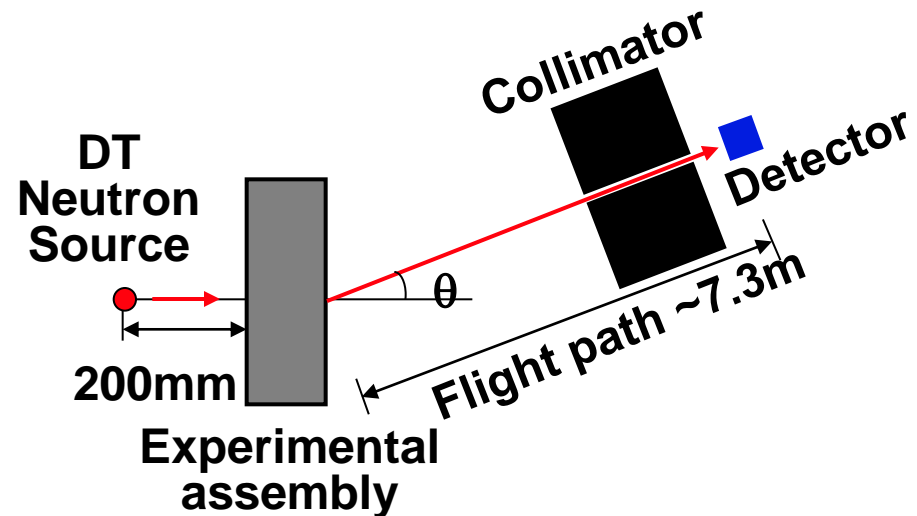
Experimental activities related to nuclear data

JAEA/FNS [4]

- We have performed fusion neutronics **experiments for nuclear data benchmarking** with FNS since 1981.
 - **In-situ experiment**
 - **Time-Of-Flight (TOF) experiment**
 - Li_2O , Beryllium, Graphite, SiC, **Titanium**, Vanadium, Iron, SS316, Copper, Lead, Tungsten, etc.



In-situ experiment



TOF experiment

Calculation activities related to nuclear data

JAEA/FNS [5]

- ❑ We **analyze these experiments with nuclear data** and investigate **problems on nuclear data** including ACE, MATXS and AMPX files and nuclear analysis codes.
 - Nuclear data library : JENDL, ENDF/B, JEFF, FENDL, etc.
 - Processing code : NJOY, TRANSX, SCAMPI
 - Transport code : MCNP, TRIPOLI, DOORS
 - Sensitivity/uncertainty code : SUSDFNS
- ❑ We also use nuclear data libraries for **nuclear designs of fusion reactors** such as International Thermonuclear Experimental Reactor (ITER), including activation analyses.

1. Introduction of our group (Fusion Neutronics Group)
2. **Our recent studies related to nuclear data**

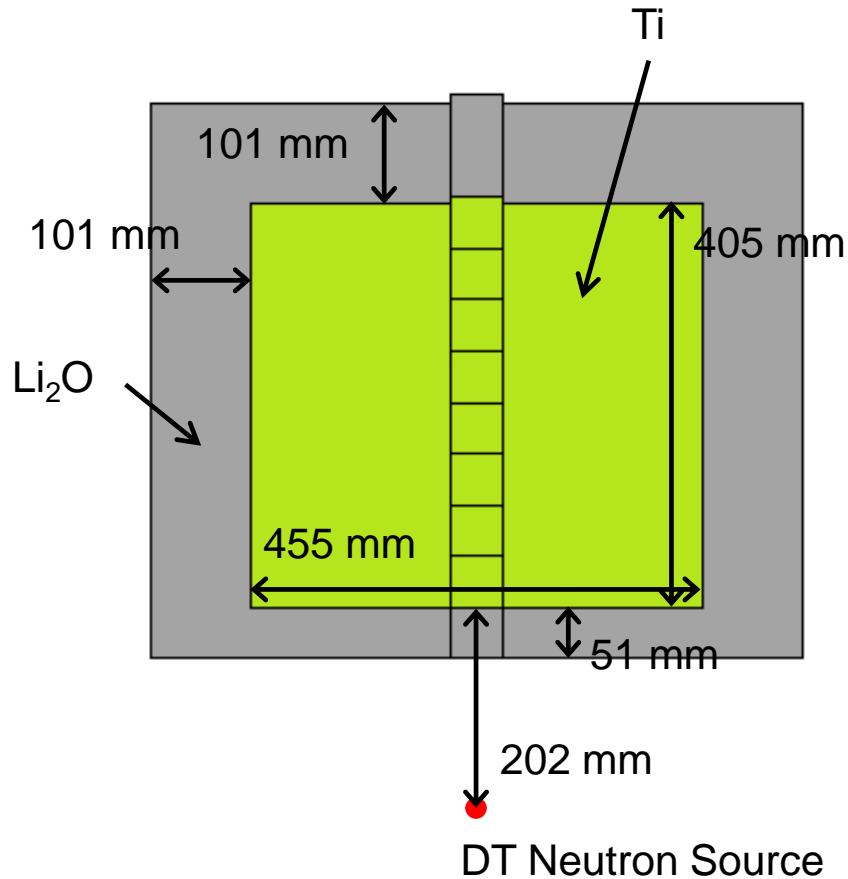
Recent studies related to nuclear data

JAEA/FNS [7]

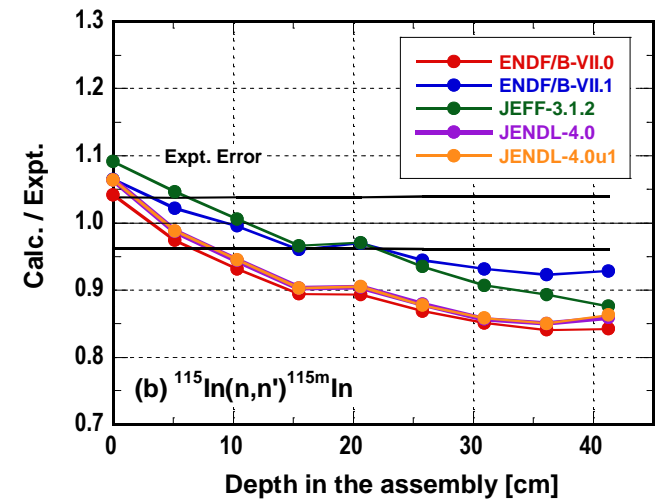
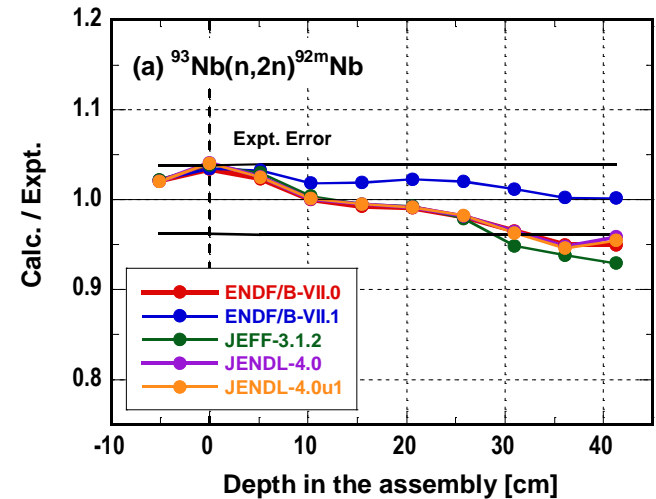
- Now I present our following studies related to nuclear data libraries and processed files in brief.
 - Latest in-situ experiment on **titanium**
 - **Weighting function** and **background cross section data** in multigroup libraries
 - Self-shielding for **unresolved resonance data**
 - **KERMA** factors

Titanium Benchmark Expt. –(1)

JAEA/FNS [8]



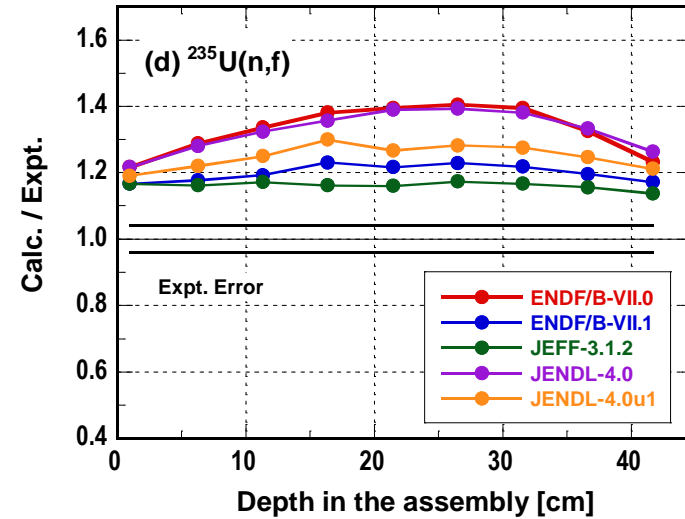
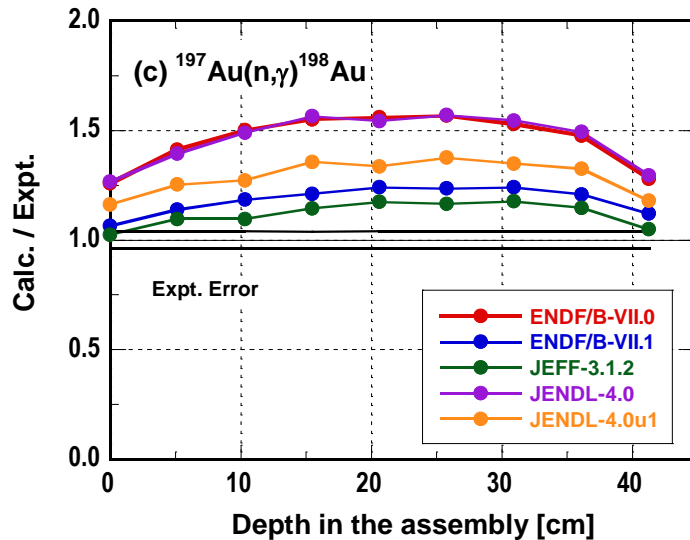
Rectangular assembly



Comparison between calc. and expt. results

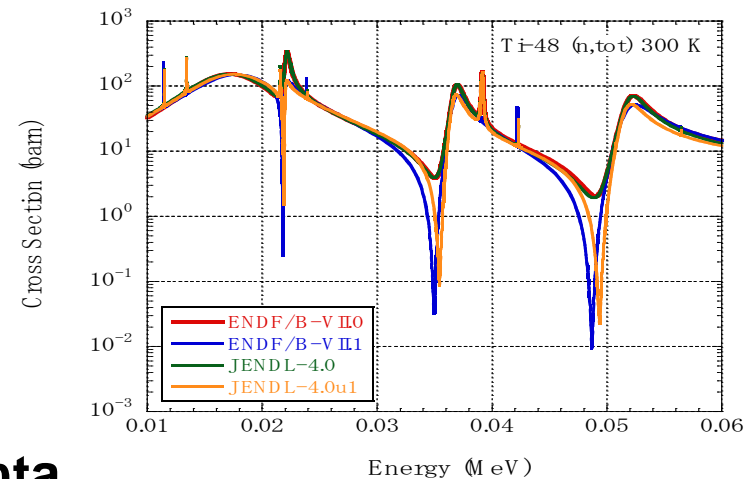
Titanium Benchmark Expt. –(1)

JAEA/FNS [9]



Comparison between calc. and expt. results

We found out that titanium data in JENDL-4.0 should be revised for the **(n,2n) reaction cross section data and resonance parameters.**



Weighting function and background cross section data in multigroup libraries –(1)

JAEA/FNS [10]

Adequate weighting function for multigroup libraries

$$W_{\ell}(E) = \frac{C(E)}{[S_0 + S_t(E)]^{\ell+1}}$$

$C(E)$: smooth flux
 σ_0 : background cross section
 σ_t : total cross section

This ℓ is not considered in most cases.

The lowest background cross section is too high in most cases.

σ_0 of ~ 0.02 barn is required for ^{56}Fe in natural iron, but it is 0.1 or 1.0 in most cases.

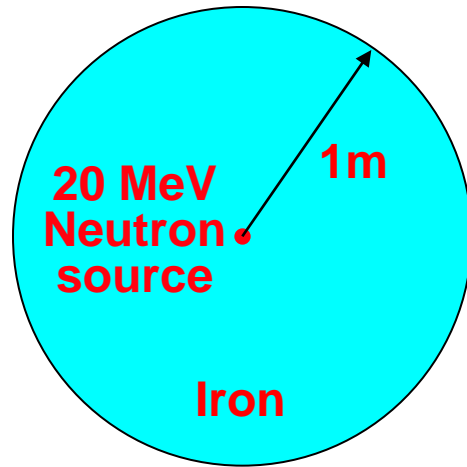


Self-shielding correction is not adequate in most cases!

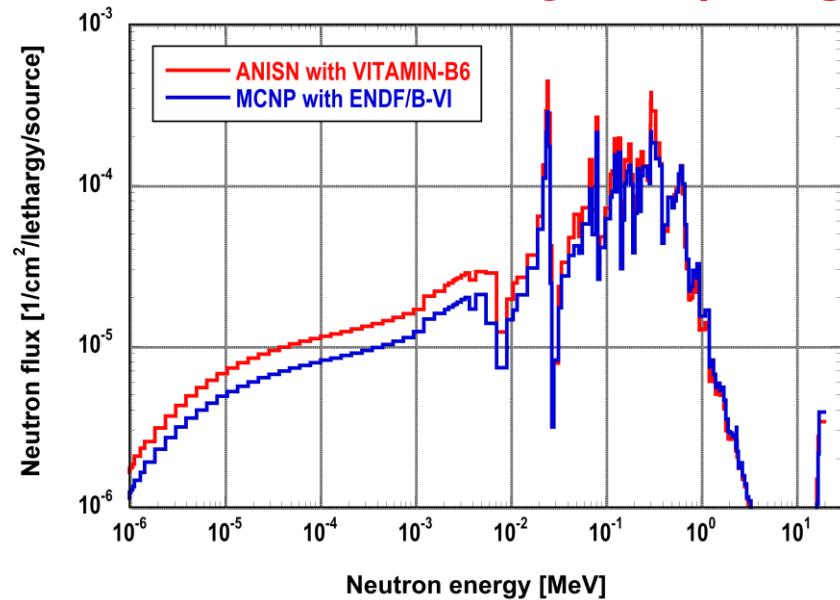
We reviewed the latest multigroup libraries, VITAMIN-B-6, -B7, VITENEA-J, VITJEFF31.BOLIB, MATJEFF31.BOLIB, V7-200N47G in SCALE6, MATXS file in ADS-2.0, HILO2k and MATXSLIB-J33, -J40 through the simple benchmark calculation test.

Weighting function and background cross section data in multigroup libraries –(2)

JAEA/FNS [11]



Calculation model



Neutron spectra at 40cm from center

The self-shielding correction in **VITAMIN-B6, -B7, VITENEA-J, VITJEFF31.BOLIB, V7-200N47G** in **SCALE6, MATJEFF31.BOLIB, MTXS** file in **ADS-2.0** and **HILO2k** (except Fe) is not always sufficient, calculations with which can cause unexpected results.

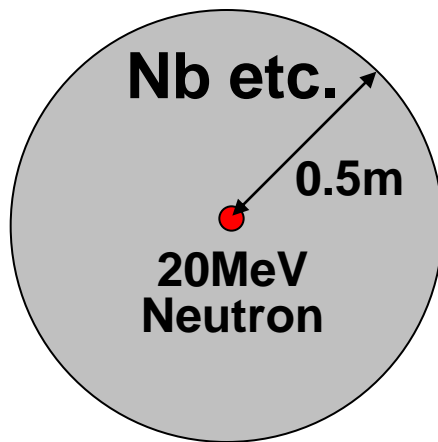
C. Konno et al., Prog. Nucl. Sci. Tech. 1 (2011), 32-35.

C. Konno et al., Prog. Nucl. Sci. Tech. 2 (2011), 341-345.

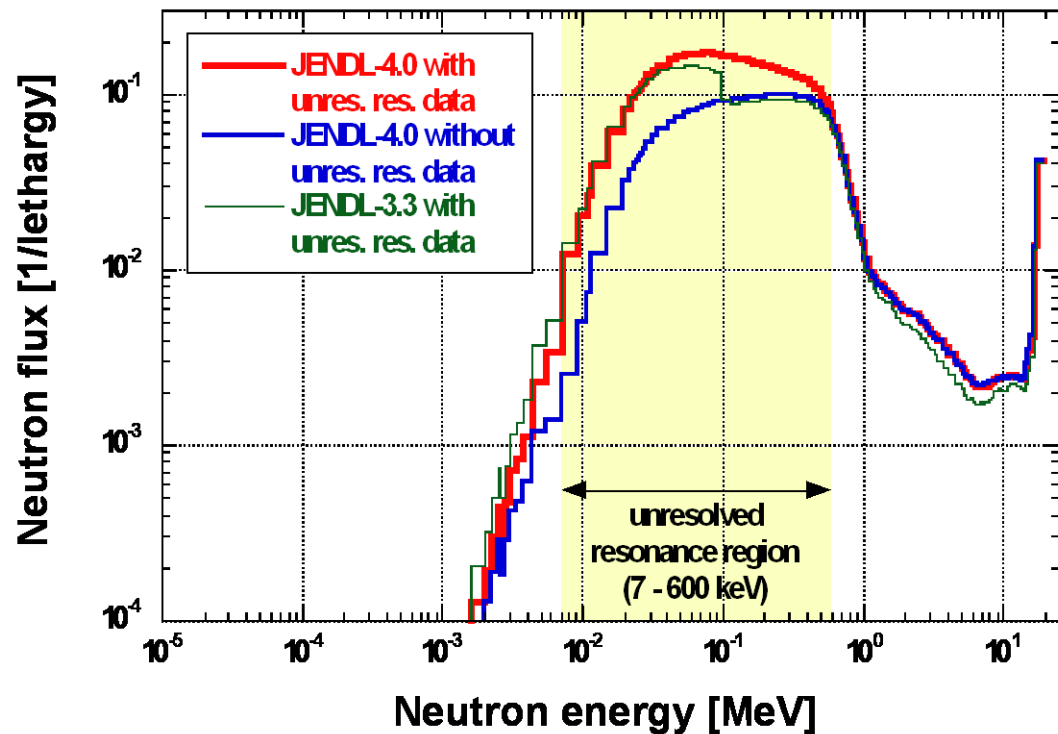
Self-shielding for unresolved resonance data -(1)

JAEA/FNS [12]

- We found out that **the self-shielding was too large around the upper energy of the unresolved resonance region** in JENDL-3.3, ENDF/B-VI.8 and JEFF-3.1.
- JENDL-4.0 tried to solve this problem by increasing the upper energy of the unresolved resonance region.



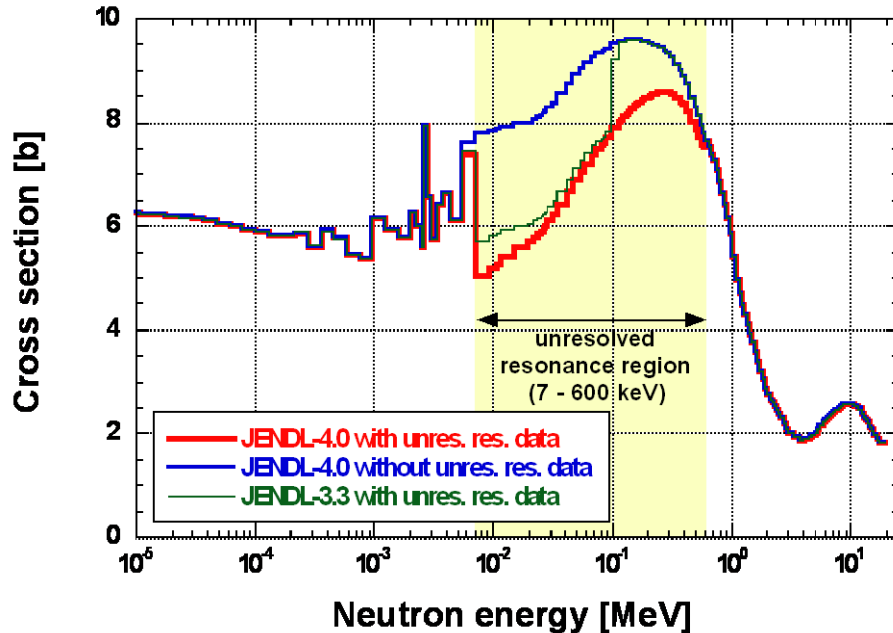
Calculation model



Leakage neutron spectra (MCNP)

Self-shielding for unresolved resonance data -(2)

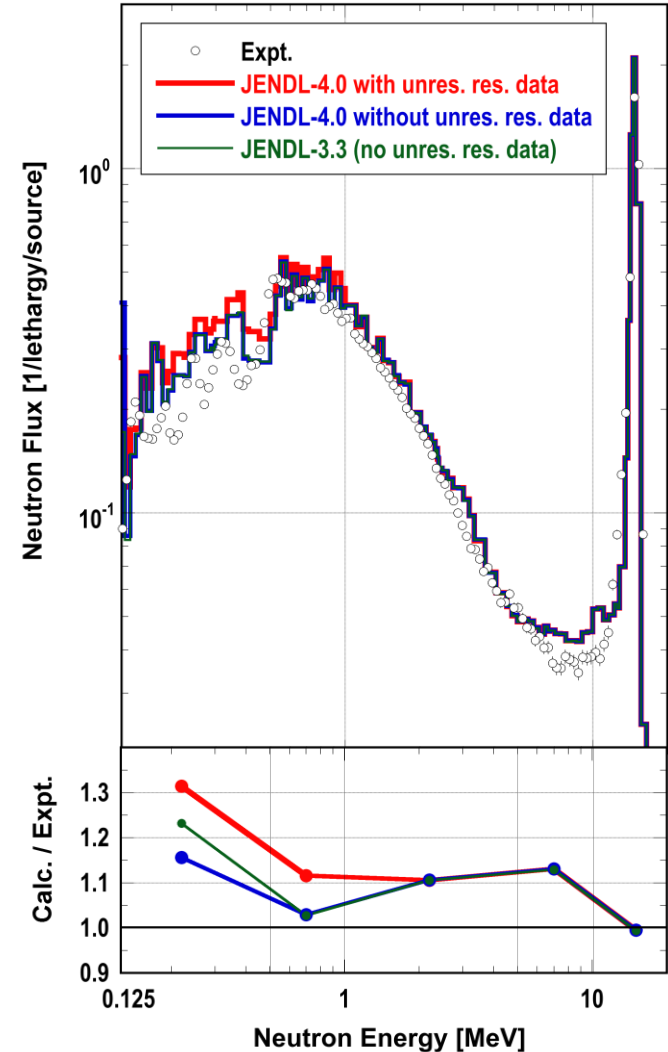
JAEA/FNS [13]



Elastic scattering cross section of ^{93}Nb
(deduced from MATXS files with TRANSX)

Are the unresolved resonance data adequate from the view of self-shielding?

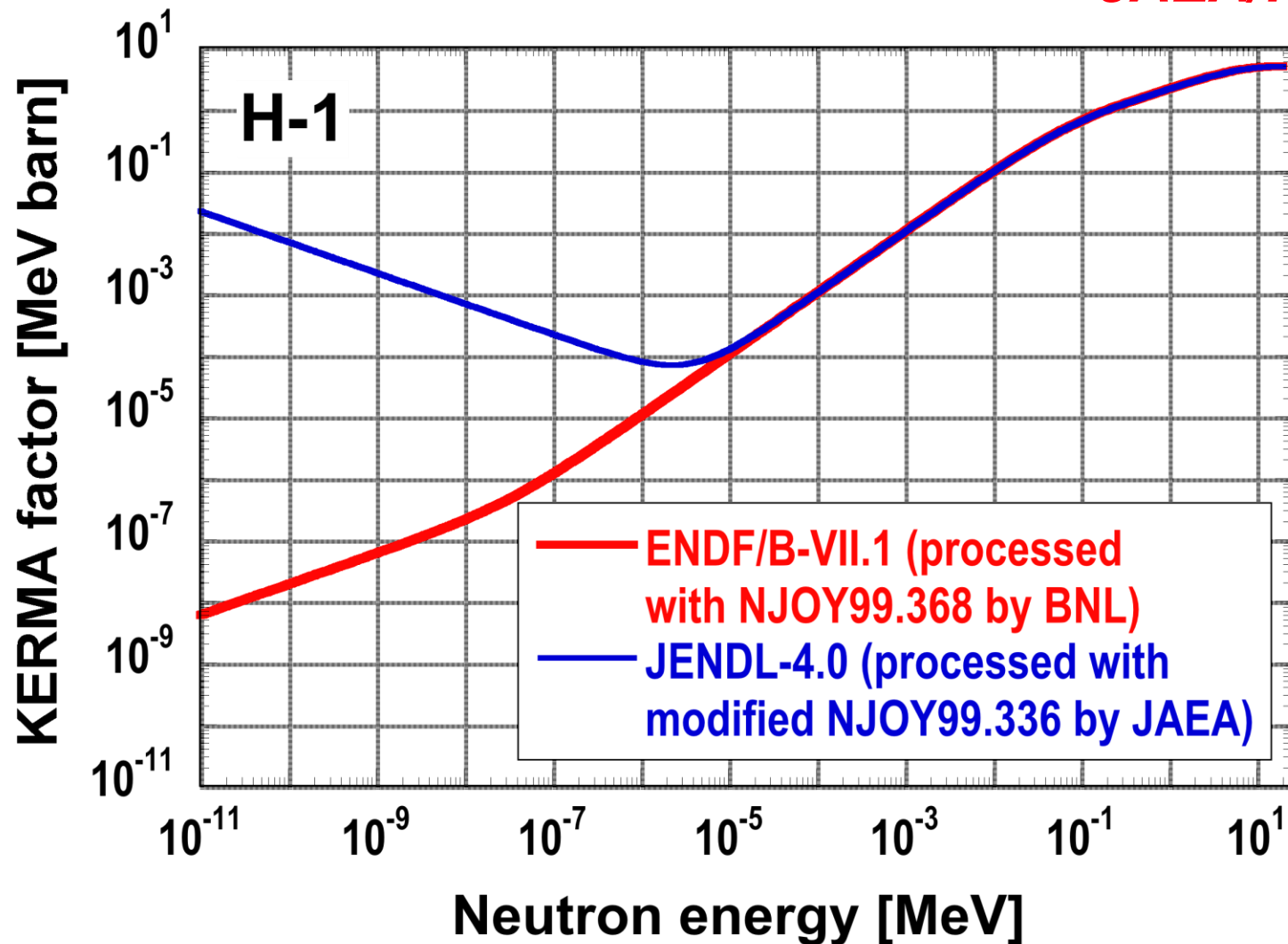
Submitted to Prog. Nucl. Sci. Tech.



Leakage neutron spectra in Mn TOF experiment at OKTAVIAN

KERMA factor -(1)

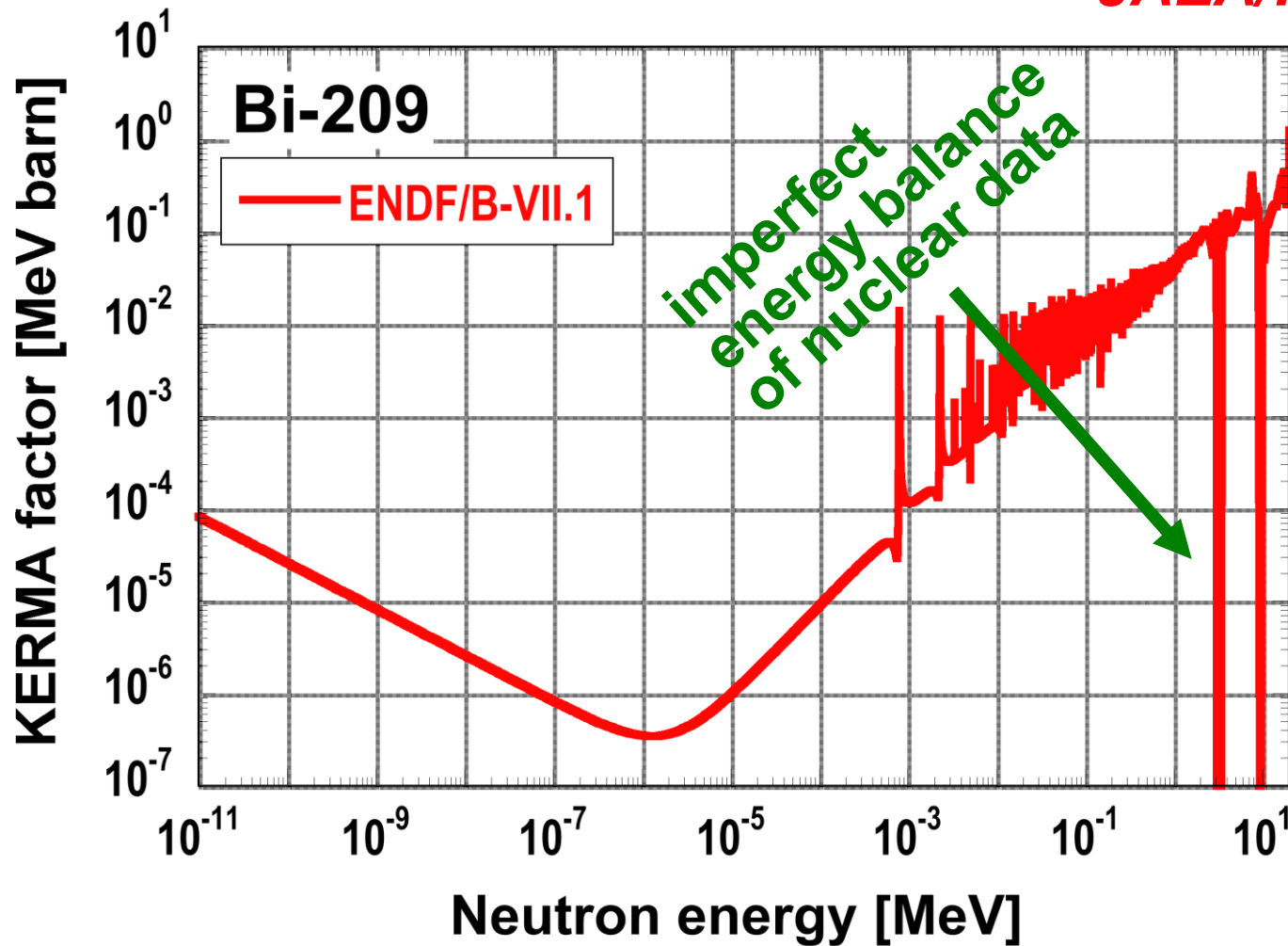
JAEA/FNS [14]



NJOY has a bug in KERMA calculation of ^1H .

KERMA factor –(2)

JAEA/FNS [15]



A lot of nuclear data do not keep energy balance.
Neutron KERMA factors in ACE files are not always correct.

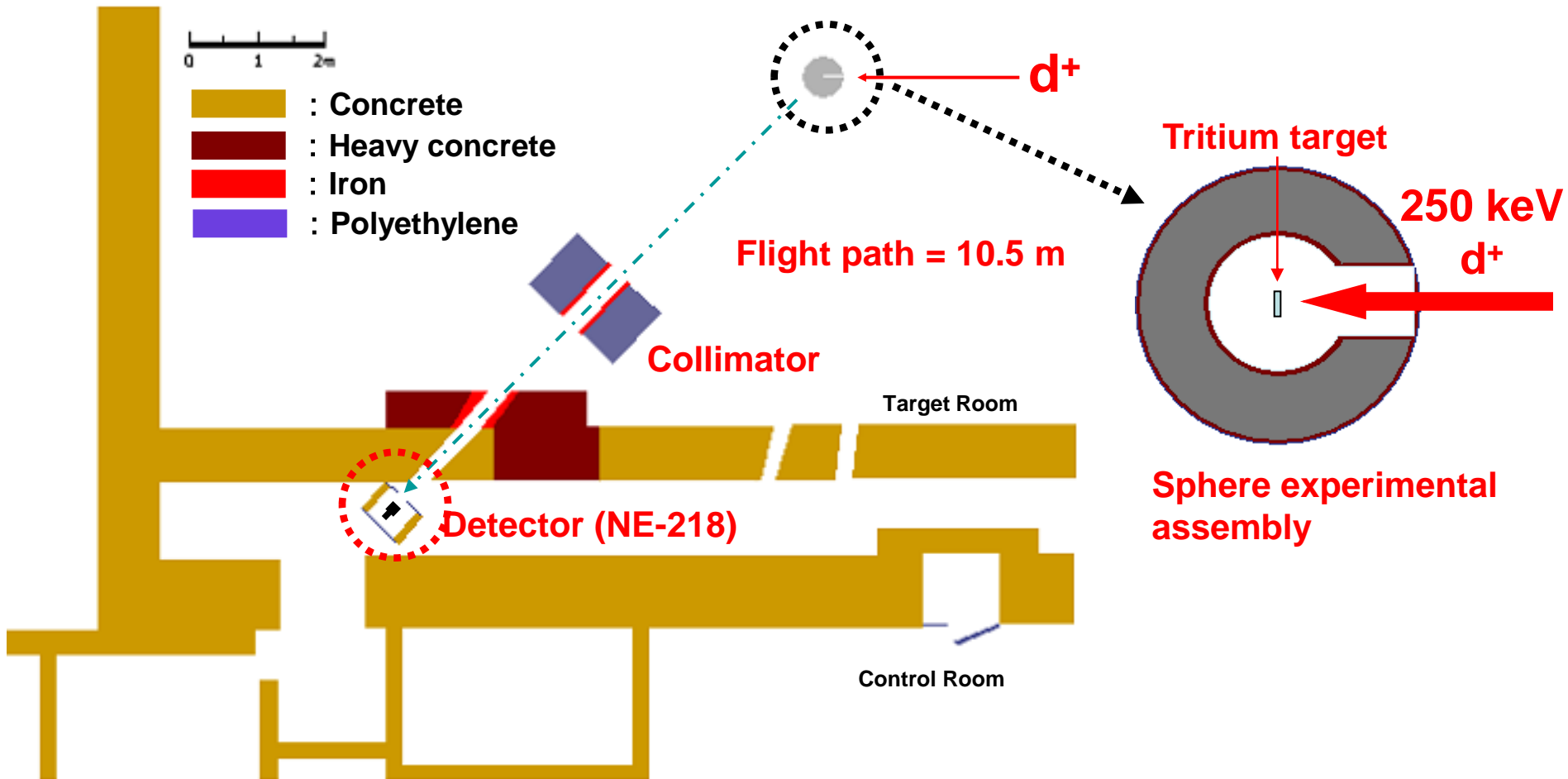
Conclusion and Requests

JAEA/FNS [16]

- ❑ We introduced our **recent studies related to nuclear data.**
 - Latest in-situ experiment on titanium
 - Weighting function and background cross section data in multigroup libraries
 - Self-shielding for unresolved resonance data
 - KERMA factors
- ❑ We request the following issues,
 - **Adequate codes for nuclear data processing and files processed from nuclear data libraries for nuclear analysis codes**
 - **Unresolved resonance data adequate from the view of self-shielding**
 - **Nuclear data libraries with complete energy balance including secondary gamma data**

OKTAVIAN Mn Sphere TOF Experiment –(1)

JAEA/FNS [17]



Insufficient self-shielding correction due to weighting function

JAEA/FNS [18]

Why does the weighting flux lead the inadequate self-shielding correction?

$$S_{ltg}^{PN} = \frac{\int_g S_t(E) W_\ell(E) dE}{\int_g W_\ell(E) dE} \quad \sigma_{lg \leftarrow g'}^{PN} = \frac{\int_{g'} dE' \int_g \sigma_\ell(E' \rightarrow E) W_\ell(E') dE}{\int_{g'} W_\ell(E') dE'}$$

$$S_{lg \rightarrow g}^{SN} = S_{lg \rightarrow g}^{PN} \quad \text{for } g \neq g'$$

$$\sigma_{lg \leftarrow g}^{SN} = \sigma_{lg \leftarrow g}^{PN} - (\sigma_{ltg}^{PN} - \sigma_{0tg}^{PN}) - \Delta_g^N$$

$$\sigma_g^{SN} = \sigma_{0tg}^{PN} - \Delta_g^N$$

0 : Consistent-P Approximation

Inadequate cross section

Insufficient self-shielding correction

= 0 for $\ell \geq 1$: VITAMIN-B6 [$W_{\ell \neq 1}(E) = W_0(E)$]
 $\neq 0$ for $\ell \geq 1$: $W_{\ell \neq 1}(E) \neq W_0(E)$

≈ 0 for $\ell \geq 1$: at no resonance regions
(weighting flux : a smooth shape)