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Benchmarking of the JEF2.2 based EJ2_XMAS neutron cross-section library

R.C.L. van der Stad

H. Gruppelaar

J.L. Kloosterman

Y. Wang

Netherlands Energy Research Foundation ECN

Nuclear Analysis Department

P.O. Box 1, NL-1755 ZG Petten, The Netherlands

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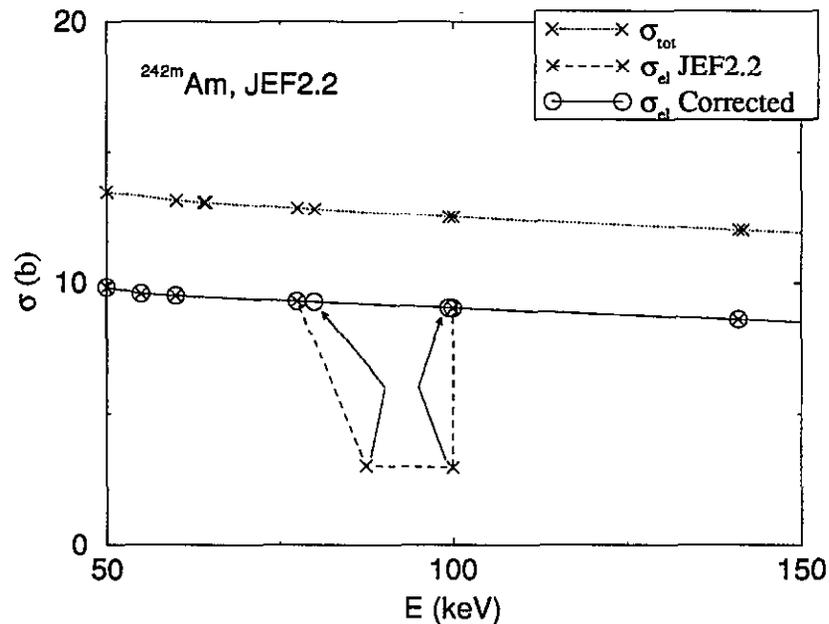
- The EJ2_XMAS library
 - Specifications
 - Remarks about JEF2.2 evaluation
- Benchmarking
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Specifications EJ2_XMAS library

- JEF2.2 based in XMAS group structure
- NJOY89.63+/NSLINK
- AMPX-M library for SCALE4.1 code system.
- Shielding: combined Nordheim/Bondarenko method for several T's and σ_0 's if parameters given on JEF2.2.
 - Actinides: Nordheim/Bondarenko (Reich-Moore parameters converted to Breit-Wigner).
 - Structural materials with RM: Bondarenko. Other structural materials: Nordheim/Bondarenko.
 - Fission products: 20 most important Nordheim/Bondarenko. Next 20 important FP's: Bondarenko.
 - Light and heavy nuclides: Nordheim/Bondarenko.
- about 300 nuclides (6 nuclides could be processed).
- all GENDF's stored for conversion to other formats.
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Remarks about JEF2.2 evaluation

1. $^{242m}_{95}\text{Am}$: Two wrong values of σ_{el} and discontinuity at 100 keV. Not all points of partials are contained in σ_t .



2. $^{230}_{92}\text{Th}$: Threshold between resolved and unresolved background (MF3) should be moved to upper limit of resolved resonance region (from 248 eV to 251 eV; 248 eV is the energy E_0 of highest resolved resonance).
3. ^4_2He : AWR is given in amu, should be given in amu divided by the neutron mass in amu (1.0085).
4. $^{209}_{83}\text{Bi}$, $^{239}_{93}\text{Np}$, $^{242g}_{95}\text{Am}$: contains discontinuities in σ_t .
5. $^{233}_{92}\text{U}$: large background in resolved resonance range (large self-shielding calculated with Nordheim method is incorrect).
6. Large background in unresolved resonance range for following nuclides $^{238}_{93}\text{Np}$, $^{244}_{96}\text{Cm}$, $^{249}_{97}\text{Bk}$, $^{249}_{98}\text{Cf}$, $^{253}_{98}\text{Cf}$, $^{253}_{99}\text{Es}$, $^{103}_{45}\text{Rh}$, $^{145}_{60}\text{Nd}$, $^{101}_{44}\text{Ru}$, $^{127}_{51}\text{I}$, $^{141}_{59}\text{Pr}$, $^{139}_{57}\text{La}$, $^{146}_{60}\text{Nd}$, $^{108}_{46}\text{Pd}$, $^{107}_{46}\text{Pd}$, $^{144}_{60}\text{Nd}$, ... (large self-shielding incorrect).
7. $^{58}_{27}\text{Co}$, $^{59}_{28}\text{Ni}$, $^{64}_{30}\text{Zn}$, $^{112}_{50}\text{Sn}$ evaluations not complete (MF5)

Benchmark 'Rowlands' definition

Parameters of PWR Pin Cell Benchmark Model.

Region Material Outer radius (cm)	Fuel UO ₂ 0.4	Clad Zirconium 0.45	Moderator Water 0.6770275
CASE 1			
Temperature (K)	293		293
Nuclide densities ^a	U235	7.0803E-4	Zr 4.3241E-2
	U238	2.2604E-2	H 6.6988E-2
	O	4.6624E-2	O 3.3341E-2
CASE 2			
Temperature (K)	293		293
Nuclide densities ^a	U235	7.0803E-4	Zr 4.3241E-2
	U238	2.2604E-2	H 4.6892E-2
	O	4.6624E-2	O 2.3390E-2
CASE 3			
Temperature (K)	900		550
Nuclide densities ^a	U235	7.0803E-4	Zr 4.3241E-2
	U238	2.2604E-2	H 4.6892E-2
	O	4.6624E-2	O 2.3390E-2

^aNuclide densities are given in barn⁻¹ cm⁻¹

Benchmark 'Rowlands' results
 k_{∞} values from CEN, AEA and ECN

Case	Fuel temperature (K)	Moderator density factor	CEN APOLLO-2 XMAS ^a	ECN PASC IJ1-XMAS ^b	AEA LWR-WIMS XMAS ^c	ECN PASC EJ2-XMAS ^c	ECN MCNP4 JEF2.2 ^d
1	293	1.0	1.37905	1.38353	1.38636	1.38454	1.38952
2	293	0.7	1.32548	1.33252	1.33567	1.33431	
3	900	0.7	1.29185	1.29767	1.30322	1.30147	
Differences(pcm) †							
2 - 1			-5357	-5101	-5069	-5023	
3 - 2			-33363	-3485	-3245	-3284	
3 - 1			-8720	-8586	-8314	-8307	
Temperature coefficient $\Delta\rho/\Delta T$ 1 - 3 (mN/°C)			-8.1	-7.9	-7.6	-7.6	

† differences (pcm) are defined as a direct subtraction of k_{∞} values multiplied by 10^5

^a based on JEF2.0)

^b based on JEF1.1 (IJ1-XMAS library processed by IRI, Delft)

^c based on JEF2.2

^d MCNP library based on JEF2.2

Difference discrete ordinate method (ECN PASC) and Monte Carlo (ECN MCNP) possibly partly (about 0.2 % in k_{∞}) due to a too coarse group structure in 4 lowest energy resonances of ^{238}U in the XMAS group structure.

See paper W. de Kruijf, 'Detailed Resonance Absorption Calculations with the Monte Carlo Code MCNP and a Collision Probability Version of the Slowing Down Code ROLAIDS', to be submitted for the 'International Conference on Reactor Physics and Reactor Computations', Tel Aviv, Israel (January 1994).

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Benchmark 'Rowlands' results

Neutron balance breakdown for ECN and AEA results.

	Absorption			Fission			Neutron yields		
	ECN ij1-xmas	AEA xmas	ECN ej2-xmas	ECN ij1-xmas	AEA xmas	ECN ej2-xmas	ECN ij1-xmas	AEA xmas	ECN ej2-xmas
Case 1									
H	5.606E-2	5.654E-2	5.626E-2						
O	3.563E-3	4.807E-3	4.592E-3						
Zr	7.766E-3	7.261E-3	7.309E-3						
U235	6.442E-1	6.447E-1	6.440E-1	5.344E-1	5.359E-1	5.358E-1	1.303E+0	1.307E+0	1.307E+0
U238	2.884E-1	2.867E-1	2.878E-1	2.861E-2	2.832E-2	2.773E-2	8.038E-2	7.913E-2	7.740E-2
Total	1.0	1.0	1.0	5.630E-1	5.642E-1	5.636E-1	1.383E+0	1.386E+0	1.384E+0
Case 2									
H	3.620E-2	3.658E-2	3.638E-2						
O	3.642E-3	4.901E-3	4.681E-3						
Zr	9.276E-3	8.585E-3	8.652E-3						
U235	6.163E-1	6.167E-1	6.163E-1	5.058E-1	5.077E-1	5.079E-1	1.234E+0	1.239E+0	1.239E+0
U238	3.346E-1	3.332E-1	3.340E-1	3.522E-2	3.481E-2	3.415E-2	9.881E-2	9.713E-2	9.516E-2
Total	1.0	1.0	1.0	5.411E-1	5.425E-1	5.420E-1	1.332E+0	1.336E+0	1.334E+0
Case 3									
H	3.494E-2	3.539E-2	3.510E-2						
O	3.642E-3	4.900E-3	4.681E-3						
Zr	9.151E-3	8.471E-3	8.533E-3						
U235	6.013E-1	6.012E-1	6.011E-1	4.915E-1	4.943E-1	4.944E-1	1.199E+0	1.206E+0	1.206E+0
U238	3.510E-1	3.501E-1	3.506E-1	3.523E-2	3.481E-2	3.415E-2	9.882E-2	9.713E-2	9.517E-2
Total	1.0	1.0	1.0	5.268E-1	5.291E-1	5.285E-1	1.298E+0	1.303E+0	1.301E+0

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Benchmark 'Rowlands' results

Neutron balance breakdown in 6 energy groups [†] for ECN results
Case 1

Case 1. Absorption						
Group	H	O16	Zr	U235	U238	Total
1	1.1269E-05	4.5643E-03	2.2853E-04	2.8827E-03	3.1528E-02	3.9214E-02
2	1.2378E-05	1.2556E-08	1.6858E-04	2.4854E-03	6.6992E-03	9.3656E-03
3	3.5072E-05	2.3022E-08	3.4660E-04	2.5801E-03	1.1732E-02	1.4694E-02
4	1.8589E-03	9.8030E-07	3.5882E-03	6.3682E-02	1.5563E-01	2.2476E-01
5	9.3332E-03	4.9128E-06	5.4354E-04	9.2550E-02	1.8113E-02	1.2054E-01
6	4.5014E-02	2.1728E-05	2.4337E-03	4.7980E-01	6.4151E-02	5.9142E-01
Total	5.6265E-02	4.5919E-03	7.3091E-03	6.4398E-01	2.8785E-01	1.0000E+00
Case 1. Fission						
1				2.7521E-03	2.7689E-02	3.0441E-02
2				2.1186E-03	3.9766E-05	2.1584E-03
3				1.9543E-03	4.8010E-06	1.9591E-03
4				4.1035E-02	1.0388E-05	4.1046E-02
5				7.7064E-02	7.2398E-08	7.7064E-02
6				4.1090E-01	2.7788E-07	4.1090E-01
Total				5.3583E-01	2.7744E-02	5.6357E-01
Case 1. Neutron yields						
1				7.4266E-03	7.7258E-02	8.4684E-02
2				5.2434E-03	1.0083E-04	5.3443E-03
3				4.7708E-03	1.1959E-05	4.7827E-03
4				9.9850E-02	2.5859E-05	9.9876E-02
5				1.8796E-01	1.8022E-07	1.8796E-01
6				1.0018E+00	6.9172E-07	1.0018E+00
Total				1.3070E+00	7.7397E-02	1.3844E+00

[†] upper boundaries (eV): 1.96403E+07, 8.20850E+05, 1.11090E+05, 9.11882E+03, 4.00000E+00, 1.40000E-01

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Benchmark 'Rowlands' results

Neutron balance breakdown in 6 energy groups [†] for ECN results
Case 2

Case 2. Absorption						
Group	H	O16	Zr	U235	U238	Total
1	9.9961E-06	4.6595E-03	2.8573E-04	3.5933E-03	3.8940E-02	4.7488E-02
2	1.2061E-05	1.4652E-08	2.3303E-04	3.4172E-03	9.2050E-03	1.2867E-02
3	3.4271E-05	2.6993E-08	4.8366E-04	3.6010E-03	1.6311E-02	2.0430E-02
4	1.6884E-03	1.0654E-06	4.8956E-03	8.3052E-02	1.9231E-01	2.8195E-01
5	7.7627E-03	4.8875E-06	6.4667E-04	1.0911E-01	2.1701E-02	1.3923E-01
6	2.6873E-02	1.5445E-05	2.1077E-03	4.1348E-01	5.5556E-02	4.9803E-01
Total	3.6380E-02	4.6809E-03	8.6524E-03	6.1626E-01	3.3403E-01	1.0000E+00
Case 2. Fission						
1				3.4290E-03	3.4085E-02	3.7514E-02
2				2.9121E-03	5.4010E-05	2.9661E-03
3				2.7283E-03	6.6943E-06	2.7350E-03
4				5.3767E-02	1.3884E-05	5.3781E-02
5				9.0809E-02	8.6453E-08	9.0809E-02
6				3.5423E-01	2.4066E-07	3.5423E-01
Total				5.0788E-01	3.4160E-02	5.4204E-01
Case 2. Neutron yields						
1				9.2396E-03	9.4968E-02	1.0421E-01
2				7.2064E-03	1.3695E-04	7.3434E-03
3				6.6602E-03	1.6675E-05	6.6768E-03
4				1.3083E-01	3.4564E-05	1.3086E-01
5				2.2148E-01	2.1520E-07	2.2148E-01
6				8.6363E-01	5.9909E-07	8.6363E-01
Total				1.2390E+00	9.5157E-02	1.3342E+00

[†] upper boundaries (eV): 1.96403E+07, 8.20850E+05, 1.11090E+05, 9.11882E+03, 4.00000E+00, 1.40000E-01

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Benchmark 'Rowlands' results

Neutron balance breakdown in 6 energy groups [†] for ECN results Case 3.

Case 3. Absorption						
Group	H	O16	Zr	U235	U238	Total
1	9.9958E-06	4.6595E-03	2.8571E-04	3.5935E-03	3.8942E-02	4.7491E-02
2	1.2060E-05	1.4652E-08	2.3303E-04	3.4171E-03	9.2269E-03	1.2889E-02
3	3.4259E-05	2.6983E-08	4.8345E-04	3.5997E-03	1.6633E-02	2.0750E-02
4	1.6506E-03	1.0400E-06	4.8412E-03	8.2350E-02	2.0892E-01	2.9776E-01
5	1.0469E-02	6.5741E-06	8.7102E-04	1.5485E-01	2.8170E-02	1.9437E-01
6	2.2926E-02	1.3434E-05	1.8185E-03	3.5329E-01	4.8687E-02	4.2674E-01
Total	3.5102E-02	4.6806E-03	8.5329E-03	6.0111E-01	3.5057E-01	1.0000E+00
Case 3. Fission						
1				3.4293E-03	3.4088E-02	3.7517E-02
2				2.9123E-03	5.4012E-05	2.9663E-03
3				2.7274E-03	6.6902E-06	2.7341E-03
4				5.3221E-02	1.4858E-05	5.3236E-02
5				1.2924E-01	1.1425E-07	1.2924E-01
6				3.0284E-01	2.1071E-07	3.0284E-01
Total				4.9437E-01	3.4164E-02	5.2853E-01
Case 3. Neutron yields						
1				9.2401E-03	9.4977E-02	1.0422E-01
2				7.2066E-03	1.3696E-04	7.3435E-03
3				6.6579E-03	1.6664E-05	6.6746E-03
4				1.2951E-01	3.6986E-05	1.2954E-01
5				3.1522E-01	2.8441E-07	3.1522E-01
6				7.3840E-01	5.2450E-07	7.3840E-01
Total				1.2062E+00	9.5168E-02	1.3014E+00

[†] upper boundaries (eV): 1.96403E+07, 8.20850E+05, 1.11090E+05, 9.11882E+03, 4.00000E+00, 1.40000E-01

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Benchmark ORNL results

k_{∞} values from ECN, by PASC with IJ1-XMAS, EJ2-XMAS and EJ2+O16(IJ1) libraries

Assembly		ORNL-1	ORNL-2	ORNL-3	ORNL-4	ORNL-10	Average
		H/U235 1378	H/U235 1177	H/U235 1033	H/U235 972	H/U235 1835	
		K_{eff}	K_{eff}	K_{eff}	K_{eff}	K_{eff}	K_{eff}
experimental values		1.00026	0.99975	0.99994	0.99924	1.00031	0.9999
ECN PASC-4	IJ1-XMAS ^a	1.00130	1.00091	0.99777	0.99919	1.00023	0.99988
	EJ2-XMAS ^b	0.99832	0.99800	0.99496	0.99641	0.99801	0.99714
	EJ2+O16(IJ1) ^c	1.00040	1.00010	0.997052	0.99851	0.999435	0.99910

^abased on JEF1.1 (IJ1-XMAS library processed by IRI, Delft)

^bbased on JEF2.2 (EJ2-XMAS library processed by ECN)

^ccross sections of O-16 taking from IJ1-XMAS library and the others from the EJ2-XMAS library.

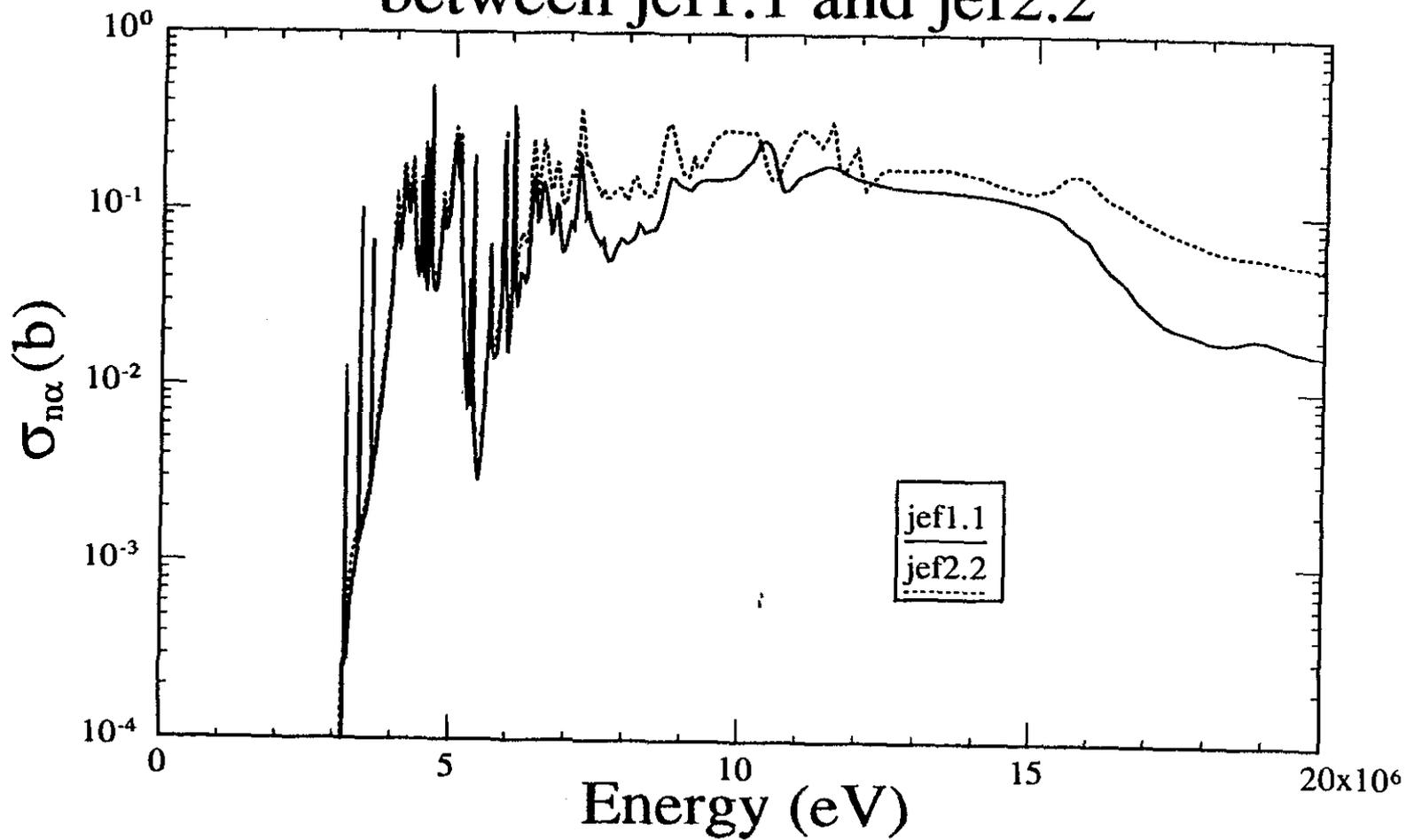
Benchmark ORNL results
 Neutron balance breakdown (ORNL-1) under k_{eff} calculation

	IJ1-XMAS			EJ2-XMAS			EJ2+O16(IJ1)		
	Absorption	Fission	Neutron Yield	Absorption	Fission	Neutron Yield	Absorption	Fission	Neutron Yield
H	3.311E-1			3.305E-1			3.312E-1		
N14	5.534E-4			5.524E-3			5.536E-3		
O16	2.875E-3			3.732E-3			2.835E-3		
U234	1.036E-3	9.734E-6	2.476E-5	1.037E-3	9.746E-6	2.477E-5	1.036E-3	9.757E-6	2.481E-5
U235	4.835E-1	4.109E-1	1.00125	4.821E-1	4.095E-1	0.998270	4.831E-1	4.103E-1	1.00036
U236	4.154E-5	1.165E-6	2.962E-6	4.143E-5	1.164E-6	2.958E-6	4.153E-5	1.166E-6	2.964E-6
U238	6.032E-4	7.506E-6	2.118E-5	6.006E-4	7.261E-6	2.036E-5	6.020E-4	7.273E-6	2.041E-5
Total	0.819709	0.410918	1.001299	0.823535	0.409518	0.998318	0.824351	0.4103182	1.000408

The contribution to the total absorption of ^{16}O is 0.1 % higher with JEF2.2 data than with JEF1.1
 This leads to a 0.2 % lower k_{eff} with JEF2.2

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Figure 6: Comparison of $\sigma_{n\alpha}$ for ^{16}O between jef1.1 and jef2.2



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Benchmark TRX1: Preliminary results

	k_{eff}	ρ^{28}	δ^{25}	δ^{28}	C^*
experiment	1.000	1.320±0.021	0.0987±0.001	0.0946±0.0041	0.797±0.008
ECN/PASC-4 with EJ2-XMAS ^a	0.9973	1.3012	0.0939	0.0941	0.7863
ECN/PASC-4 with IJ1-XMAS ^b	1.0001	1.3134	0.0944	0.0959	0.7864
ECN/PASC-3 ^c with ECNJEF1.1 ^d	0.9968	1.3266	0.0950	0.0962	0.7913
IKE ^e JEF-1	0.9960	1.3569	0.0992	0.1001	0.7984
ENDF-311 ^f ENDF/B-IV	0.9876	1.3820	0.0994	0.0955	0.8060
ENDF-311 ENDF/B-V	0.9961	1.3590	0.1003	0.0983	0.7980

^abased on JEF2.2 (EJ2-XMAS library processed by ECN)

^bcross sections of O-16 taking from IJ1-XMAS library, the others taking from EJ2-XMAS library

^bbased on JEF1.1 (IJ1-XMAS library processed by IRI, Delft)

^cJ. Li et al., ECN report, NFA-LWR-92-01

^dbased on JEF1.1, 219 group library

^eW. Bernnat et al., IKE 6-157

^fENDF-311, Benchmark data testing of ENDF/B-V, BNL-NCS-31531

ρ^{28} = ratio of epithermal-to-thermal ²³⁸U captures

δ^{25} = ratio of epithermal-to-thermal ²³⁵U fissions

δ^{28} = ratio of ²³⁸U fissions to ²³⁵U fissions

C^* = ratio of ²³⁸U captures to ²³⁵U fissions

Benchmark TRX2: Preliminary results

	k_{eff}	ρ^{28}	δ^{25}	δ^{28}	C^*
experiment	1.000	0.837 ± 0.016	0.0614 ± 0.0008	0.0693 ± 0.0035	0.647 ± 0.006
ECN/PASC-4 with EJ2-XMAS ^a	0.9918	0.8180	0.0580	0.0671	0.6376
ECN/PASC-4 with IJ1-XMAS ^b	0.9948	0.8255	0.0583	0.0685	0.6366
ECN/PASC-3 ^c with ECNJEF1.1 ^d	0.9923	0.833	0.0588	0.0688	0.6396
IKE ^e JEF-1	0.9973	0.8374	0.0608	0.0714	0.6393
ENDF-311 ^f ENDF/B-IV	0.9935	0.863	0.0609	0.0676	0.6470
ENDF-311 ENDF/B-V	0.9984	0.846	0.0614	0.0699	0.6420

Benchmark TRX3: Preliminary results

	k_{eff}	ρ^{28}	δ^{25}	δ^{28}	C^*
experiment	1.000	3.03 ± 0.05	0.231 ± 0.003	0.167 ± 0.008	1.255 ± 0.011
ECN/PASC-4 with EJ2-XMAS ^a	0.99998	2.9729	0.2264	0.1758	1.2360
ECN/PASC-4 with IJ1-XMAS ^b	1.0028	3.005	0.2278	0.1789	1.2409
ECN/PASC-3 ^c with ECNJEF1.1 ^d	0.9967	3.0486	0.2272	0.1783	1.2546
IKE ^e JEF-1	1.001	3.013	0.236	0.1725	1.2402
ENDF-311 ^f ENDF/B-IV	0.9908	3.07	0.235	0.174	1.252

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Benchmark TRX4: Preliminary results

	k_{eff}	ρ^{28}	δ^{25}	δ^{28}	C^*
experiment	1.000	0.481 ± 0.011	0.0358 ± 0.005	0.0482 ± 0.002	0.531 ± 0.004
ECN/PASC-4 with EJ2-XMAS ^a	0.9957	0.4764	0.0337	0.0472	0.5271
ECN/PASC-4 with IJ1-XMAS ^b	0.9987	0.4805	0.0338	0.0484	0.5254
ECN/PASC-3 ^c with ECNJEF1.1 ^d	0.9921	0.4891	0.0345	0.0491	0.5288
IKE ^e JEF-1	0.9979	0.4760	0.0344	0.0473	0.5244
ENDF-311 ^f ENDF/B-IV	0.9937	0.491	0.0345	0.0467	0.5270

Conclusions

- The EJ2_XMAS library leads to satisfactory results. The results of the benchmarks are in general in good agreement with the rest of the world or the experimental results.
- The k_{eff} results of the benchmark results are in general in good agreement with the rest of the world or the experimental results but tend to be a little low.
- Special attention has to be paid to the influence of ^{16}O on the results.
- Special attention has to be paid to the discrepancy between discrete ordinate results and MCNP results.