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STATUS OF ^{239}Pu CROSS-SECTIONS EVALUATION
IN THE UNRESOLVED REGION

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METHODE OF EVALUATION.

The evaluation is performed by using the HAUSER-FESHBACH statistical code FISINGA. Starting from the average parameters obtained from a statistical analysis of the Reich-Moore resonance parameters of the energy range thermal to 1 kev, a preliminary calculation gives the statistical parameters for the best representation of the average experimental cross-sections in the energy range 1 kev to 30 kev. A single set of S_0 , S_1 , average capture width for s and p wave neutrons, fission barrier heights for the 0^+ , 1^+ , 1^- , 2^- channels is obtained, allowing a smooth representation of the cross-sections in the energy range considered. The parameters S_0 , S_1 and the fission barrier heights (mainly for the 1^+ fission channel) are then modified in order to reproduce the local fluctuations of the experimental total and fission cross-sections ; the corresponding channel widths are found in FISINGA output and are introduced in the evaluated data file. Finally, the consistency of the evaluation is checked by performing a NJOY-THEMIS calculation from the evaluated data file.

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PRELIMINARY RESULTS.

Preliminary calculations (1) were performed in the energy range 1 kev to 4 kev starting with the average parameters obtained in (2) ; for the fission, one partially open 1^+ channel, one fully open 0^+ channel and one partially open 0^+ channel were used. The effectif number, N_{ef} , of 0^+ fission channels was about 1.3. It was found that the fission cross-sections calculated by FISINGA and those calculated by NJOY-THEMIS from an equivalent set of parameters were not consistent in several energy points. The differences were mainly due to the fact that NJOY-THEMIS did not calculate the width fluctuation factor corrections for the 0^+ fission channels in the same way than FISINGA ; only integer values are allowed for the AMUF parameter in ENDF/B format. A N_{ef} of 1.3 is not taken into account in the NJOY-THEMIS calculations.

NEW RESULTS.

The recent statistical analysis (3) of the Reich-Moore multilevel resonance parameters obtained in the energy range 0 ev to 1 kev (3) has shown that the average 0^+ fission width is equal to about 2 800 mev, instead of 1 600 mev obtained in early analysis (2) in the energy range 0 to 200 ev. This new result is consistent with the one expected fully open 0^+ fission channel (contributing to about 1 500 mev in the total fission width) and with 2 or more not fully open fission channels (each contributing to several hundred mev in the total fission width). The corresponding N_{ef} is very close to the integer 2. Starting from this new basis, which should allow consistent calculation of the width fluctuation factors, much better agreement is obtained between FISINGA and NJOY-THEMIS calculations. The calculated cross-sections averaged over 1 kev intervals agree within 1% to 2%. Some problems remain in the energy points where the fission cross-section is particularly small, in relation with very small values of the 1^+ fission channel width and with the $(n, \gamma f)$ component. The final set of average parameters for the evaluated data file should be adjusted by using NJOY-THEMIS calculations.

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Table I and II give the current values of the fission and capture cross-sections calculated by NJOY-THEMIS in the energy range 1 kev to 4 kev.

REFERENCES.

1. H. DERRIEN,
Preliminary evaluation of ^{239}Pu cross-sections in thermal to 4 kev energy range for testing purpose ; available in ENDF/B-V format at NEA DB, SACLAY (1987).
2. H. DERRIEN,
Thèse série A n° 1172, ORSAY (1979).
3. H. DERRIEN and G. de SAUSSURE,
R-Matrix analysis of ^{239}Pu neutron cross-sections in the energy range 0 ev to 1000 ev (distributed at this meeting).

ENERGY(KEV)	FISSION(BARNS)		CAPTURE(BARNS)	
	A	B	C	D
1.025	7.007	7.064	4.123	4.108
1.075	4.614	4.615	5.481	5.494
1.125	7.225	7.294	4.392	4.377
1.175	4.860	4.910	3.721	3.691
1.225	3.972	4.010	3.148	3.117
1.275	5.334	5.332	6.111	6.023
1.325	7.543	7.575	3.002	2.907
1.375	6.892	6.919	2.887	2.880
1.425	3.673	3.673	4.202	4.196
1.475	4.326	4.362	3.291	3.262
1.525	2.797	2.756	3.080	3.093
1.575	2.184	2.188	4.128	4.105
1.625	4.616	4.616	4.285	4.283
1.675	3.440	3.444	3.996	3.982
1.725	2.922	2.954	4.318	4.347
1.775	3.785	3.785	3.755	3.755
1.825	6.068	5.987	4.553	4.500
1.875	4.165	4.198	3.092	3.063
1.925	2.207	2.210	3.591	3.562
1.975	1.844	1.850	4.330	4.296
2.050	1.910	1.911	3.551	3.520
2.150	3.149	3.151	3.700	3.674
2.250	2.621	2.619	3.242	3.213
2.350	3.456	3.457	3.692	3.665
2.450	4.269	4.270	2.856	2.831
2.550	2.647	2.646	3.171	3.134
2.650	3.159	3.161	3.746	3.710
2.750	4.200	4.180	2.392	2.373
2.850	4.147	4.126	2.455	2.434
2.950	3.333	3.333	4.004	3.961
3.050	3.034	3.020	2.404	2.371
3.150	4.943	4.908	2.269	2.254
3.250	3.959	3.932	2.626	2.599
3.350	1.717	1.721	2.561	2.509
3.450	2.208	2.210	2.918	2.862
3.550	2.197	2.196	2.300	2.252
3.650	2.396	2.394	3.242	3.180
3.750	3.117	3.072	1.988	1.963
3.850	3.580	3.534	2.448	2.420
3.950	2.965	2.943	2.712	2.666

TABLE I

²³⁹Pu Fission and capture cross-sections in
the energy range 1 kev to 4 kev

A and D calculated from NJOY-THEMIS

B calculated from selected experimental data

C calculated from FISINGA

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TABLE II

AVERAGE CROSS SECTIONS IN BARNS IN THE ENERGY INTERVAL 1 KEV-4 KEV

ENERGY RANGE KEV	A	B	C	D	E	D/A	F	G
1-2	4.487	4.474	4.274	3.957	3.974	0.882	0.834	0.927
2-3	3.285	3.289	3.173	3.252	3.281	0.990	1.028	1.108
3-4	2.993	3.015	2.918	2.508	2.547	0.838	0.820	0.895

A FISSION CALCULATED FROM NJOY-THEMIS
 B FISSION FROM SELECTED EXPERIMENTAL DATA
 RENORMALIZED ON ENDF/B-VI PROPOSED STANDARD
 C FISSION FROM WESTON 1984
 D CAPTURE CALCULATED FROM NJOY-THEMIS
 E CAPTURE CALCULATED FROM FISINGA
 D/A ALPHA CALCULATED FROM A AND D
 F ALPHA AVERAGED FROM 11 EXPERIMENTAL DATA
 G ALPHA FROM SOWERBY-KONSHIN 1971

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