

## CONCEPTUAL STUDY OF NEUTRON IRRADIATOR-DRIVEN BY ELECTRON ACCELERATOR

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### Abstract

Spallation neutron sources, though very effective in neutron production, are large, expensive and presently would involve certain difficulties in their operation (e.g., beam trips). Contrary, an electron driver, although much less effective in neutron production, is rather cheap and compact machine that, at the same time, might bring advantages in terms of reliability. Here we investigate the use of an external neutron source (irradiator) driven by an electron accelerator. A schematic layout and design of a compact neutron irradiator is proposed with its neutronics and safety being analysed and discussed in detail. The system is based on a spherical geometry with an electron beam interacting with the target-envelope. Neutrons are produced in the natural or enriched uranium by photo-nuclear reactions. The system is well sub-critical ( $k_{\text{eff}} < 0.8$ ) and uranium enrichment is below 20%. Neutron balance is optimised by using different geometry and material configurations. Our preliminary calculations show that variable (up to 10% thermal and/or up to 30% with energies higher than 1 MeV) neutron fluxes of a few  $10^{14} \text{ n s}^{-1} \text{ cm}^{-2}$  could be obtained for different irradiation purposes. An electron machine of ~8 MW power and 100 MeV incident energy should be sufficient to produce external neutrons to drive the system.

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## Introduction

Recently, a world-wide interest in photo-nuclear processes is experienced, what is motivated by a number of different applications such as shielding problems of medical or fundamental research accelerators, the need of new cost effective neutron sources, transmutation of nuclear waste either directly by photons [1] or by neutrons created from photo-nuclear reactions, [2] radioactive nuclear beam factories based on photo-fission, [3] etc. For a long time photo-nuclear processes were neglected by particle transport codes mainly due to the lack of the evaluated photo-nuclear data files. In 1996, in order to make up this backlog, IAEA started a co-ordinated research programme for compilation and evaluation of photo-nuclear data for applications. As a result of this effort, a photo-nuclear data file in ENDF format for 164 isotopes became available in 2000. [4] One of the first attempts to benchmark these new data files have been performed recently with well known Monte Carlo codes as MCNPX [5] and MCNP, [6] enhanced with a photo-nuclear capability independently by LANL (US) [7] and KFKI (Hungary). [8]

In this paper an unusual system to produce neutrons for irradiation purposes is described eliminating most of the potential difficulties encountered in conventional ADS. The accelerator is an electron machine, being cheaper, more reliable and more compact than high energy high power proton linac. Neutron are produced in a natural or enriched uranium target by photo-nuclear rather than spallation process. A schematic layout and design of a compact neutron irradiator is proposed with its neutronics and safety being analysed and discussed in detail. In all calculations we employ already benchmarked MCNP code enhanced with photo-nuclear capability [8] together with the recommended IAEA photo-nuclear data files. [4]

## Neutron yield and cost

Photo-nuclear reactions as  $(\gamma,n)$  and  $(\gamma,2n)$  can be induced in any material by specific gamma rays exciting the Giant Dipole Resonance (GDR) of the nuclei, while  $(\gamma,fiss)$  may occur only in the case of actinides. For example, in the case of  $^{238}\text{U}$  a maximum fission probability of 160 mb can be obtained for photons having energy around 15 MeV. Unfortunately, the most common way for producing high gamma fluxes in the GDR region is the bremsstrahlung process resulting from electrons passing through the matter. This process has a cross-section linear with energy above 20 MeV. The resulting bremsstrahlung spectrum is widely spread in the energy range from zero to the incident energy of electron, and only a small fraction of these photons are “useful” photons, i.e. lying in the GDR range of  $15\pm 5$  MeV. Therefore, the overall efficiency of neutron production is much lower than one might expect by having in mind the direct photo-nuclear process.

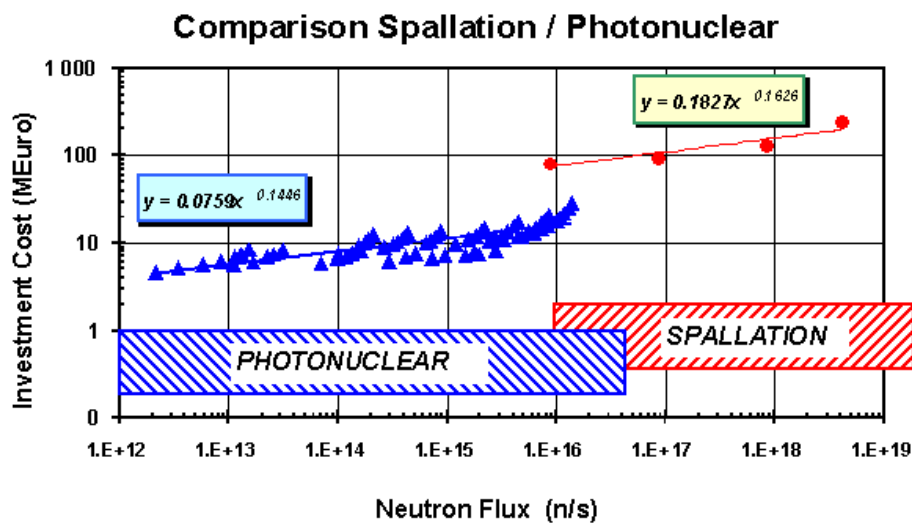
Let us take an example. The number of fissions per incident electron impinging on an infinite natural uranium target approximately follows a linear law with a threshold energy about 8.5 MeV [2,3]:

$$N_{[fiss/e-]} = 1.9 \times 10^{-4} (E_{[in MeV]} - 8.5).$$

In other words, an electron having an energy of 100 MeV will induce  $\sim 0.017$  fissions. The neutron production efficiency can then be estimated taking into account that each fission will release about  $\nu = 3.4$  prompt neutrons in addition to the contribution of other photo-nuclear reactions as  $(\gamma,n)$  and  $(\gamma,2n)$ , which contribute nearly the same number of neutrons as the photo-fission process. [2,3] The total number of neutrons produced for a 100 MeV electron is then  $\sim 0.11$  n/e. In this case the neutron cost is about 900 MeV. This is much larger than the neutron produced by the spallation process (e.g., a 1 GeV proton on lead target), where each proton can create about 30 neutrons. Here

the neutron cost is around 30 MeV, i.e. ~30 times cheaper than the photo-nuclear one. On the other hand, even the neutron cost is higher, the accelerator cost is much lower in the case of electron machine. Therefore, for the same neutron flux required, a higher electron intensity (and beam power) will be needed due to the lower efficiency. Thus, above a given neutron flux, the spallation will be preferred while for the lower fluxes, the photo-nuclear process will tend to be cheaper. This is illustrated in Figure 1, where for a given neutron flux, both an electron machine as well as a proton accelerator has been cost effectively estimated. Note that this is only machine cost, which does not include manpower or buildings (which again are certainly cheaper for the electron machine). In brief, for neutron source intensity higher than  $10^{17}$  n/s, the spallation process will start to appear more effective, while below this value the photo-nuclear process is favoured.

Figure 1. Spallation versus photo-nuclear process for neutron production [2]



### Modelling procedure and geometry considerations

A simplified model of the neutron irradiator has been created using a typical MCNP [6] geometry set-up in 3D. MCNP was also used to obtain the  $k_{eff}$  eigenvalues and neutron fluxes. Neutron production with electrons was modelled by the same MCNP code enhanced with photo-nuclear capability. [8] In all cases the recommended IAEA photo-nuclear data files have been used. [4] Both  $(\gamma,n)$ ,  $(\gamma,2n)$  and  $(\gamma,fiss)$  reactions were taken into account explicitly for all materials used in the problem and with a corresponding full secondary neutron transport.

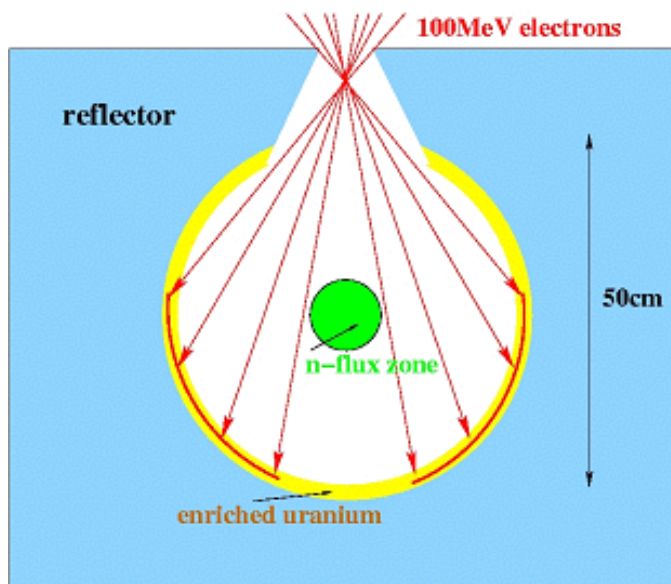
Below we present two different geometry configurations, although in both cases the neutron production target is a spherical uranium envelope. The major difference between them is that in one case electrons interact with the target from inside, while in the second case – from outside as explained in more detail below.

### Spherical geometry G1

A proposed electron target is 2 cm thick and made of enriched uranium ( $\sim 19\text{g/cm}^3$ ). Its total volume and mass is  $\sim 17\,000\text{ cm}^3$  and  $\sim 323\text{ kg}$  respectively. An electron beam is dispersed at the entrance of the system, so it can interact with nearly half of the actual surface of the inner uranium

envelope as shown in Figure 1. We choose 100 MeV electrons since neutron production is nearly linear as a function of the incident beam energy, i.e. neutron production is constant for a given beam power as discussed above. Our major observable is the neutron flux in the central sphere with its radius of 5 cm (“n-flux zone” in Figure 2). The optimisation of the system is done by testing different reflector-moderator materials and different enrichment of uranium target.

Figure 2. A simplified geometry (G1) of the neutron irradiator driven by an electron accelerator



See Table 1 for details.

Table 1. Neutronics model: zone dimension and material compositions for a neutron irradiator driven by an electron accelerator

Zone name	Radius (cm) $R_i - R_{i+1}$	Thickness (cm)	Material composition
<i>n-flux zone</i>	0-5	5	<i>Irrad. sample</i>
e-beam zone	5-25	20	void
u-blanket	25-27	2	enriched U (<20%)
Reflector	27-127	100	C or Pb or Be or D <sub>2</sub> O

Also see Figure 2.

As long as we use the target made of enriched uranium, surrounded by moderator-reflector,  $k_{eff}$  becomes an important parameter of the system. There are at least two points to be mentioned. Obviously, (a) higher  $k_{eff}$  is higher neutron flux in the n-flux zone will be due to the neutron multiplication factor  $\sim 1/(1-k_{eff})$ . On the other hand, (b)  $k_{eff}$  should be well below one even in accidental situations (e.g., break of the target wall and reflector filling the inner part of the sphere). We suppose that the system can operate safely with  $k_{eff} \sim 0.80$ , while  $k_{eff} \sim 0.95$  is the maximal value in the case of the inner wall failure. These two conditions have defined very precisely the maximum enrichment of the uranium target what is presented in Table 2 together with the corresponding  $k_{eff}$  values. In addition, the uranium enrichment of 20% we consider being the maximum allowed value for this type of application. Major neutronics parameters of the system with different reflector-enrichment combinations are presented in Table 3.

There is a number of important points which have to be emphasised. First of all, the presence of reflector is indispensable to reduce the neutron leakage. In addition, with a help of an effective reflector the enrichment of uranium may be decreased considerably, while still giving a desired neutron flux intensity in the irradiation zone of our interest (see column “ $\Phi_n$ ” in Table 3). Secondly, the reflector will play the role of a moderator, so one can really profit the presence of  $^{235}\text{U}$  in the target. Finally, different reflector materials will have different moderation characteristics, what results in different thermalisation level of the flux in the n-flux zone as discussed below. Figures 3-4 summarises the consequences as above.

Table 2. **Operational  $k_{\text{eff}}$  as a function of reflector materials and different uranium enrichment**

Reflector material	U enriched by (%)	$k_{\text{eff}}$
No reflector	20.0	0.26
<b>Heavy water (*)</b>	<b>0.0</b>	<b>0.14</b>
Heavy water	1.5	0.82
Beryllium	2.0	0.80
Natural carbon	5.0	0.82
Natural lead	20.0	0.67

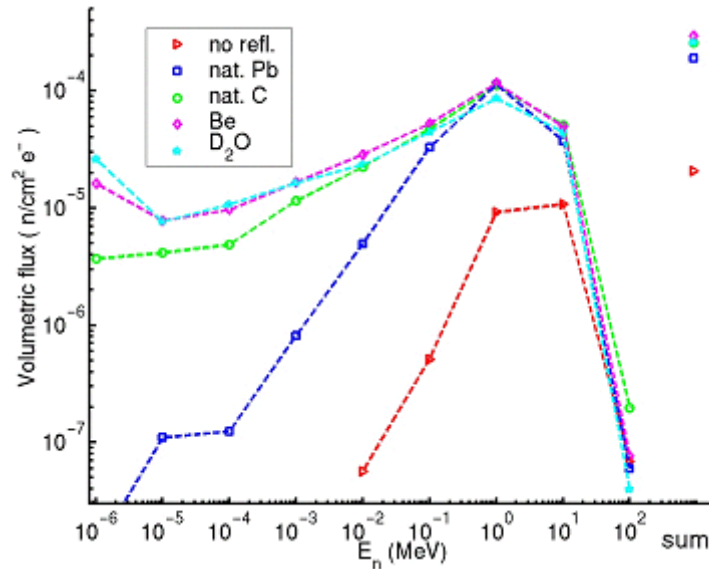
Table 3. **Neutronics of the system** (normalised per incident electron)

Reflector material	$M_n$ (%/e)	( $\gamma$ ,fiss) (%/e)	(n,fiss) (%/e)	$\Phi_n$ (n/cm <sup>2</sup> /e)	$\Phi_n$ % ( $E_n < 1$ eV)	$\Phi_n$ % ( $E_n > 1$ MeV)
<i>No reflector</i>	<i>7.1</i>	<i>1.4</i>	<i>0.7</i>	<i>2.1e-5</i>	<i>0.0</i>	<i>52.4</i>
<b>Heavy water (*)</b>	<b>8.0</b>	<b>1.3</b>	<b>0.6</b>	<b>7.0e-5</b>	<b>29.0</b>	<b>15.1</b>
Heavy water	8.0	1.3	14.4	2.6e-4	10.0	17.1
Beryllium	7.8	1.3	12.6	3.0e-4	5.4	16.6
Natural carbon	7.1	1.4	11.8	2.6e-4	1.4	19.9
Natural lead	7.7	1.4	6.6	1.9e-4	$\ll 1.0$	19.5

Note:  $M_n$  stands for neutron production efficiency per incident electron due to photo-nuclear reactions, while  $\Phi_n$  is a total volumetric flux in the “n-flux zone” of 10 cm diameter (see Table 2 for uranium enrichment values and Figure 2 for geometry). Heavy water (\*) is the case with pure  $^{238}\text{U}$ .

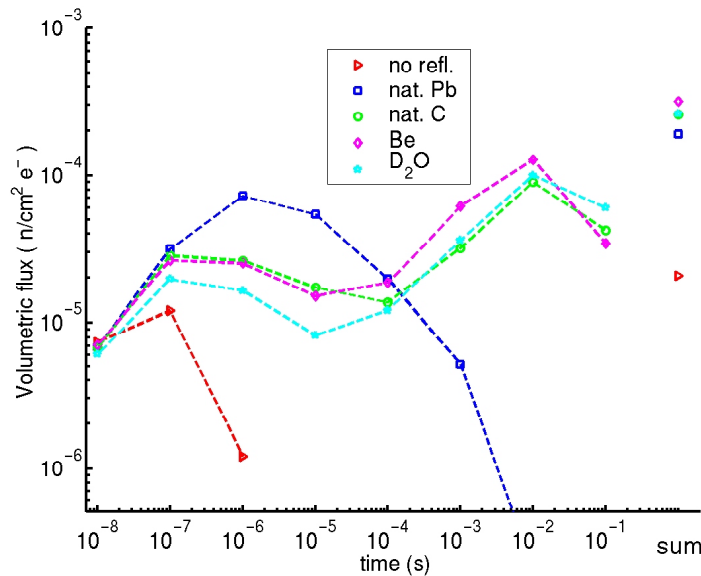
As it is shown in Figure 3, by placing more efficient reflector material one may increase considerably both the absolute value of the neutron flux (by a factor of  $\sim 15$  as well as the contribution of thermal neutrons (from  $\sim 0\%$  to  $\sim 10\%$ ). A contribution of fission neutrons due to  $^{235}\text{U}$  is easily seen from the neutron time dependence presented in the Figure 4. In the time range from  $1e-8$  to  $1e-5$  s neutrons had not enough time to be thermalised. After that (on their way back from the reflector to the target) they will not only contribute to the thermal part of the total neutron flux but also will create secondary fast fission neutrons via neutron induced fission on  $^{235}\text{U}$  (see the increment of neutron flux in the time range from  $1e-4$  to  $1e-1$  s). In this case photo-neutrons will contribute less than 20% to the total neutron flux in the n-flux zone (see Table 3 for details).

Figure 3. Neutron energy spectra as a function of different reflector material



Note: The estimated flux corresponds to the average volumetric flux in the n-flux zone. See Figure 2

Figure 4. Neutron time dependence as a function of different reflector material



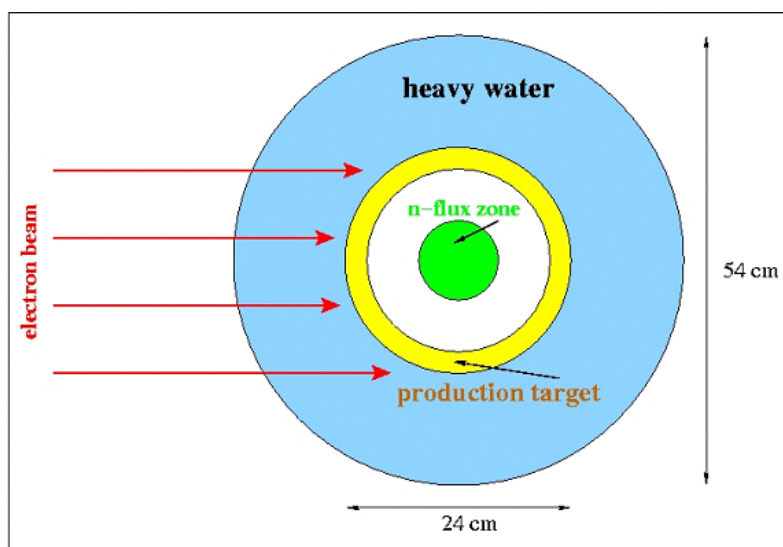
Note: The estimated flux corresponds to the average volumetric flux in the n-flux zone. See Figure 2.

### Spherical geometry G2

Similarly like in the first geometry presentation G1, here we consider a neutron production target made of either natural or enriched metallic uranium ( $\sim 19\text{g/cm}^3$  and 2 cm thick). However, now its total volume and mass is nearly by a factor 5.5 smaller, namely  $\sim 3\,050\text{ cm}^3$  and  $\sim 58\text{ kg}$  respectively. This is possible only because electrons are interacting from outside of the target. An electron beam is dispersed on the uranium sphere, so it can interact with nearly half of the actual surface of outer

envelope as shown in Figure 5. An important difference (compared to the G1 geometry) is that the beam electrons first should penetrate heavy water before they can interact with uranium. For this reason neutron production becomes not any longer linear as a function of the incident beam energy. As a matter of fact, the optimal performance will be obtained for a given thickness of the moderator at given electron energy (see Table 5 for details). Like in the previous case, our major observable is the neutron flux in the central sphere with its radius of 5 cm (“n-flux irradiation zone” in Figure 5). The characterisation of the system is done by testing different reflector-moderator thickness, enrichment of uranium target and incident electron energy.

Figure 5. A simplified geometry (G2) of the neutron irradiator driven by an electron accelerator



See Table 4 for details.

Table 4. Neutronics model: zone dimension and material compositions for a neutron irradiator driven by an electron accelerator

Zone name	Radius (cm) $R_i - R_{i+1}$	Thickness (cm)	Material composition
<i>n-flux zone</i>	0-5	5	<i>Irrad. Sample</i>
void zone	5-10	5	Void
u-blanket	10-12	2	natural or enriched U (<20%)
Reflector *	12-A; A = [12 37]	B; B = [0 25]	D <sub>2</sub> O

Also see Figure 5. Note that the thickness of heavy water zone is a parameter of optimisation.

After a number of optimisation calculations were finished, we selected five, in our opinion, the most representative cases for a more detailed analysis. The corresponding results are summarised in Table 6 and Figures 6-7. Note that in the case No 5 we have also divided all results by a factor of two. This is simply because of two times higher incident electron energy used when compared to the cases 1-4. In this way all results can be easily scaled to the same incident beam power for 100 MeV electrons.

Table 5. Neutronics of the system (normalised per incident electron) as a function of reflector thickness and incident electron energy

D <sub>2</sub> O thickness, (cm)	E <sub>e</sub> = 100 MeV		E <sub>e</sub> = 200 MeV	
	M <sub>n</sub> (%/e)	Φ <sub>n</sub> (n/cm <sup>2</sup> /e)	M <sub>n</sub> (%/e)	Φ <sub>n</sub> (n/cm <sup>2</sup> /e)
0	8.8	1.3e-4	17.0	2.5e-4
5	7.9	1.8e-4	16.8	4.0e-4
<b>10</b>	<b>7.2</b>	<b>2.0e-4</b>	16.0	4.5e-4
<b>15</b>	6.5	1.9e-4	<b>15.5</b>	<b>4.8e-4</b>
20	5.6	1.7e-4	14.7	4.7e-4
25	4.6	1.4e-4	13.5	4.4e-4

Here in all cases 2 cm thick natural uranium was used.

Table 6. Neutronics of the system (normalised per incident electron) as a function of different configurations

Configuration no and description	M <sub>n</sub> %/e	(γ,fiss) %/e	(n,fiss) %/e	Φ <sub>n</sub> n/cm <sup>2</sup> /e	Φ <sub>n</sub> % (E <sub>n</sub> <1eV)	Φ <sub>n</sub> % (E <sub>n</sub> >1MeV)
1) E <sub>e</sub> = 100 MeV; d(D <sub>2</sub> O) = 0 cm <sup>238</sup> U 100%	8.8	1.7	0.6	1.2e-4	0.0	56.0
2) E <sub>e</sub> = 100 MeV; d(D <sub>2</sub> O) = 10 cm <sup>238</sup> U 100%	7.4	1.4	0.5	2.0e-4	2.9	28.8
3) E <sub>e</sub> = 100 MeV; d(D <sub>2</sub> O) = 20 cm <sup>238</sup> U 100%	5.6	1.0	0.4	1.7e-4	10.4	24.8
4) E <sub>e</sub> = 100 MeV; d(D <sub>2</sub> O) = 20 cm <sup>238</sup> U 80% & 20% <sup>235</sup> U)	5.7	1.1	4.2	3.2e-4	1.2	29.8
5) E <sub>e</sub> = 200 MeV; d(D <sub>2</sub> O) = 15 cm <sup>238</sup> U 100%	15.5	2.9	1.1	4.8e-4	6.8	27.9
5) x 0.5 (see text for details)	7.8	1.5	0.6	2.4e-4		

Like in the case of the geometry G1 a number of important findings should be pointed out. First of all, the presence of a reflector is indispensable to reduce the neutron leakage. Although it decreases slightly (by a factor of 1.2) the primary neutron production M<sub>n</sub> (compare case No 1 and No 2 in Table 6), the total neutron flux Φ<sub>n</sub> is increased by a factor of 1.7. In addition, by varying reflector thickness one can create variable neutrons fluxes (see Figure 6), say, up to 10% thermal (case No 3). As soon as neutrons become in part thermalised, one can also profit the presence of <sup>235</sup>U in the target (see Table 6 and Figures 6-7 for the case No 4). A contribution of fission neutrons due to (n<sub>th</sub> + <sup>235</sup>U) now is easily seen from the neutron time-dependence presented in Figure 7. In the time range from 1e-8 to 1e-5 s neutrons had not enough time to be thermalised. After that (on their way back from the reflector to the target) they will not only contribute to the thermal part of the total neutron flux (cases 3 and 5) but also will create secondary fast fission neutrons (see the increment of neutron flux in the time range from 1e-4 to 1e-2 s). In this case the external source and secondary neutron production will contribute almost equally (k<sub>eff</sub> ~0.5) to the total neutron flux in the n-flux zone (see Table 6 for details). Finally, the case No 5 shows that higher electron energies are favoured to improve the system's performance. This is clearly seen by comparing the case (No 5x0.5) with the case No 2. As we mentioned earlier, higher energy is preferred since electrons still have to pass the moderator region before interacting with uranium.



Figure 6. **Neutron energy spectra as a function of different reflector material**  
 The estimated flux corresponds to the average volumetric flux in the n-flux zone. See Figure 5.

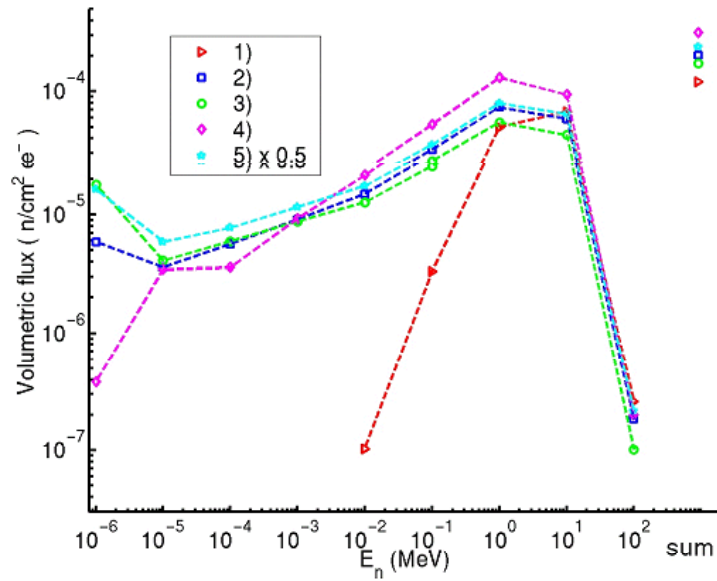
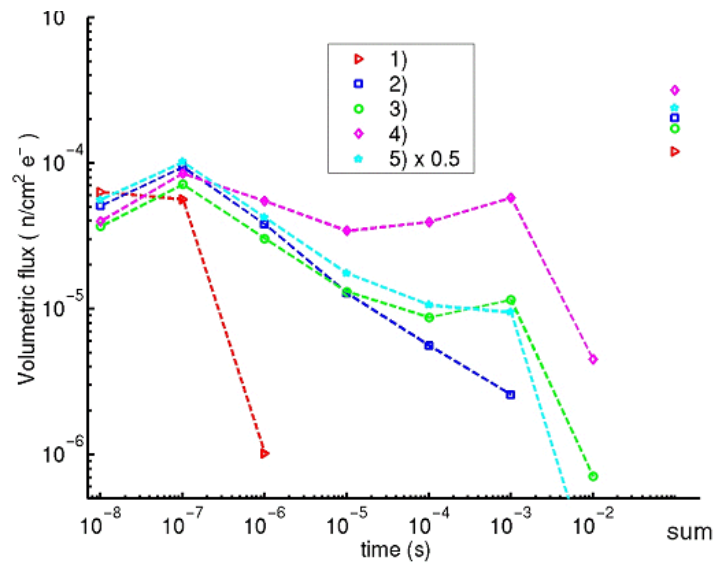


Figure 7. **Neutron time dependence as a function of different reflector material**  
 The estimated flux corresponds to the average volumetric flux in the n-flux zone. See Figure 5.



## Discussion

From Tables 3 and 6 one can conclude that the geometry G2 is more attractive than G1. Although the performance of the system is comparable in both cases, G2 is much more compact, uses less uranium (by a factor of 5.5) and not necessarily needs target enrichment by  $^{235}\text{U}$ . By changing the

thickness of heavy water neutron thermalisation level can vary from 0 to 10%, what makes the system rather flexible. In addition, some of the beam power will be dissipated directly in the D<sub>2</sub>O before even reaching the uranium target, e.g. up to 50% in the configuration No 3 (see Table 6). This fact, together with others mentioned above, is an important advantage, since the production target is supposed to withstand beam power of a few MW as discussed below.

So what are the electron accelerator requirements? Let us assume that our goal is to have a neutron flux of the order of  $10^{14}$  n s<sup>-1</sup> cm<sup>-2</sup> to be compatible with a typical experimental reactor installation. It is easy to estimate with a help of Tables 3-6 that in this case one would need an incident 100 MeV electron beam (current) of the order of ~8 MW (80mA), i.e. as many as  $\sim 5 \times 10^{17}$  pps. Working with even higher beam power might be not an easy task due to the target heat dissipation. We note that the electron target will be heated not only by the incident beam (a few MW) but also by a non negligible fission power (up to 1 MW). This results in the total average power density of, say, 0.5-1.5 kW/cm<sup>3</sup> (depending on the configuration) deposited in the uranium target. In our opinion, this value could still be tolerated but going much further might become difficult if not technologically impossible. In any case, it is also clear that this situation favours a liquid reflector-moderator, which could be used as a coolant at the same time. More detailed study on the target heating-cooling is definitely needed. In addition, one should also make an estimate for how long the target could operate in nominal operation conditions. Finally, we would like to mention that the radio-protection and non-proliferation issues should be addressed separately.

## Conclusions

A non-conventional ADS system has been proposed to produce neutrons via photo-nuclear reactions. It includes an electron accelerator and a spherical natural or enriched uranium target-blanket (2 cm thick) surrounded by a reflecting material. The system is well sub-critical ( $k_{\text{eff}} < 0.8$ ) and uranium enrichment is below 20%. Very encouraging preliminary calculations have been performed using the MCNP code enhanced with a photo-nuclear capability. It is shown that variable (from 0 to 10% thermal) neutron fluxes of a few  $10^{14}$  n s<sup>-1</sup> cm<sup>-2</sup> could be obtained for different irradiation purposes. Nearly 30% of these neutrons have energies higher than 1 MeV. The electron machine at ~8 MW power and 100 MeV incident energy should be sufficient to produce external neutrons to drive the system. This results in the total average power density of ~1.5 kW/cm<sup>3</sup> deposited in the target, perhaps being the limiting factor. More detailed calculations related to safety and heat dissipation issues are in progress and will be reported elsewhere.

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