

# **Environmental and Public Neutron Exposure from UF<sub>6</sub>**

*By*

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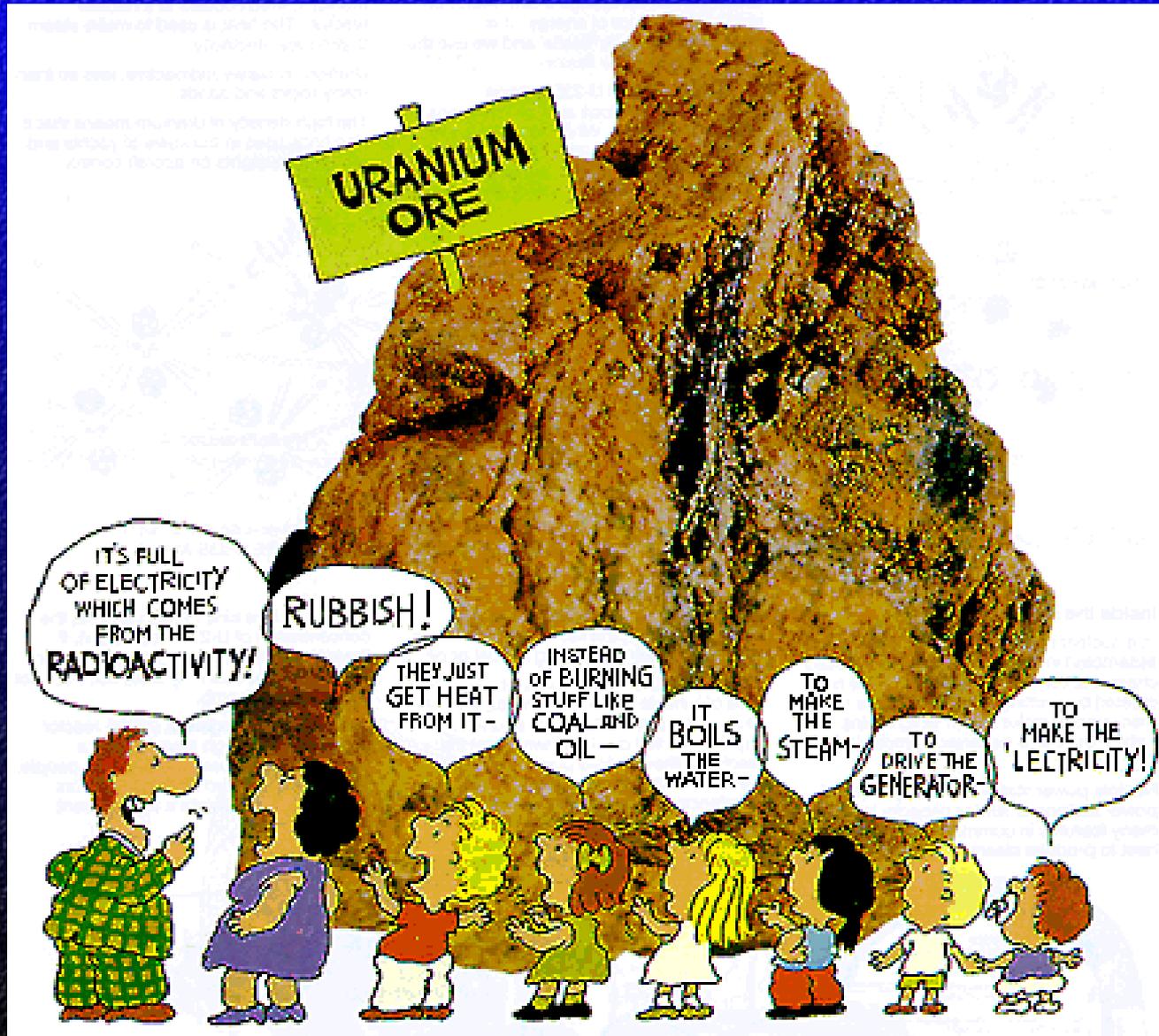
**Cameco Corporation**

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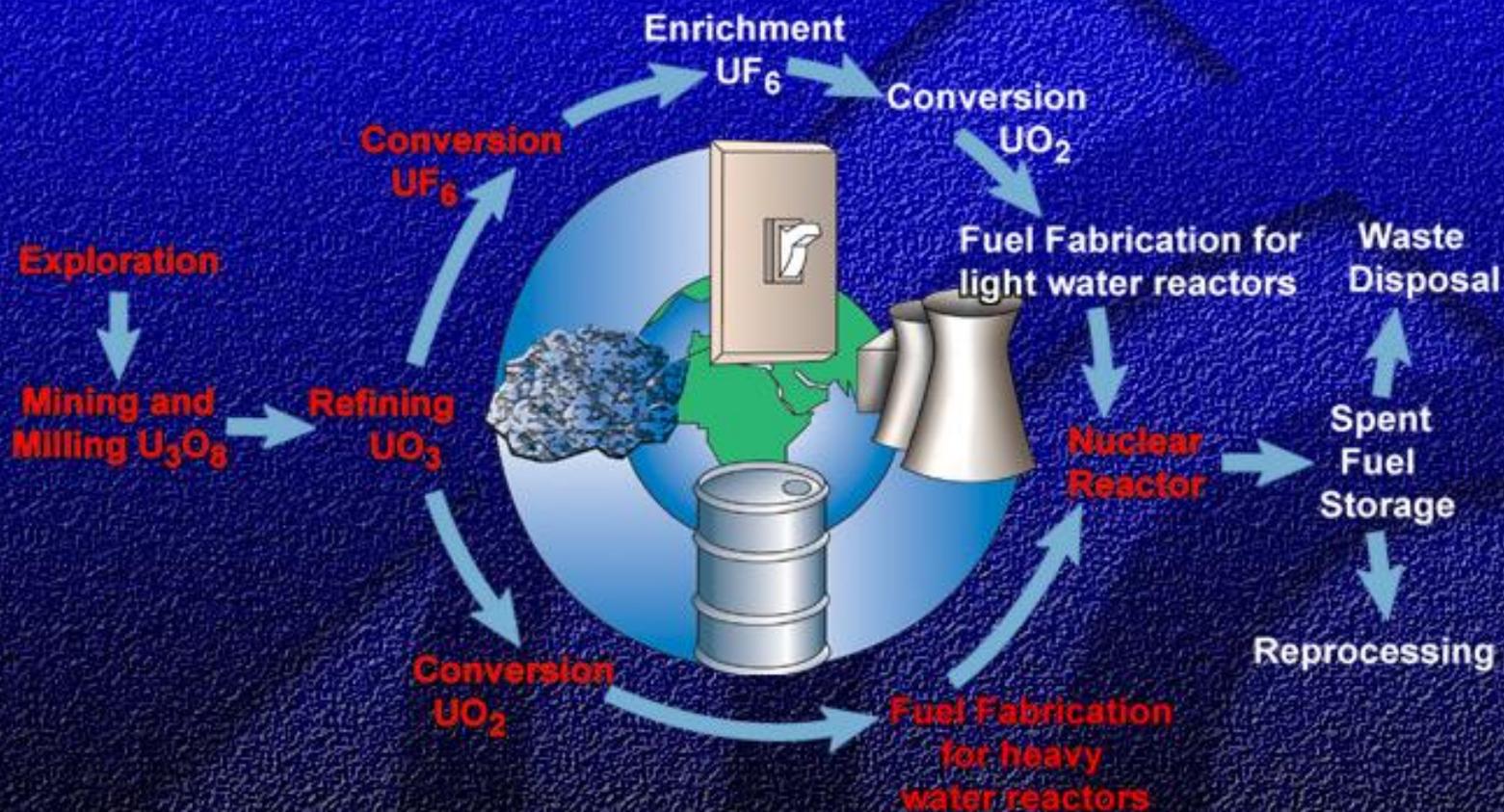
**October 16 to 18, 2007 Wilmington NC**

**Fuel Cycle Safety – Past, Present and Future**

# What is Uranium?



# Nuclear Fuel Cycle



Carried on by Cameco and others  
Carried on by others

# Port Hope Radium Refinery



# Port Hope conversion facility



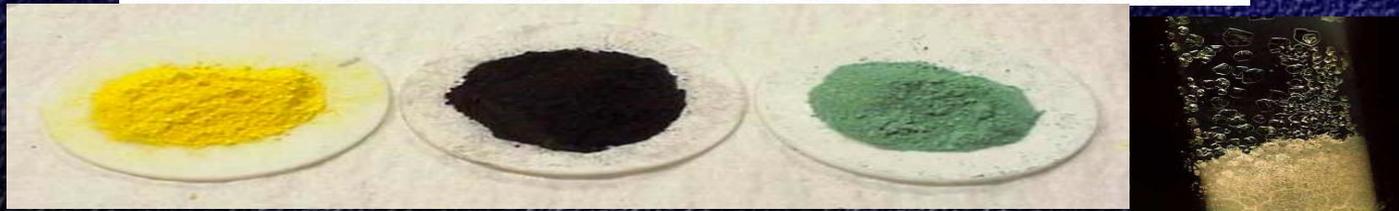
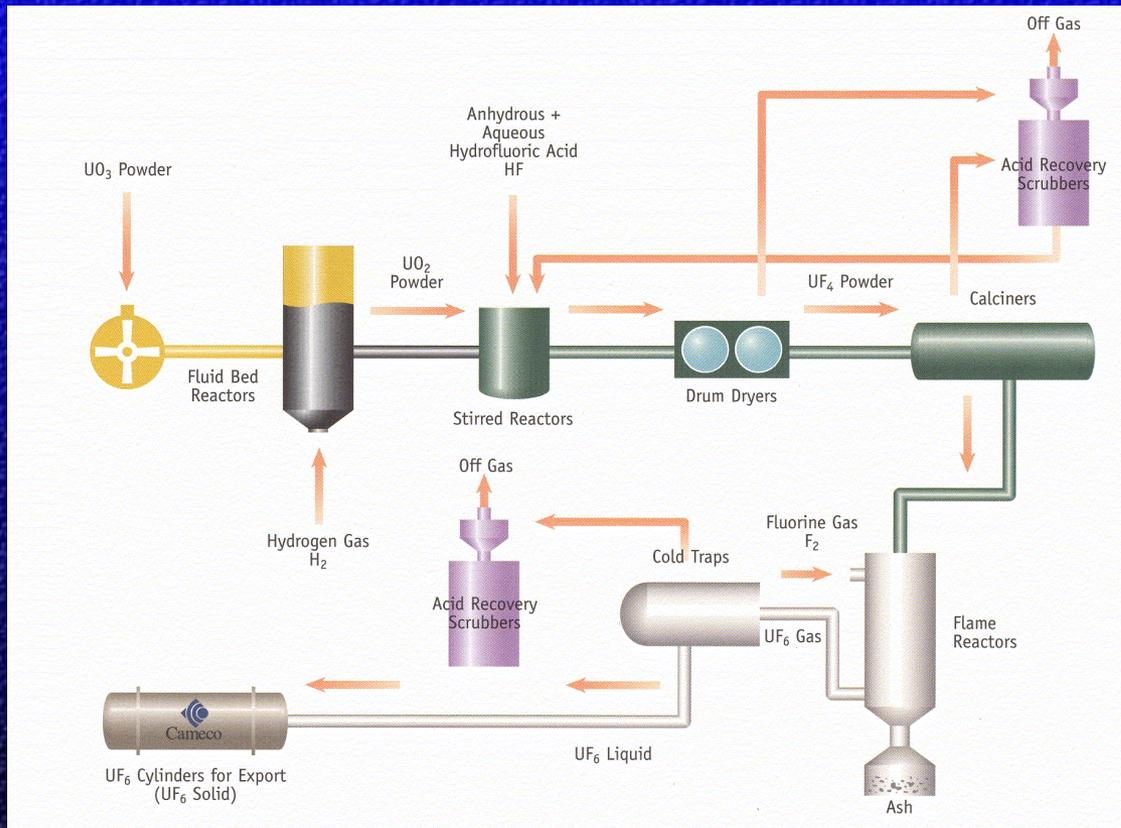
Building 50 – UF<sub>6</sub> Plant



# Cameco UF<sub>6</sub> conversion facility



# UF<sub>6</sub> Conversion Process



# Solid $\text{UF}_6$











# UF<sub>6</sub> Cylinder Transport Canada to USA



Figure 1. Photograph of open trailer and truck loaded with 2000 48X cylinders for transport to the USA.

# Neutron Measurements at PHCF



- ◆ Neutrons are generated at a low rate from spontaneous fission of uranium and at significantly higher rates from uranium-fluorine compounds through the reaction  $^{19}\text{F} + \alpha \rightarrow \text{n} + ^{22}\text{Na}$ .
- ◆ Laboratory measurements and theoretical calculations indicate that the dose rate due to neutrons around  $\text{UF}_6$  cylinders could be in the range of 0.5 to 2  $\mu\text{Sv/h}$ . These values suggest the doses to workers in the  $\text{UF}_6$  plant should be less than 1  $\text{mSv/y}$  because only a small fraction of a typical work year is spent in close proximity to  $\text{UF}_6$  cylinders.

# Neutron Measurements at PHCF



- ◆ To determine the validity of these studies, as they apply to the situation at Port Hope, measurements were conducted with a neutron survey meter.
- ◆ Neutron fields near large sources of  $UF_6$  were easily measurable, but were less than  $0.5 \mu\text{Sv/h}$  in virtually every work area assessed.
- ◆ This means that doses to workers from neutrons are much less than  $1 \text{ mSv/y}$ .

# Neutron Measurements at PHCF



- ◆ A neutron survey was conducted on December 14, 2000 with an ADM-300C Gamma/Neutron survey meter owned by Ontario Power Generation (OPG).
- ◆ The meter has energy compensated response to neutrons that measures in units of  $\mu\text{Sv}$ . The ADM meter can operate as either a rate meter or in a dose integration mode.
- ◆ The unit had to be used in dose integration mode and kept still for 4 minutes during the measurement period.

# Neutron Measurements at PHCF



- ◆ Priority was given to locations with large  $UF_6$  sources nearby because  $UF_6$  is the strongest source of neutrons.
- ◆ Contact measurements were done on large uranium-fluorine sources (e.g.,  $UF_6$  cylinders) to determine the maximum neutron dose rates at the facility; these measurements were called engineering locations.

# Neutron Measurements at PHCF



- ◆ **High-occupancy work areas in the general vicinity of large  $UF_6$  sources were also measured to give a realistic estimate of dose rates in these important locations; these measurements were called workplace locations.**
- ◆ **In addition, measurements were taken at some of the routine TLD stations along the fence line to determine if there is any measurable dose to the public; these measurements were called fence line locations.**

# Neutron Measurements at PHCF



## Neutron Dose Rate $\mu\text{Sv/h}$

Location Type	No.	Average	Minimum	Maximum
Engineering	7	1.51	0.05	3.00
Workplace	13	0.14	0.00	0.56
Fence line	8	0.03	0.00	0.05

# Neutron Measurements at PHCF



- ◆ As anticipated, the engineering samples had the highest results and in the approximate range expected based on the literature review of the topic.
- ◆ Contact readings on  $\text{UF}_6$  cylinders ranged between 1.5 to 3.0  $\mu\text{Sv/h}$  and one reading on top of the head of the primary filter in the second floor flame reactor area had a value of 0.2  $\mu\text{Sv/h}$ .
- ◆ Lying on a full  $\text{UF}_6$  cylinder would give a neutron dose rate of about 2  $\mu\text{Sv/h}$ , while the dose rate for a worker standing between two  $\text{UF}_6$  cylinders, which one would hope to be a more realistic exposure scenario, would be about 1  $\mu\text{Sv/h}$ .

# Neutron Measurements at PHCF



- ◆ In typical workplace locations the neutron dose rate ranged from non-detectable to about  $0.6 \mu\text{Sv/h}$ , with an average value of  $0.1 \mu\text{Sv/h}$ .
- ◆ Even close to  $\text{UF}_6$  cylinders the dose rate was low; for example, 2 m from a row of 15 full cylinders the dose rate was only  $0.4 \mu\text{Sv/h}$ .
- ◆ None of the control rooms measured had any appreciable neutron fields. With maximum dose rates at  $1 \mu\text{Sv/h}$  and more typical locations having an average about  $0.1 \mu\text{Sv/h}$ , the survey shows that doses to workers from neutrons should be less than  $1 \text{mSv/y}$ .

# Neutron Measurements at PHCF



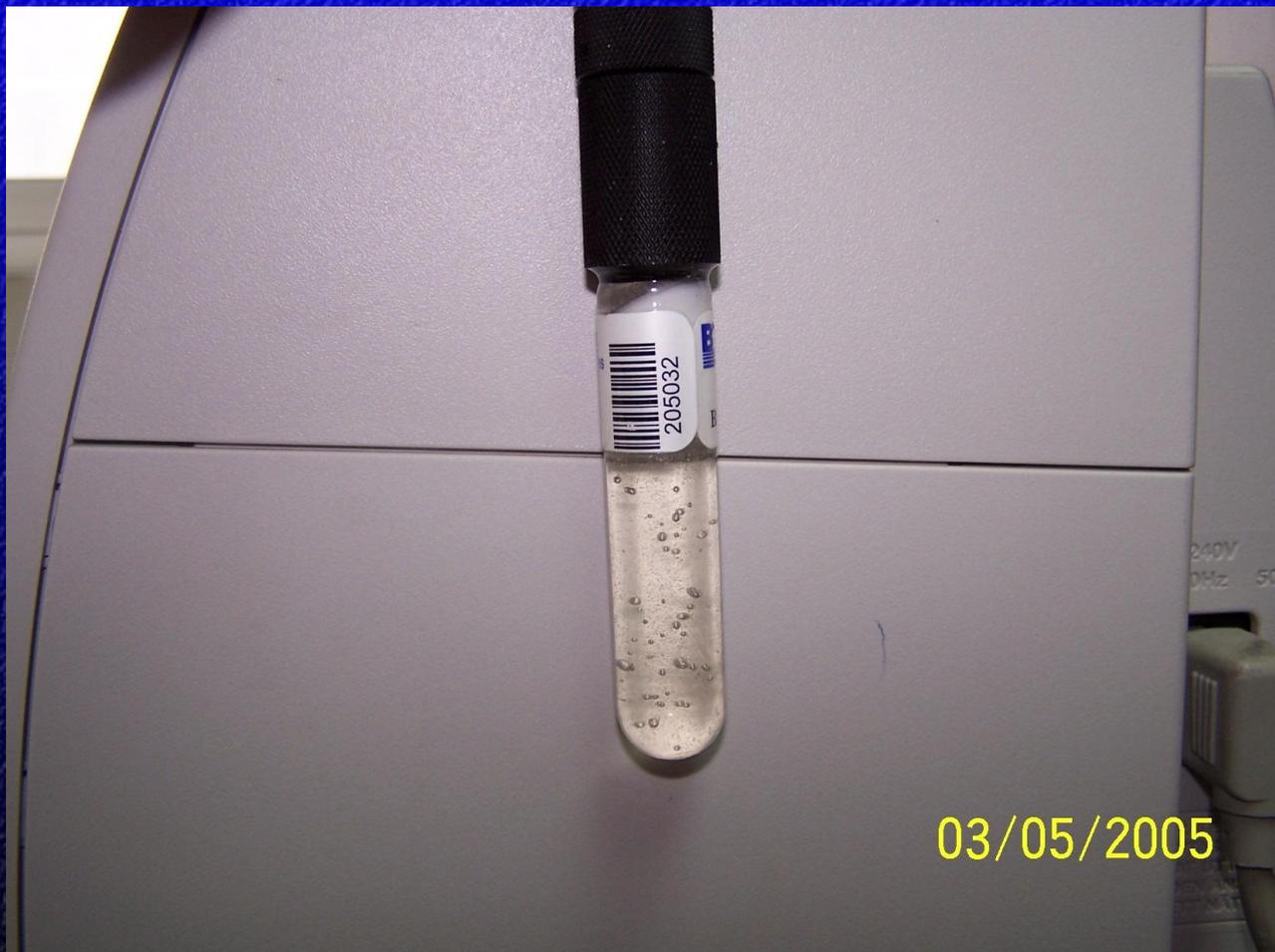
- ◆ **The fence line monitoring detected a low-level neutron field around the UF<sub>6</sub> plant. Neutron fields are less than the gamma fields at these locations, but possibly a substantial fraction of the gamma dose rate.**
- ◆ **Neutron Bubble Detector manufactured by Bubble Technologies Industries (BTI) used for environmental monitoring of neutrons has been recommended by Cameco's Health Physicist .**

# BTI – Bubble Detectors

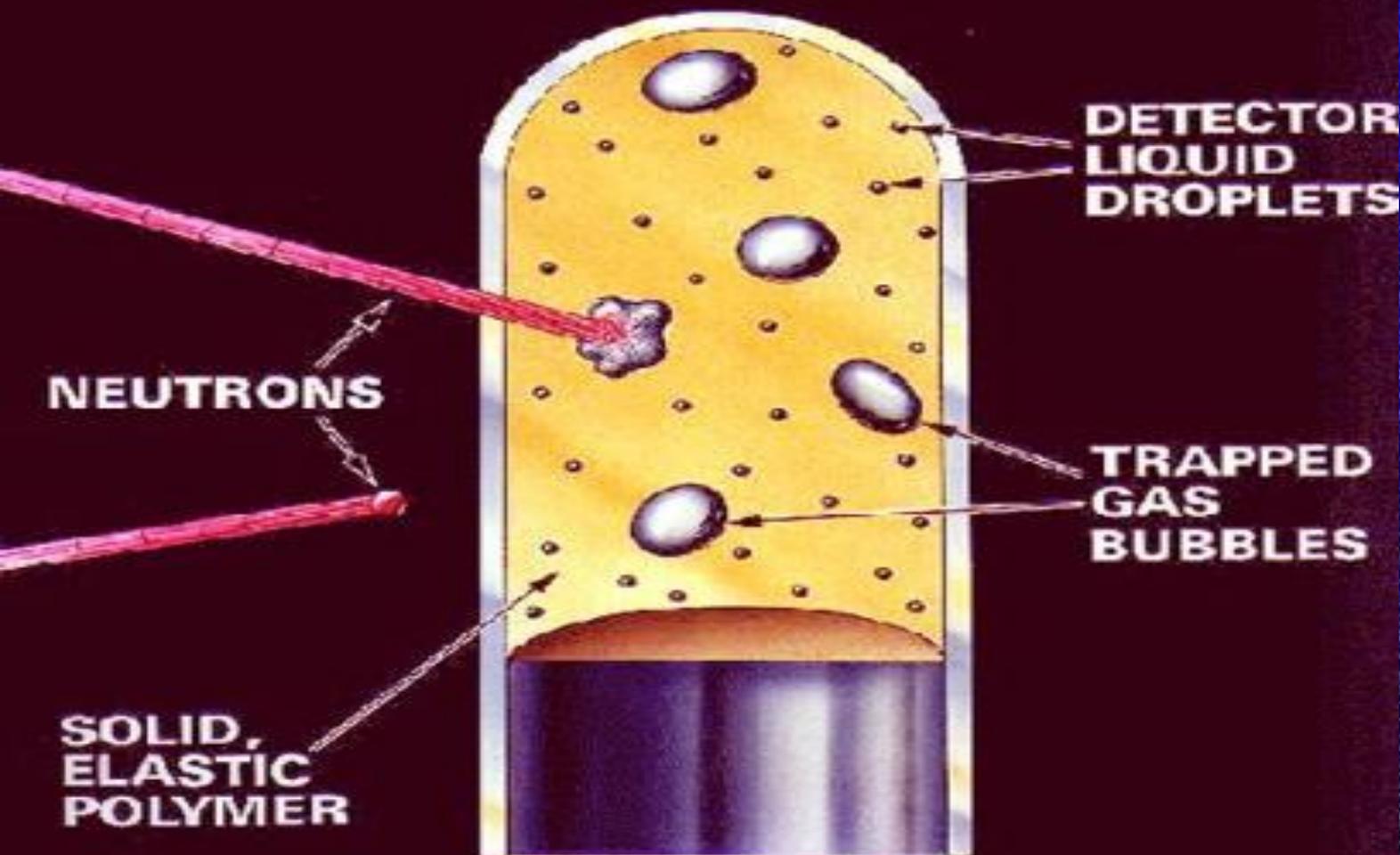


- ◆ **Bubble tube dosimeters from (BTI) a dynamic company with world-renowned expertise in the field of radiation detection. These dosimeters have a detection limit of  $0.01\mu\text{Sv}$ .**
- ◆ **Neutron Bubble Detectors are reusable, integrating, passive dosimeters that allow instant visible detection of neutron radiation.**
- ◆ **Bubble Detectors consist of minute droplets of a superheated liquid dispersed throughout an elastic polymer. When neutrons strike these droplets they form small gas bubbles that remain fixed in the polymer, providing a real time, immediate visual record of the dose.**

# BTI – Bubble Detectors



# HOW A BUBBLE DETECTOR WORKS



# Neutron Measurements at PHCF



- ◆ Four fence line locations were selected for the Bubble technology trial in 2001.

Location	Number of trials	Highest results ( $\mu\text{Sv/h}$ )
South of UF6 Plant	5	0.062
West of UF6 Plant	5	0.080
North of UF6 Plant	5	0.230
Critical Receptor at Mill Street	5	0.030

# Neutron Measurements at PHCF



- ◆ Neutron radiation studies have been conducted at the PHCF in 2005 to confirm that this source of radiation does not pose a health risk to employees. For these trials Cameco have used two different devices to measure neutron radiation.
- ◆ Neutron dosimeters from Landauer Inc. These dosimeters have a minimum detection level of 100  $\mu\text{Sv}$ .
- ◆ Bubble tube dosimeters from (BTI). These dosimeters have a detection limit of 0.01 $\mu\text{Sv}$ .

# Neutron Measurements at PHCF



## Personal monitoring

- ◆ **April and May 2005 - Employees from UF<sub>6</sub> plant, UO<sub>2</sub> plant and Materials handling wore a special Landauer CR-39 fast neutron dosimeter designed to neutron radiation exposure**
- ◆ **These three groups represent a broad cross section of employees at our site and will provide the data we need to assess the potential impact of neutron radiation on our entire work force.**
- ◆ **Results were below the detection level of 200 μSv.**

# Neutron Measurements at PHCF



**CR-39**  
Fast/Intermediate/  
Thermal Neutron

# Neutron Measurements at PHCF



- ◆ **The Neutrak 144 detector is a CR-39 (allyl diglycol carbonate) based, solid-state nuclear track detector that is not sensitive to x, beta or gamma radiation, and can be packaged specifically for neutron detection only.**
- ◆ **The CR-39 is laser engraved for permanent identification, and wrapped with a 2-D bar code to assure efficient chain-of-custody.**

# Neutron Measurements at PHCF



- ◆ Track Etch Technology consists of etching the CR-39 for 15 hours in a chemical bath to enlarge exposure tracks.
- ◆ The fast neutron dose is measured by counting the tracks generated as a result of the proton recoil with the polyethylene radiator, while the thermal/intermediate dose is measured by counting the alpha tracks generated with the boron radiator.

# Neutron Measurements at PHCF



- ◆ On April 1, 2005, thirty-three Landauer fast neutron dosimeters were deployed along Port Hope Conversion Facility fence line. After 30 days of exposure dosimeters show results below the minimum detection level of 100  $\mu$ Sv.
- ◆ A second trial has been initiated on December 1, 2006 and detectors are to be collected on December 1, 2007.

# Landauer – Neutrak Detector



CR-39  
Fast/Intermediate/  
Thermal Neutron dosimeter

# Landauer – Neutrak Detector



# BTI – Bubble Detectors



- ◆ **2005-2006 - An extensive study of neutron radiation was conducted at the UF<sub>6</sub> operation plant using BTI neutron bubble detectors.**
- ◆ **The average neutron radiation dose inside the UF<sub>6</sub> plant was less than 0.3 μSv/h.**
- ◆ **The operators in the cylinder lay down area logged the highest dose rate at 0.12 μSv/h.**
- ◆ **All other operators showed lower dose rates.**

# BTI – Bubble Detectors



- ◆ The study also looked at neutron dose to the public and the environment. The maximum neutron dose along the fence line was 0.28  $\mu\text{Sv/h}$ .
- ◆ The studies concluded that neutron radiation at the PHCF represents a small fraction of the total whole body dose received by employees and therefore does not pose a risk to employees.
- ◆ Cameco will continue to use these devices to demonstrate that neutron radiation levels at the facility remain constant.

**The End**