

The logo for IRSN, featuring the letters 'IRSN' in a bold, sans-serif font. The 'I' and 'R' are red, the 'S' is blue, and the 'N' is red.

INSTITUT
DE RADIOPROTECTION
ET DE SÛRETÉ NUCLÉAIRE

Nuclear Data Activities at IRSN

NJOY User Meeting - November 26, 2007

Wim Haeck, IRSN, wim.haeck@irsn.fr

Joachim Miss, IRSN

Nuclear data activities at IRSN

- No “pure” nuclear data but activities with strong ties to nuclear data
- **Experimental programs: data & code validation**
 - Fission product experimental program
 - Plutonium temperature effect experimental program
 - Structural materials experimental program
- **Development of calculation tools: data related developments**
 - CRISTAL: criticality safety package
 - MORET: Monte Carlo calculations
 - VESTA: a generic interface for depletion calculations

MORET

- **Multi-group/continuous energy Monte Carlo for criticality and reactor calculations**
- **The distributed multi-group version: MORET 4**
 - 3D modular combinatorial geometry
 - Coupled with APOLLO2 deterministic code self-shielding and homogenization of cells in the CRISTAL standard calculation route
 - Source convergence, simulation strategies and weakly coupled systems
- **The future: MORET5**
 - Continuous energy mode (beta version at the end of 2007)
 - Woodcock tracking
 - Cross section libraries from any evaluation
 - Compatibility with various deterministic cell codes for the multi-group mode
- **Nuclear data needs**
 - Neutron transport data
 - Various formats: multi-group & continuous energy

VESTA

- **A generic Monte Carlo evolution interface**
 - Provide the normal MC accuracy for the smallest computing effort possible
 - Creating a generic interface for Monte Carlo depletion calculations
 - For now limited to MCNP(X) and a modified version of ORIGEN 2.2
 - Efficient, flexible and easy to use
 - A first version foreseen in 2008

- **VESTA has very specific nuclear data needs like any depletion code**
 - Cross section data read from ENDF files generated by NJOY which are also used by MCNP(X)
 - ENDF libraries contain additional data needed for depletion:
 - Fission yield data and radio-active decay data
 - Branching ratios for the production of isomeric states
 - Fission Q-value data

C++ software for nuclear data manipulation: ENDF++

- Under developed and investigation in the framework of our main activities
- All the advantages of object oriented design
 - Modularity, generality and reusability
 - Optimisation and maintenance
 - They can function as a standalone application or can be easily integrated into other dedicated applications
 - High level design with low level interface
- ENDF++ is currently:
 - An ENDF parser to read/format/write nuclear data
 - A number of function objects or functors:
 - Linearisation of point wise data
 - Integration of linearised functions
 - Unionisation of linearised functions

Parsing ENDF files with ENDF++

- **Read, write and extract ENDF data**
 - MF1: fission neutron yield, fission Q-values, etc.
 - MF2: resonance data
 - MF3/MF9/MF10/MF23: cross section and multiplicity data
 - MF7: thermal neutron scattering data
 - MF8: radioactive decay data, fission yield data, etc.

- **Format nuclear data in the ENDF format**
 - MF1: fission neutron yield, fission Q-values, etc.
 - MF3/MF9/MF10/MF23: cross section and multiplicity data
 - MF7 MT4: incoherent inelastic scattering data - $S(\alpha, \beta)$ data

- **Easily extendable to other data contained in the ENDF file**

Parsing ENDF files with ENDF++

- The parser is currently used inside VESTA, to read the required nuclear data for a depletion calculation
- Future applications for this parser will include:
 - Quality Assurance for basic evaluations
 - Test for formatting errors
 - Test for inconsistencies between various sub-libraries
 - Help in automated NJOY processing
 - Checks important parameters for nuclide tailored NJOY input
 - E.g. the temperature, whether or not the nuclide has resolved/unresolved resonances, etc.
 - Quality Assurance for processed evaluations
 - ENDF format is used by NJOY to pass on data between modules
 - Provide testing for undetected reconstruction errors (e.g. JEFF 3.1 ⁵⁸Co capture)
 - Merging data together into a single ENDF tape
 - Etc.

Functors within ENDF++

■ Integrating linearised data

$$f_g = \int_{E_{g-1}}^{E_g} f(E) dE \qquad f_g^h = \int_{E_{g-1}}^{E_g} h(E) f(E) dE$$

- For simple integrals
- For calculating group cross sections
- For weighted integrals, like for instance isomeric production branching ratios

■ Linearise data

- Generate linearised data within a given tolerance
- NJOY does not linearise and reconstruct all data from an ENDF file
- For example: MF9 - multiplicities for radio-active isotope production

Functors within ENDF++

- **Functors can be developed independently with relative ease in little time**
 - Tasks should be split up as much as possible
 - A functor that integrates data should not have to linearise it
 - The remaining tasks are thus very specific
 - Therefore easy to implement
 - Anybody can contribute!
- **Functors should have a basic and simple interface**
 - Only low level data is manipulated: integers, real values, arrays, etc.
 - No “advanced” concepts
- **Great optimisation potential**
 - Multiple implementations are possible and the best can be chosen
 - Gains the potential of automatic validation

Other R&D topics in the near and far future

- The continuous energy version of MORET will use a “universal” energy grid
 - To avoid exploding the resulting energy grid, we will need to look into energy grid thinning
 - Study the different algorithms that can be used
 - Implement them for use in MORET 5
- MORET 5 will use probability tables in the unresolved resonance region
 - This is currently limited to probability tables from NJOY
- These topics could all become basic functors in ENDF++

Conclusion and outlook

- ENDF++ provides us with a software framework:
 - For nuclear data testing and Quality Assurance
 - For easy access to data in the ENDF format
 - For basic operations on that data

- Ultimately, ENDF++ will contain all the basic building blocks for a more complicated task like cross section reconstruction:
 - Reading and extracting the basic data
 - Linearising the data
 - Thinning the resulting energy grid
 - Reconstructing resonance data

- We are open for business and are willing to collaborate with all of you!